

April 10, 1986

Docket No. 50-373

Mr. Dennis L. Farrar  
Director of Licensing  
Commonwealth Edison Company  
P.O. Box 767  
Chicago, Illinois 60690

Dear Mr. Farrar:

Subject: Corrected Page to the Technical Specifications for  
Amendment No. 36 of La Salle County Station, Unit 1

Our letter dated March 13, 1986, transmitted Amendment No. 36 for La Salle County Station, Unit 1 addressing the change to the Main Steam Line Low Pressure instrument response time, from 1 to 2 seconds on Table 3.3.2-3. In our March 13th transmittal, an inadvertent error was made in that the Reactor Vessel Water Level, Low Level instrument response time was changed from 1 to 2 seconds on Table 3.3.2-3.

Please replace the Technical Specification page 3/4 3-18 with the enclosed corrected page.

We regret any inconvenience caused by this error.

Sincerely,

/s/

Elinor G. Adensam, Director  
BWR Project Directorate No. 3  
Division of BWR Licensing

Enclosure:  
As stated

cc: See next page

OB  
BWD-3:DBL  
ABournia/hmc  
4/9/86

LA BWD-3:DBL  
EHyton  
4/9/86

D: BWD-3:DRL  
EAdensam  
4/10/86

8604160326 860410  
PDR ADOCK 05000373  
P PDR

Mr. Dennis L. Farrar  
Commonwealth Edison Company

La Salle County Nuclear Power Station  
Units 1 & 2

cc:  
Philip P. Steptoe, Esquire  
Suite 4200  
One First National Plaza  
Chicago, Illinois 60603

John W. McCaffrey  
Chief, Public Utilities Division  
160 North La Salle Street, Room 900  
Chicago, Illinois 60601

Assistant Attorney General  
188 West Randolph Street  
Suite 2315  
Chicago, Illinois 60601

Resident Inspector/LaSalle, NPS  
U.S. Nuclear Regulatory Commission  
Rural Route No. 1  
Post Office Box 224  
Marseilles, Illinois 61341

Chairman  
La Salle County Board of Supervisors  
La Salle County Courthouse  
Ottawa, Illinois 61350

Attorney General  
500 South 2nd Street  
Springfield, Illinois 62701

Chairman  
Illinois Commerce Commission  
Leland Building  
527 East Capitol Avenue  
Springfield, Illinois 62706

Mr. Gary N. Wright, Manager  
Nuclear Facility Safety  
Illinois Department of Nuclear Safety  
1035 Outer Park Drive, 5th Floor  
Springfield, Illinois 62704

Regional Administrator, Region III  
U. S. Nuclear Regulatory Commission  
799 Roosevelt Road  
Glen Ellyn, Illinois 60137

TABLE 3.3.2-3

ISOLATION SYSTEM INSTRUMENTATION RESPONSE TIME

<u>TRIP FUNCTION</u>	<u>RESPONSE TIME (Seconds)#</u>
<u>A. AUTOMATIC INITIATION</u>	
<u>1. PRIMARY CONTAINMENT ISOLATION</u>	
a. Reactor Vessel Water Level	NA
1) Low, Level 3	$< 1.0^* / < 13^{(a)**}$
2) Low Low, Level 2	$< 13^{(a)**}$
b. Drywell Pressure - High	$< 13^{(a)**}$
c. Main Steam Line	
1) Radiation - High <sup>(b)</sup>	$< 1.0^* / < 13^{(a)**}$
2) Pressure - Low	$< 2.0^* / < 13^{(a)**}$
3) Flow - High	$< 0.5^* / < 13^{(a)**}$
d. Main Steam Line Tunnel Temperature - High	NA
e. Condenser Vacuum - Low	NA
f. Main Steam Line Tunnel $\Delta$ Temperature - High	NA
<u>2. SECONDARY CONTAINMENT ISOLATION</u>	
a. Reactor Building Vent Exhaust Plenum Radiation - High <sup>(b)</sup>	$< 13^{(a)}$
b. Drywell Pressure - High	$< 13^{(a)}$
c. Reactor Vessel Water Level - Low, Level <sup>(2)</sup>	$< 13^{(a)}$
d. Fuel Pool Vent Exhaust Radiation - High <sup>(b)</sup>	$< 13^{(a)}$
<u>3. REACTOR WATER CLEANUP SYSTEM ISOLATION</u>	
a. $\Delta$ Flow - High	$< 13^{(a)##}$
b. Heat Exchanger Area Temperature - High	NA
c. Heat Exchanger Area Ventilation $\Delta T$ -High	NA
d. SLCS Initiation	NA
e. Reactor Vessel Water Level - Low Low, Level 2	$< 13^{(a)}$
<u>4. REACTOR CORE ISOLATION COOLING SYSTEM ISOLATION</u>	
a. RCIC Steam Line Flow - High	$< 13^{(a)###}$
b. RCIC Steam Supply Pressure - Low	$< 13^{(a)}$
c. RCIC Turbine Exhaust Diaphragm Pressure - High	NA
d. RCIC Equipment Room Temperature - High	NA
e. RCIC Steam Line Tunnel Temperature - High	NA
f. RCIC Steam Line Tunnel $\Delta$ Temperature - High	NA
g. Drywell Pressure - High	NA
h. RCIC Equipment Room $\Delta$ Temperature - High	NA
<u>5. RHR SYSTEM STEAM CONDENSING MODE ISOLATION</u>	
a. RHR Equipment Area $\Delta$ Temperature - High	NA
b. RHR Area Cooler Temperature - High	NA
c. RHR Heat Exchanger Steam Supply Flow High	NA

B604160335 860410  
PDR ADDOCK 05000373  
P PDR

April 10, 1986

DISTRIBUTION:

~~Docket No.~~ 50-373  
NRC PDR  
Local PDR  
PRC System  
NSIC  
BWD-3 r/f  
ABournia (4)  
EHylton (2)  
EAdensam  
Attorney, OELD  
CMiles  
RDiggs  
JPartlow  
BGrimes  
EJordan  
LHarmon  
TBarnhart (4)