UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

APR 11 1986

Docket Nos: 50-373
and 50-374

Mr. Dennis L. Farrar
Director of Licensing
Commonwealth Edison Company
P.O. Box 767
Chicago, Illinois 60690

Dear Mr. Farrar:

Subject: Issuance of Amendment No. 38 to Facility Operating License No. NPF-11 and Amendment No. 20 to Facility Operating License No. NPF-18 - La Salle County Station, Units 1 and 2

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 38 to Facility Operating License No. NPF-11 and Amendment No. 20 to Facility Operating License No. NPF-18 for the La Salle County Station, Units 1 and 2. These amendments are in response to your letter dated February 7, 1986, as supplemented by letter dated March 5, 1986.

The amendments revise the La Salle Units 1 and 2 Technical Specifications to eliminate the chlorine detector monitoring instrument system because of recent surveys indicating that chlorine is not and has not been shipped in bulk quantities by highway, railroad, or river near the La Salle County Station.

A copy of the related safety evaluation supporting Amendment No. 38 to Facility Operating License No. NPF-11 and Amendment No. 20 to Facility Operating License NPF-18 is enclosed.

Sincerely,

Elinor G. Adensam

Elinor G. Adensam, Director
BWR Project Directorate No. 3
Division of BWR Licensing

Enclosures:
1. Amendment No. 38 to NPF-11
2. Amendment No. 20 to NPF-18
3. Safety Evaluation

DESIGNATED ORIGINAL

Certified By

cc w/enclosure:
See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-373

LA SALLE COUNTY STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 38
License No. NPF-11

1. The Nuclear Regulatory Commission (the Commission or the NRC) having found that:

   A. The application for amendment filed by the Commonwealth Edison Company (the licensee), dated February 7, 1986, as supplemented by letter dated March 5, 1986, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;

   C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the enclosure to this license amendment and paragraph 2.C.(2) of the Facility Operating License No. NPF-11 is hereby amended to read as follows:

   (2) Technical Specifications and Environmental Protection Plan

   The Technical Specifications contained in Appendix A, as revised through Amendment No. 38, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This amendment is effective as of date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Elinor G. Adensam

Elinor G. Adensam, Director
BWR Project Directorate No. 3
Division of BWR Licensing

Enclosure:
Changes to the Technical
Specifications

Date of Issuance: APR 1 : [illegible]

ENCLOSURE TO LICENSE AMENDMENT NO. 38

FACILITY OPERATING LICENSE NO. NPF-11

DOCKET NO. 50-373

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain a vertical line indicating the area of change.

REMOVE	INSERT
V	V
3/4 3-74	3/4 3-74
B3/4 3-5	B3/4 3-5
3/4 7-5	3/4 7-5

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LIMITING CONDITIONS FOR OPERATION AND SURVEILLANCE REQUIREMENTS

INSTRUMENTATION

AMMONIA DETECTION SYSTEM

LIMITING CONDITION FOR OPERATION

3.3.7.8 Two independent ammonia detection system subsystems shall be OPERABLE,* each with two ammonia detectors, with their alarm/trip setpoints adjusted to actuate at an ammonia concentration of less than or equal to 25 ppm.

APPLICABILITY: All OPERATIONAL CONDITIONS.

ACTION:

a. With one ammonia detector in either detection subsystem inoperable, restore the inoperable detector(s) to OPERABLE status within 7 days or, within the next 6 hours, initiate and maintain operation of at least one control room charcoal filter system train in the recirculation mode of operation.

b. With both ammonia detection subsystems inoperable, within 1 hour initiate and maintain operation of at least one control room charcoal filter system train in the recirculation mode of operation.

c. The provisions of Specification 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

4.3.7.8 Each of the above required ammonia detection system subsystems shall be demonstrated OPERABLE by performance of a CHANNEL FUNCTIONAL TEST at least once per 31 days and a CHANNEL CALIBRATION at least once per 18 months.

*The normal or emergency power source may be inoperable in OPERATIONAL CONDITION 4 or 5.

MONITORING INSTRUMENTATION (Continued)

3/4.3.7.5 ACCIDENT MONITORING INSTRUMENTATION

The OPERABILITY of the accident monitoring instrumentation ensures that sufficient information is available on selected plant parameters to monitor and assess important variables following an accident. This capability is consistent with the recommendations of Regulatory Guide 1.97, "Instrumentation for Light Water Cooled Nuclear Power Plants to Assess Plant Conditions During and Following an Accident," December 1975 and NUREG-0578, "TMI-2 Lessons Learned Task Force Status Report and Short-Term Recommendations."

3/4.3.7.6 SOURCE RANGE MONITORS

The source range monitors provide the operator with information of the status of the neutron level in the core at very low power levels during startup and shutdown. At these power levels, reactivity additions should not be made without this flux level information available to the operator. When the intermediate range monitors are on scale adequate information is available without the SRMs and they can be retracted.

3/4.3.7.7 TRAVERSING IN-CORE PROBE SYSTEM

The OPERABILITY of the traversing in-core probe system with the specified minimum complement of equipment ensures that the measurements obtained from use of this equipment accurately represent the spatial neutron flux distribution of the reactor core.

3/4.3.7.8 AMMONIA DETECTION SYSTEM

The OPERABILITY of the ammonia detection system ensures that an accidental ammonia release will be detected promptly and the necessary protective actions will be automatically initiated to provide protection for control room personnel. Upon detection of a high concentration of ammonia, the control room emergency ventilation system will automatically be placed in the recirculation mode of operation to provide the required protection. The detection systems required by this specification are consistent with the recommendations of Regulatory Guide 1.78 "Assumptions for Evaluating the Habitability of a Nuclear Power Plant Control Room During a Postulated Hazardous Chemical Release."

3/4.3.7.9 FIRE DETECTION INSTRUMENTATION

OPERABILITY of the fire detection instrumentation ensures that adequate warning capability is available for the prompt detection of fires. This capability is required in order to detect and locate fires in their early stages. Prompt detection of fires will reduce the potential for damage to safety-related equipment and is an integral element in the overall facility fire protection program.

SURVEILLANCE REQUIREMENTS (Continued)

b. At least once per 18 months[##] or (1) after any structural maintenance on the HEPA filter or charcoal adsorber housings, or (2) following painting, fire or chemical release in any ventilation zone communicating with the train by:

1. Verifying that the train satisfies the in-place testing acceptance criteria and uses the test procedures of Regulatory Positions C.5.a, C.5.c and C.5.d of Regulatory Guide 1.52, Revision 2, March 1978, and the train flow rate is 4000 cfm ± 10%.

2. Verifying within 31 days after removal that a laboratory analysis of a representative carbon sample obtained in accordance with Regulatory Position C.6.b of Regulatory Guide 1.52, Revision 2, March 1978, meets the laboratory testing criteria of Regulatory Position C.6.a of Regulatory Guide 1.52, Revision 2, March 1978.

3. Verifying a train flow rate of 4000 cfm ± 10% during subsystem operation when tested in accordance with ANSI N510-1975.

c. After every 720** hours of charcoal adsorber operation by verifying within 31 days after removal that a laboratory analysis of a representative carbon sample obtained in accordance with Regulatory Positon C.6.b of Regulatory Guide 1.52, Revision 2, March 1978, meets the laboratory testing criteria of Regulatory Position C.6.a of Regulatory Guide 1.52, Revision 2, March 1978.

d. At least once per 18 months by:

1. Verifying that the pressure drop across the combined HEPA filters and charcoal adsorber banks is less than 8 inches Water Gauge while operating the train at a flow rate of 4000 cfm ± 10%.

2. Verifying that on a simulated ammonia signal, the recirculating charcoal filter automatically switches to the recirculation mode of operation and the isolation dampers close within 6 seconds.

[##]This surveillance shall include the recirculating charcoal filter, "odor eater," in the normal control room supply filter train using ANSI N510-1975 as a guide to verify $\geq$ 70% efficiency in removing freon test gas.

**Except that recirculating charcoal filter samples shall be removed and analyzed at least once per 18 months.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-374

LA SALLE COUNTY STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 20
License No. NPF-18

1. The Nuclear Regulatory Commission (the Commission or the NRC) having found that:

   A. The application for amendment filed by the Commonwealth Edison Company (the licensee), dated February 7, 1986, as supplemented by letter dated March 5, 1986, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;

   C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the enclosure to this license amendment and paragraph 2.C.(2) of the Facility Operating License No. NPF-18 is hereby amended to read as follows:

   (2) Technical Specifications and Environmental Protection Plan

   The Technical Specifications contained in Appendix A, as revised through Amendment No. 20, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This amendment is effective upon date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Elinor G. Adensam

Elinor G. Adensam, Director
BWR Project Directorate No. 3
Division of BWR Licensing

Enclosure:
Changes to the Technical
Specifications

Date of Issuance: APR [illegible]

ENCLOSURE TO LICENSE AMENDMENT NO. 20

FACILITY OPERATING LICENSE NO. NPF-18

DOCKET NO. 50-374

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain a vertical line indicating the area of change.

REMOVE	INSERT
V	V
3/4 3-74	3/4 3-74
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3/4 7-5	3/4 7-5

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LIMITING CONDITION FOR OPERATION

3.3.7.8 Two independent ammonia detection system subsystems shall be OPERABLE,* each with two ammonia detectors, with their alarm/trip setpoints adjusted to actuate at an ammonia concentration of less than or equal to 25 ppm.

APPLICABILITY: All OPERATIONAL CONDITIONS.

ACTION:

a. With one ammonia detector in either detection subsystem inoperable, restore the inoperable detector(s) to OPERABLE status within 7 days or, within the next 6 hours, initiate and maintain operation of at least one control room charcoal filter system train in the recirculation mode of operation.

b. With both ammonia detection subsystems inoperable, within 1 hour initiate and maintain operation of at least one control room charcoal filter system train in the recirculation mode of operation.

c. The provisions of Specification 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

4.3.7.8 Each of the above required ammonia detection system subsystems shall shall be demonstrated OPERABLE by performance of a CHANNEL FUNCTIONAL TEST at least once per 31 days and a CHANNEL CALIBRATION at least once per 18 months.

*The normal or emergency power source may be inoperable in OPERATIONAL CONDITION 4 or 5.

MONITORING INSTRUMENTATION (Continued)

3/4.3.7.5 ACCIDENT MONITORING INSTRUMENTATION

The OPERABILITY of the accident monitoring instrumentation ensures that sufficient information is available on selected plant parameters to monitor and assess important variables following an accident. This capability is consistent with the recommendations of Regulatory Guide 1.97, "Instrumentation for Light Water Cooled Nuclear Power Plants to Assess Plant Conditions During and Following an Accident," December 1975 and NUREG-0578, "TMI-2 Lessons Learned Task Force Status Report and Short-Term Recommendations".

3/4.3.7.6 SOURCE RANGE MONITORS

The source range monitors provide the operator with information of the status of the neutron level in the core at very low power levels during startup and shutdown. At these power levels, reactivity additions should not be made without this flux level information available to the operator. When the intermediate range monitors are on scale adequate information is available without the SRMs and they can be retracted.

3/4.3.7.7 TRAVERSING IN-CORE PROBE SYSTEM

The OPERABILITY of the traversing in-core probe system with the specified minimum complement of equipment ensures that the measurements obtained from use of this equipment accurately represent the spatial neutron flux distribution of the reactor core.

3/4.3.7.8 AMMONIA DETECTION SYSTEM

The OPERABILITY of the ammonia detection system ensures that an accidental ammonia release will be detected promptly and the necessary protective actions will be automatically initiated to provide protection for control room personnel. Upon detection of a high concentration of ammonia, the control room emergency ventilation system will automatically be placed in the recirculation mode of operation to provide the required protection. The detection systems required by this specification are consistent with the recommendations of Regulatory Guide 1.78, "Assumptions for Evaluating the Habitability of a Nuclear Power Plant Control Room During a Postulated Hazardous Chemical Release."

3/4.3.7.9 FIRE DETECTION INSTRUMENTATION

OPERABILITY of the fire detection instrumentation ensures that adequate warning capability is available for the prompt detection of fires. This capability is required in order to detect and locate fires in their early stages. Prompt detection of fires will reduce the potential for damage to safety-related equipment and is an integral element in the overall facility fire protection program.

SURVEILLANCE REQUIREMENTS (Continued)

b. At least once per 18 months[##] or (1) after any structural maintenance on the HEPA filter or charcoal adsorber housings, or (2) following painting, fire or chemical release in any ventilation zone communicating with the train by:

1. Verifying that the train satisfies the in-place testing acceptance criteria and uses the test procedures of Regulatory Positions C.5.a, C.5.c and C.5.d of Regulatory Guide 1.52, Revision 2, March 1978, and the train flow rate is 4000 cfm ± 10%.

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3. Verifying a train flow rate of 4000 cfm ± 10% during subsystem operation when tested in accordance with ANSI N510-1975.

c. After every 720** hours of charcoal adsorber operation by verifying within 31 days after removal that a laboratory analysis of a representative carbon sample obtained in accordance with Regulatory Positon C.6.b of Regulatory Guide 1.52, Revision 2, March 1978, meets the laboratory testing criteria of Regulatory Position C.6.a of Regulatory Guide 1.52, Revision 2, March 1978.

d. At least once per 18 months by:

1. Verifying that the pressure drop across the combined HEPA filters and charcoal adsorber banks is less than 8 inches Water Gauge while operating the train at a flow rate of 4000 cfm ± 10%.

2. Verifying that on a simulated ammonia detection signal, the recirculating charcoal filter automatically switches to the recirculation mode of operation and the isolation dampers close within 6 seconds.

##This surveillance shall include the recirculating charcoal filter, "odor eater," in the normal control room supply filter train using ANSI N510-1975 as a guide to verify $\geq$ 70% efficiency in removing freon test gas.

**Except that recirculating charcoal filter samples shall be removed and analyzed at least once per 18 months.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 38 TO FACILITY OPERATING LICENSE NO. NPF-11 AND

AMENDMENT NO. 20 TO FACILITY OPERATING LICENSE NO. NPF-18

COMMONWEALTH EDISON COMPANY

LA SALLE COUNTY STATION, UNITS 1 AND 2

DOCKET NOS. 50-373 AND 50-374

1.0 INTRODUCTION

Commonwealth Edison Company (licensee), in its submittal dated February 7, 1986, and as supplemented by letters dated March 5, 1986, proposed to amend Appendix A, Technical Specification to Facility Operating License NPF-11 and NPF-18. The proposed changes are to eliminate the chlorine detectors from the control room habitability ventilation system because of new information which indicates the very low frequency of chlorine shipments in bulk quantities near the La Salle County Station.

2.0 EVALUATION

Chlorine detectors have been a high maintenance item which require significant manhours to survey and repair at the La Salle County Station. These detectors have, on numerous times, spuriously actuated trips initiating the recirculation mode of the control room and the auxiliary electric equipment room ventilation systems. Both of these systems are engineered safety features.

The licensee indicates that chlorine detectors are not required for the La Salle County Station for reasons discussed below.

On the basis of Regulatory Guide 1.78, "Assumptions for Evaluating the Habitability of a Nuclear Power Plant Control Room during a Postulated Hazardous Chemical Release," chlorine, as a hazardous chemical, requires a habitability analysis in case there is an accidental chlorine release from stationary or mobile sources near the plant. In addition, Position 1 of Regulatory Guide 1.78 states that chlorine stored or situated at distances greater than five miles from the control room need not be considered in evaluating the habitability of the control room.

The closest industries to La Salle County Station where chlorine may be stored are greater than five miles away. The three modes of transportation of chemicals for these industries are railroads, highways and the Illinois River. The railroads and highways are all located farther than five miles from the station. Only the Illinois River is located approximately 4.7 miles north of the station. However, early in 1986, Sargent and Lundy, the licensee's architect/engineer, conducted a survey of chlorine shipments on the Illinois River, indicating that the frequency of bulk chlorine shipments is extremely low.

In addition, the topographic profile of the La Salle County Station site is favorable with respect to the settling effect of chlorine in air, since the Illinois River is located over 200 feet below the grade elevation of the Station. Moreover, the control room ventilation inlets are an additional 130 feet above grade. Hence, even in the case of a rare occurrence of chlorine release from the shipping barges on the Illinois River, the likelihood of lethal concentrations of chlorine reaching the control room is highly unlikely.

The staff reviewed the licensee's submittal and contacted the Chlorine Institute of New York, the U.S. Coast Guard in Washington, D.C., the U.S. Army Corps of Engineers, local manufacturers and two transportation terminals in the vicinity of the La Salle County Station site. The staff finds that the information from the licensee's survey of chlorine shipments is acceptable.

On the basis of the above evaluation, we find that the proposed changes will not endanger the safety of the control room operators. Therefore, the staff concludes that removal of chlorine detectors at the La Salle County Station is acceptable. However, due to potential future changes in the chlorine shipping patterns within five miles of the La Salle County Station, the staff recommended, and the licensee agreed in a letter dated March 5, 1986, to conduct a survey of chlorine shipments on the Illinois River every three years and document the results in the Annual Report of the La Salle Station.

3.0 ENVIRONMENTAL CONSIDERATION

These amendments involve a change in the installation or use of facility components located within the restricted area as defined in 10 CFR Part 20 and changes in surveillance requirements. The staff has determined that these amendments involve no significant increase in the amounts, and no significant changes in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that these amendments involve no significant hazards consideration and there has been no public comment on such finding. Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

4.0 CONCLUSION

The Commission made a proposed determination that these amendments involve no significant hazards consideration which was published in the Federal Register (51 FR 8588) on March 12, 1986, and consulted with the state of Illinois. No public comments were received, and the state of Illinois did not have any comments.

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Angela Chu, NRR

Dated: APR 1 1 1984

AMENDMENT NO. 38 TO FACILITY OPERATING LICENSE NO. NPF-11 - LA SALLE, UNIT 1 and
AMENDMENT NO. 20 to FACILITY OPERATING LICENSE NO. NPF-18 - LA SALLE, UNIT 2

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