



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

July 25, 1986

Docket No. 50-373

Mr. Dennis L. Farrar
Director of Licensing
Commonwealth Edison Company
P.O. Box 767
Chicago, Illinois 60690

Dear Mr. Farrar:

Subject: Issuance of Amendment No. 45 to Facility Operating License
No. NPF-11 - La Salle County Station, Unit 1

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 45 to Facility Operating License No. NPF-11 for the La Salle County Station, Unit 1. This amendment is in response to your letter dated June 10, 1986.

The amendment revises the La Salle Unit 1 Technical Specifications to reflect the correct Rod Block Monitor setpoints for both dual and single loop operation performed for the Cycle 2 Reload package which was not included in the May 9, 1986, Cycle 2 amendment. In addition, an administrative change is made to correct a typographical error by deleting the Thermal Hydraulic Stability Specification from Index page VI. The page number for Figure 3.4.1.1-1 had been previously corrected by letter dated May 19, 1986.

A copy of the related safety evaluation supporting Amendment No. 45 to Facility Operating License No. NPF-11 is enclosed.

Sincerely,

Elinor G. Adensam, Director
BWR Project Directorate No. 3
Division of BWR Licensing

Enclosures:

1. Amendment No. 45 to NPF-11
2. Safety Evaluation

cc w/enclosure:
See next page

DESIGNATED ORIGINAL

Certified By

8608010063 860725
PDR ADOCK 05000373
PDR

Mr. Dennis L. Farrar
Commonwealth Edison Company

La Salle County Nuclear Power Station
Units 1 & 2

cc:
Philip P. Steptoe, Esquire
Suite 4200
One First National Plaza
Chicago, Illinois 60603

John W. McCaffrey
Chief, Public Utilities Division
160 North La Salle Street, Room 900
Chicago, Illinois 60601

Assistant Attorney General
188 West Randolph Street
Suite 2315
Chicago, Illinois 60601

Resident Inspector/LaSalle, NPS
U.S. Nuclear Regulatory Commission
Rural Route No. 1
P.O. Box 224
Marseilles, Illinois 61341

Chairman
La Salle County Board of Supervisors
La Salle County Courthouse
Ottawa, Illinois 61350

Attorney General
500 South 2nd Street
Springfield, Illinois 62701

Chairman
Illinois Commerce Commission
Leland Building
527 East Capitol Avenue
Springfield, Illinois 62706

Mr. Gary N. Wright, Manager
Nuclear Facility Safety
Illinois Department of Nuclear Safety
1035 Outer Park Drive, 5th Floor
Springfield, Illinois 62704

Regional Administrator, Region III
U. S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, Illinois 60137

AMENDMENT NO. 45 TO FACILITY OPERATING LICENSE NO. NPF-11 - LA SALLE, UNIT 1

DISTRIBUTION:

Docket No. 50-373

NPC PDR

Local PDR

PRC System

NSIC

BWD-3 r/f

ABournia (4)

EHylton (2)

EAdensam

Attorney, OELD

CMiles

RDiggs

JPartlow

BGrimes

EJordan

LHarmon

TBarnhart (4)

FEltawila

Mary Johns, PIII

EButcher



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-373

LA SALLE COUNTY STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 45
License No. NPF-11

1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:
 - A. The application for amendment filed by the Commonwealth Edison Company (the licensee), dated June 10, 1986, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the enclosure to this license amendment and paragraph 2.C.(2) of the Facility Operating License No. NPF-11 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 45, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

8608010067 860725
PDR ADOCK 05000373
P PDR

3. This amendment is effective as of date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Elinor G. Adensam

Elinor G. Adensam, Director
BWR Project Directorate No. 3
Division of BWR Licensing

Enclosure:
Changes to the Technical
Specifications

Date of Issuance: July 25, 1986

ENCLOSURE TO LICENSE AMENDMENT NO. 45

FACILITY OPERATING LICENSE NO. NPF-11

DOCKET NO. 50-373

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain a vertical line indicating the area of change.

REMOVE

VI
3/4 3-53

INSERT

VI
3/4 3-53

INDEX

LIMITING CONDITIONS FOR OPERATION AND SURVEILLANCE REQUIREMENTS

<u>SECTION</u>	<u>PAGE</u>
<u>3/4.4 REACTOR COOLANT SYSTEM</u>	
3/4.4.1 RECIRCULATION SYSTEM	
Recirculation Loops.....	3/4 4-1
Jet Pumps.....	3/4 4-2
Recirculation Loop Flow.....	3/4 4-3
Idle Recirculation Loop Startup.....	3/4 4-4
3/4.4.2 SAFETY/RELIEF VALVES.....	3/4 4-5
3/4 4.3 REACTOR COOLANT SYSTEM LEAKAGE	
Leakage Detection Systems.....	3/4 4-6
Operational Leakage.....	3/4 4-7
3/4.4.4 CHEMISTRY.....	3/4 4-10
3/4.4.5 SPECIFIC ACTIVITY.....	3/4 4-13
3/4.4.6 PRESSURE/TEMPERATURE LIMITS	
Reactor Coolant System.....	3/4 4-16
Reactor Steam Dome.....	3/4 4-20
3/4.4.7 MAIN STEAM LINE ISOLATION VALVES.....	3/4 4-21
3/4.4.8 STRUCTURAL INTEGRITY.....	3/4 4-22
3/4.4.9 RESIDUAL HEAT REMOVAL	
Hot Shutdown.....	3/4 4-23
Cold Shutdown.....	3/4/4-24
<u>3/4.5 EMERGENCY CORE COOLING SYSTEMS</u>	
3/4.5.1 ECCS-OPERATING.....	3/4 5-1
3/4.5.2 ECCS-SHUTDOWN.....	3/4 5-6
3/4.5.3 SUPPRESSION CHAMBER.....	3/4 5-8

TABLE 3.3.6-2

CONTROL ROD WITHDRAWAL BLOCK INSTRUMENTATION SETPOINTS

<u>TRIP FUNCTION</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUE</u>
1. <u>ROD BLOCK MONITOR</u>		
a. Upscale		
1) Two Recirculation Loop Operation	$\leq 0.66 W + 38\%$	$\leq 0.66 W + 41\%$
2) Single Recirculation Loop Operation	$\leq 0.66W + 32.7\%$	$\leq 0.66W + 35.7\%$
b. Inoperative	N.A.	N.A.
c. Downscale	$\geq 5\%$ of RATED THERMAL POWER	$\geq 3\%$ of RATED THERMAL POWER
2. <u>APRM</u>		
a. Flow Biased Simulated Thermal Power-Upscale		
1) Two Recirculation Loop Operation	$\leq 0.66 W + 42\%^*$	$\leq 0.66 W + 45\%^*$
2) Single Recirculation Loop Operation	$\leq 0.66W + 36.7\%$	$\leq 0.66W + 39.7\%^*$
b. Inoperative	N.A.	N.A.
c. Downscale	$\geq 5\%$ of RATED THERMAL POWER	$\geq 3\%$ of RATED THERMAL POWER
d. Neutron Flux-High	$\leq 12\%$ of RATED THERMAL POWER	$\leq 14\%$ of RATED THERMAL POWER
3. <u>SOURCE RANGE MONITORS</u>		
a. Detector not full in	N.A.	N.A.
b. Upscale	$< 2 \times 10^5$ cps	$< 5 \times 10^5$ cps
c. Inoperative	N.A.	N.A.
d. Downscale	≥ 0.7 cps	≥ 0.5 cps
4. <u>INTERMEDIATE RANGE MONITORS</u>		
a. Detector not full in	N.A.	N.A.
b. Upscale	$< 108/125$ of full scale	$< 110/125$ of full scale
c. Inoperative	N.A.	N.A.
d. Downscale	$\geq 5/125$ of full scale	$\geq 3/125$ of full scale



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 45 TO FACILITY OPERATING LICENSE NO. NPF-11

COMMONWEALTH EDISON COMPANY

LA SALLE COUNTY STATION, UNIT 1

DOCKET NO. 50-373

1.0 INTRODUCTION

By letter dated June 10, 1986, Commonwealth Edison (the licensee) requested an amendment to the Technical Specifications for Facility Operating License NPF-11 for La Salle County Station Unit 1. The amendment reflects the corrections to the Rod Block Monitor setpoints and a typographical error on Index page VI.

2.0 EVALUATION

The NRC recently approved Amendment 40 dated May 9, 1986, to the Technical Specifications of La Salle Unit 1 relating to the Reload for Cycle 2. It has subsequently been discovered by Commonwealth Edison (the licensee), and reported in a proposed amendment dated June 10, 1986, that the submittal for that Cycle 2 amendment did not include a necessary change to Table 3.3.6-2 for the Rod Block Monitor setpoints for single and two recirculation loop operation. This change is required by the selection of the rod block setpoint of 107 percent used in the rod withdrawal error at power analysis performed for the reload. The use of a 107 percent setpoint is within a normally expected range so that the requested change to Table 3.3.6-2 is acceptable.

Also in the June 10th submittal, it was indicated that an administrative error relating to the inclusion of a thermal-hydraulic stability section in the Index to the Technical Specifications should be corrected. This change does not affect plant operation and is also acceptable. The page number for Figure 3.4.1.1-1 was previously corrected by letter dated May 19, 1986.

3.0 ENVIRONMENTAL CONSIDERATION

This amendment involves a change in the use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes in surveillance requirements and a typographical error. The staff has determined that this amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

4.0 CONCLUSION

The Commission made a proposed determination that the amendment involves no significant hazards consideration which was published in the Federal Register (51 FR 23173) on June 25, 1986. No public comments were received, and the state of Illinois did not have any comments.

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: H. Richings, NRR

Dated: July 25, 1986