

January 17, 1984

Docket No.: 50-373

CPI

Mr. Dennis L. Farrar  
Director of Nuclear Licensing  
Commonwealth Edison Company  
P. O. Box 767  
Chicago, Illinois 60690

Dear Mr. Farrar:

Subject: Issuance Amendment No.15 to Facility Operating License  
No. NPF-11 -La Salle County Station, Unit 1

The Nuclear Regulatory Commission has issued the enclosed Amendment No.15 to Facility Operating License No. NPF-11 for the La Salle County Station, Unit 1. The amendment consists of changes to the Technical Specifications in response to your application submitted on January 4, 1984. The amendment was authorized by telephone on January 4, 1984, and was confirmed by letter dated January 9, 1984.

The amendment would insert an item 3 to Specification 3.1.3.1 and item c to Specification 3.1.3.6 to specify in both items that the provisions of Specification 3.0.4 are not applicable.

A copy of the related safety evaluation supporting Amendment No.15 to Facility Operating License NPF-11 is enclosed.

Sincerely,

Original signed by  
A. Schwencer, Chief  
Licensing Branch No. 2  
Division of Licensing

Enclosures:

1. Amendment No.15 to NPF-11
2. Safety Evaluation

cc w/ enclosures:

See next page

\*SEE PREVIOUS CONCURRENCES

DL:LB#2/PM*	DL:LB#2/LA*	DL:LB#2/BC*	DL:AD/L*
ABournia:pt	EGHylton	ASchwencer	TMNovak
1/09/84	1/09/84	1/09/84	1/17/84

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P PDR

La Salle

Mr. Dennis L. Farrar  
Director of Nuclear Licensing  
Commonwealth Edison Company  
P. O. Box 767  
Chicago, Illinois 60690

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Chairman  
La Salle County Board of Supervisors  
La Salle County Courthouse  
Ottawa, Illinois 61350

Attorney General  
500 South 2nd Street  
Springfield, Illinois 62701

Department of Public Health  
Attn: Chief, Division of Nuclear Safety  
535 West Jefferson  
Springfield, Illinois 62761

The Honorable Tom Corcoran  
United States House of Representatives  
Washington, D. C. 20515

Chairman  
Illinois Commerce Commission  
Leland Building  
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Springfield, Illinois 62706

Mr. Gary N. Wright, Manager  
Nuclear Facility Safety  
Illinois Department of Nuclear Safety  
1035 Outer Park Drive, 5th Floor  
Springfield, Illinois 62704



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY  
DOCKET NO. 50-373  
LA SALLE COUNTY STATION, UNIT 1  
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 15  
License No. NPF-11

1. The Nuclear Regulatory Commission (the Commission or the NRC) having found that:
  - A. The application for amendment filed by the Commonwealth Edison Company, dated January 4, 1984, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
  - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of the Facility Operating License No. NPF-11 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 15, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

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3. This amendment was effective as of January 4, 1984.

FOR THE NUCLEAR REGULATORY COMMISSION

*Original signed by*

A. Schwencer, Chief  
Licensing Branch No. 2  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: January 17, 1984

*AS*  
DL:LB#2/PM  
ABournia:pt  
1/6/84

DL:LB#2/LA  
Eaton  
1/9/84

*REM 1-14-84*  
DL:LB#2/BC  
ASchwencer  
1/6/84

*Weld*  
DL:AD  
1/7/84

*with notes attached*  
DL:AD  
TMNovak  
1/7/84

ATTACHMENT TO LICENSE AMENDMENT NO. 15  
FACILITY OPERATING LICENSE NO. NPF-11  
DOCKET NO. 50-373

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

REMOVE

3/4 1-4

3/4 1-11

INSERT

3/4 1-4

3/4 1-11

## REACTIVITY CONTROL SYSTEM

### LIMITING CONDITION FOR OPERATION-(Continued)

#### ACTION (Continued)

2. If the inoperable control rod(s) is inserted:
  - a) Within one hour disarm the associated directional control valves\*\* either:
    - 1) Electrically, or
    - 2) Hydraulically by closing the drive water and exhaust water isolation valves.
  - b) Otherwise, be in at least HOT SHUTDOWN within the next 12 hours.
3. The provisions of Specification 3.0.4 are not applicable.
- c. With more than 8 control rods inoperable, be in at least HOT SHUTDOWN within 12 hours.

#### SURVEILLANCE REQUIREMENTS

4.1.3.1.1 The scram discharge volume drain and vent valves shall be demonstrated OPERABLE by:

- a. At least once per 31 days verifying each valve to be open\*, and
- b. At least once per 92 days cycling each valve through at least one complete cycle of full travel.

4.1.3.1.2 When above the low power setpoint of the RWM and RSCS, all withdrawn control rods not required to have their directional control valves disarmed electrically or hydraulically shall be demonstrated OPERABLE by moving each control rod at least one notch:

- a. At least once per 7 days, and
- b. At least once per 24 hours when any control rod is immovable as a result of excessive friction or mechanical interference.

4.1.3.1.3 All control rods shall be demonstrated OPERABLE by performance of Surveillance Requirements 4.1.3.2, 4.1.3.4, 4.1.3.5, 4.1.3.6 and 4.1.3.7.

\*These valves may be closed intermittently for testing under administrative control.

\*\*May be rearmed intermittently, under administrative control, to permit testing associated with restoring the control rod to OPERABLE status.

REACTIVITY CONTROL SYSTEM

CONTROL ROD DRIVE COUPLING

LIMITING CONDITION FOR OPERATION

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3.1.3.6 All control rods shall be coupled to their drive mechanisms.

APPLICABILITY: OPERATIONAL CONDITIONS 1, 2 and 5\*.

ACTION:

- a. In OPERATIONAL CONDITION 1 and 2 with one control rod not coupled to its associated drive mechanism:
  1. Within 2 hours, either:
    - a) If permitted by the RWM and RSCS, insert the control rod drive mechanism to accomplish recoupling and verify recoupling by withdrawing the control rod, and:
      - 1) Observing any indicated response of the nuclear instrumentation, and
      - 2) Demonstrating that the control rod will not go to the overtravel position.
    - b) If recoupling is not accomplished on the first attempt or, if not permitted by the RWM or RSCS then until permitted by the RWM and RSCS, declare the control rod inoperable and insert the control rod and disarm the associated directional control valves\*\* either:
      - 1) Electrically, or
      - 2) Hydraulically by closing the drive water and exhaust water isolation valves.
  2. Otherwise, be in at least HOT SHUTDOWN within the next 12 hours.
- b. In OPERATIONAL CONDITION 5\* with a withdrawn control rod not coupled to its associated drive mechanism, within 2 hours, either:
  1. Insert the control rod to accomplish recoupling and verify recoupling by withdrawing the control rod and demonstrating that the control rod will not go to the overtravel position, or
  2. If recoupling is not accomplished, insert the control rod and disarm the associated directional control valves\*\* either:
    - a) Electrically, or
    - b) Hydraulically by closing the drive water and exhaust water isolation valves.
- c. The provisions of Specification 3.0.4 are not applicable.

\*At least each withdrawn control rod. Not applicable to control rods removed per Specification 3.9.10.1 or 3.9.10.2.

\*\*May be rearmed intermittently, under administrative control, to permit testing associated with restoring the control rod to OPERABLE status.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION  
AMENDMENT NO. 15 TO NPF-11  
LA SALLE COUNTY STATION, UNIT 1  
DOCKET NO. 50-373

Introduction

By letter dated January 4, 1984, Commonwealth Edison Company (the licensee) proposed an emergency Technical Specification change to Specifications 3.1.3.1 and 3.1.3.6 to exempt the applicability of Specification 3.0.4 when a control rod is inoperable for causes other than being stuck. The licensee in attempting to bring La Salle Unit 1 up to power discovered a control rod not being coupled; and therefore, declared the control rod inoperable. Specification 3.0.4 is applicable for this circumstance and this specification prohibits mode change.

Evaluation

During the preparation of starting up the La Salle County Station, Unit 1, the licensee determined that one control rod would not pass a control rod coupling check. Repeated attempts at recoupling were not successful. The current Unit 1 Technical Specifications do not allow continuation of startup with a control rod declared inoperable.

Previously licensed boiling water reactors have Technical Specifications (in various forms) which permit startup and continued operation with inoperable control rods. In particular, the Brunswick and Hatch plants along with La Salle, Unit 2 have such a statement in corresponding Technical Specifications. However, for continued operation with a control rod inoperable, adequate shutdown margin must still be assured as required by Specification 3.1.1. The issue of common mode failures is also addressed by the Technical Specifications because if more than 8 rods are inoperable, the reactor must be in hot shutdown in 12 hours.

The licensee has determined, and the the NRC staff agrees that in accordance with 10 CFR 50.92 that operation of La Salle Unit 1, with the the proposed amendment will not: (1) involve a significant increase in the probability or consequences of an accident previously evaluated because the control rod which is uncoupled during mode changes will be fully inserted into the core and will be disarmed to prevent withdrawal, (2) create the possibility of a new or different kind of accident from any accident previously evaluated because the change to the Technical Specifications is only to allow mode changes, and (3) involve a significant reduction in the margin of safety because the control rod will be in its most conservative position. Accordingly, the staff has made a final determination that this license amendment involves no significant hazards considerations.

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In addition, the problem of the flux perturbation must be considered with this control rod inserted. Since the core power distribution is monitored continually by operation personnel, any effects of the inserted control rod on core operating limits may be corrected by the operator by altering the control rod pattern. In discussing this problem with the licensee, it was indicated that the power distribution flux tilt meets the tolerance allowed for the flux pattern of the core event with this control rod inserted. Also, it was pointed out that the control rod symmetric to the inoperable rod will be partially inserted into the core to reduce the flux tilt in that region.

The licensee meets the test of prompt notification. On January 4, 1984, the need for the change was transmitted by letter (hand carried) to the NRC staff.

The NRC staff contacted the State of Illinois (Mr. G. N. Wright, Manager of Nuclear Facility Safety) by telephone at 5:15 pm on January 4, 1984. The proposed amendment was described and he was informed that a no significant hazards determination had been made by the staff. Mr. Wright had no comments.

#### Environmental Consideration

We have determined that this amendment does not authorize a change in effluent types or total amount nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that this amendment involves action which is insignificant from the standpoint of environmental impact, and pursuant to 10 CFR Section 51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

#### Conclusion

We have concluded, based on the considerations discussed above, that: (1) this amendment does not involve a significant increase in the probability or consequences of an accident previously evaluated, creation of a new or different accident from any previously evaluated, or a significant reduction in a safety margin, and therefore, does not involve a significant hazards consideration; (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: January 4, 1984

\*SEE PREVIOUS CONCURRENCES  
DL:LB#2/PM DL:LB#2/BC  
ABournia\*:pt ASchwencer\*  
1/04/84 1/04/84

DL:AD/L  
TM Novak  
1/17/84

DISTRIBUTION: (LA SALLE AMENDMENT NO. 15)

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