



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DEC 06 1985

Docket No. 50-373

Mr. Dennis L. Farrar
Director of Licensing
Commonwealth Edison Company
P.O. Box 767
Chicago, Illinois 60690


Dear Mr. Farrar:

SUBJECT: ISSUANCE OF AMENDMENT NO. 31 TO FACILITY OPERATING LICENSE
NO. NPF-11 - LA SALLE COUNTY STATION, UNIT 1

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 31 to Facility Operating License No. NPF-11 for the La Salle County Station, Unit 1. This amendment is in response to your letter dated October 11, 1985. The amendment revises the La Salle Unit 1 Technical Specifications since you are now removing the air-operated-testable-bypass-check valves installed in each of the emergency core cooling systems and the reactor core isolation cooling system because they are not required.

A copy of the related safety evaluation supporting Amendment No. 31 to Facility Operating License NPF-11 is enclosed.

Sincerely,

for 
Elinor Adensam, Director
of Project Directorate No. 3
Division of BWR Licensing

Enclosures:

1. Amendment No. 31 to NPF-11
2. Safety Evaluation

cc w/enclosures:
See next page

John M. Ch...
2000000000

8512120430 851206
PDR ADOCK 05000373
P PDR

Mr. Dennis L. Farrar
Commonwealth Edison Company

La Salle County Nuclear Power Station
Units 1 & 2

cc:

Philip P. Steptoe, Esquire
Suite 4200
One First National Plaza
Chicago, Illinois 60603

John W. McCaffrey
Chief, Public Utilities Division
160 North La Salle Street, Room 900
Chicago, Illinois 60601

Assistant Attorney General
188 West Randolph Street
Suite 2315
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Resident Inspector/LaSalle, NPS
U.S. Nuclear Regulatory Commission
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La Salle County Board of Supervisors
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Illinois Department of Nuclear Safety
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Regional Administrator, Region III
U. S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, Illinois 60137



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COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-373

LA SALLE COUNTY STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 31
License No. NPF-11

1. The Nuclear Regulatory Commission (the Commission or the NRC) having found that:
 - A. The application for amendment filed by the Commonwealth Edison Company, dated October 11, 1985, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. NPF-11 is hereby amended to read as follows:


(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 31 and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

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3. This amendment is effective upon startup following the first refueling.

FOR THE NUCLEAR REGULATORY COMMISSION


for Elinor Adensam, Director
of Project Directorate No. 3
Division of BWR Licensing

Enclosure:
Changes to the Technical
Specifications

Date of Issuance: DEC 06 1985

ENCLOSURE TO LICENSE AMENDMENT NO. 31
FACILITY OPERATING LICENSE NO. NPF-11
DOCKET NO. 50-373

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3/4 6-25

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TABLE 3.6.3-1 (Continued)
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<u>VALVE FUNCTION AND NUMBER</u>	<u>VALVE GROUP</u> ^(a)	<u>MAXIMUM ISOLATION TIME (Seconds)</u>
<u>Automatic Isolation Valves (Continued)</u>		
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10. LPCS, HPCS, RCIC, RHR Injection Testable Check Bypass Valves	N.A.	N.A.



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SAFETY EVALUATION

AMENDMENT NO. 31 TO NPF-11

LA SALLE COUNTY STATION, UNIT 1

DOCKET NO. 50-373

Introduction

By letter dated October 11, 1985, Commonwealth Edison Company (the licensee) requested an amendment to the La Salle Unit 1 Technical Specifications. The proposed changes are to remove the air-operated-testable-bypass-check valves installed in each of the emergency-core-cooling systems (ECCS) and reactor core cooling isolation (RCIC) system since they are not required.

Evaluation

The amendment proposes that the testable check bypass valves in the ECCS and RCIC system be removed. The valves affected are the testable check valves:

Low Pressure Core Injection	#1E12-F327A, B, C
Low Pressure Core Spray	#1E21-F333,
High Pressure Core Spray	#1E22-F354, and
Reactor Core Cooling Isolation	#1E51-F354, F355

The purpose of the bypass valves was to enable the exercising of the above listed testable check valves during the reactor operation by equalizing the pressure across the check valve disk. Since the testable-check valves are in the injection flow path, they must be periodically exercised; and, therefore, are included in the Pump and Valve Inservice Inspection Program per ASME Section XI. The La Salle Unit 1 Inservice Inspection Program calls for these testable-check valves to be exercised during cold shutdown. Therefore, the bypass check valves are not required.

Use of the bypass for its intended purpose would, in fact, introduce the risk of eliminating a high-low pressure boundary isolation valve during the time necessary to conduct the exercise. As proposed, removal of the bypass valves would improve safety conditions during operations by:

- 1) keeping the availability of the reactor coolant system high-low pressure interface isolation at a maximum, and
- 2) removing the high-low pressure boundary leakage path concerns.

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Hence, with the exercise of the testable check valves of the ECCS and RCIC system during cold shutdown, the staff finds the proposed amendment acceptable.

Environmental Consideration

This amendment involves a change in the installation or use of facility components located within the restricted area as defined in 10 CFR Part 20 and changes surveillance requirements. The staff has determined that this amendment involves no significant change in the types and no significant increase in the amounts of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

Conclusion

The Commission made a proposed determination that this amendment involves no significant hazards consideration which was published in the Federal Register (50 FR 48211) on November 6, 1985. No public comments were received.

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: DEC 06 1985

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FOR THE NUCLEAR REGULATORY COMMISSION


for Elinor Adensam, Director
of Project Directorate No. 3
Division of BWR Licensing

Enclosure:
Changes to the Technical
Specifications

Date of Issuance: DEC 06 1985

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for 

Elinor Adensam, Director
of Project Directorate No. 3
Division of BWR Licensing

Enclosures:

1. Amendment No. 31 to NPF-11
2. Safety Evaluation

cc w/enclosures:
See next page

Mr. Dennis L. Farrar
Commonwealth Edison Company

La Salle County Nuclear Power Station
Units 1 & 2

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-373

LA SALLE COUNTY STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 31
License No. NPF-11

1. The Nuclear Regulatory Commission (the Commission or the NRC) having found that:
 - A. The application for amendment filed by the Commonwealth Edison Company, dated October 11, 1985, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. NPF-11 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 31 and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This amendment is effective upon startup following the first refueling.

FOR THE NUCLEAR REGULATORY COMMISSION



for Elinor Adensam, Director
of Project Directorate No. 3
Division of BWR Licensing

Enclosure:
Changes to the Technical
Specifications

Date of Issuance: DEC 06 1985

ENCLOSURE TO LICENSE AMENDMENT NO. 31
FACILITY OPERATING LICENSE NO. NPF-11
DOCKET NO. 50-373

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed page. The revised page is identified by Amendment number and contains a vertical line indicating the area of change.

REMOVE

3/4 6-25

INSERT

3/4 6-25

TABLE 3.6.3-1 (Continued)

PRIMARY CONTAINMENT ISOLATION VALVES

<u>VALVE FUNCTION AND NUMBER</u>	<u>VALVE GROUP</u> ^(a)	<u>MAXIMUM ISOLATION TIME (Seconds)</u>
<u>Automatic Isolation Valves (Continued)</u>		
8. Containment Vent and Purge Valves	4	
1VQ026		< 10**
1VQ027		< 10**
1VQ029		< 10**
1VQ030		< 10**
1VQ031		< 10**
1VQ032		< 5
1VQ034		< 10**
1VQ035		< 5
1VQ036		< 10**
1VQ040		< 10**
1VQ042		< 10**
1VQ043		< 10**
1VQ047		< 5
1VQ048		< 5
1VQ050		< 5
1VQ051		< 5
1VQ068		< 5
9. RCIC Turbine Exhaust Vacuum Breaker Line Valves	9	N.A.
1E51-F080		
1E51-F086		
10. LPCS, HPCS, RCIC, RHR Injection Testable Check Bypass Valves	N.A.	N.A.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION

AMENDMENT NO. 31 TO NPF-11

LA SALLE COUNTY STATION, UNIT 1

DOCKET NO. 50-373

Introduction

By letter dated October 11, 1985, Commonwealth Edison Company (the licensee) requested an amendment to the La Salle Unit 1 Technical Specifications. The proposed changes are to remove the air-operated-testable-bypass-check valves installed in each of the emergency-core-cooling systems (ECCS) and reactor core cooling isolation (RCIC) system since they are not required.

Evaluation

The amendment proposes that the testable check bypass valves in the ECCS and RCIC system be removed. The valves affected are the testable check valves:

Low Pressure Core Injection	#1E12-F327A, B, C
Low Pressure Core Spray	#1E21-F333,
High Pressure Core Spray	#1E22-F354, and
Reactor Core Cooling Isolation	#1E51-F354, F355

The purpose of the bypass valves was to enable the exercising of the above listed testable check valves during the reactor operation by equalizing the pressure across the check valve disk. Since the testable-check valves are in the injection flow path, they must be periodically exercised; and, therefore, are included in the Pump and Valve Inservice Inspection Program per ASME Section XI. The La Salle Unit 1 Inservice Inspection Program calls for these testable-check valves to be exercised during cold shutdown. Therefore, the bypass check valves are not required.

Use of the bypass for its intended purpose would, in fact, introduce the risk of eliminating a high-low pressure boundary isolation valve during the time necessary to conduct the exercise. As proposed, removal of the bypass valves would improve safety conditions during operations by:

- 1) keeping the availability of the reactor coolant system high-low pressure interface isolation at a maximum, and
- 2) removing the high-low pressure boundary leakage path concerns.

Hence, with the exercise of the testable check valves of the ECCS and RCIC system during cold shutdown, the staff finds the proposed amendment acceptable.

Environmental Consideration

This amendment involves a change in the installation or use of facility components located within the restricted area as defined in 10 CFR Part 20 and changes surveillance requirements. The staff has determined that this amendment involves no significant change in the types and no significant increase in the amounts of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

Conclusion

The Commission made a proposed determination that this amendment involves no significant hazards consideration which was published in the Federal Register (50 FR 48211) on November 6, 1985. No public comments were received.

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: DEC 06 1985

DISTRIBUTION

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DEC 06 1985

Docket No. 50-373

Mr. Dennis L. Farrar
Director of Licensing
Commonwealth Edison Company
P.O. Box 767
Chicago, Illinois 60690


Dear Mr. Farrar:

SUBJECT: ISSUANCE OF AMENDMENT NO. 31 TO FACILITY OPERATING LICENSE
NO. NPF-11 - LA SALLE COUNTY STATION, UNIT 1

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 31 to Facility Operating License No. NPF-11 for the La Salle County Station, Unit 1. This amendment is in response to your letter dated October 11, 1985. The amendment revises the La Salle Unit 1 Technical Specifications since you are now removing the air-operated-testable-bypass-check valves installed in each of the emergency core cooling systems and the reactor core isolation cooling system because they are not required.

A copy of the related safety evaluation supporting Amendment No. 31 to Facility Operating License NPF-11 is enclosed.

Sincerely,

for 
Elinor Adensam, Director
of Project Directorate No. 3
Division of BWR Licensing

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for Elinor Adensam, Director
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Dated: DEC 06 1985