Docket Nos. 50-373/374

Mr. Dennis L. Farrar Director of Licensing Commonwealth Edison Company P.O. Box 767 Chicago, Illinois 60690

Dear Mr. Farrar:

SUBJECT: ISSUANCE OF AMENDMENT NO. 19 TO FACILITY OPERATING LICENSE

NO. NPF-11 AND AMENDMENT NO. 5 TO FACILITY OPERATING LICENSE

NO. NPF-18 - LA SALLE COUNTY STATION, UNITS 1 AND 2

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 19 to Facility Operating License No. NPF-11 and Amendment No. 5 to Facility Operating License No. NPF-18 for the La Salle County Station, Units 1 and 2. These amendments are in response to your letter dated July 25, 1984. The amendments would revise the La Salle, Units 1 and 2 Technical Specifications to modify limits on monitors in accordance with the guidance specified in Generic Letter No. 83-36, "NUREG-0737 Technical Specifications".

A copy of the related safety evaluation supporting Amendment No. 19 to Facility Operating License NPF-11 and Amendment No. 5 to Facility Operating License NPF-18 is enclosed.

Sincerely,

A. Schwencer, Chief Licensing Branch No. 2 Division of Licensing

Enclosures:

1. Amendment No. 19 to NPF-11

2. Amendment No. 5 to NPF-18

3. Safety Evaluation

cc w/enclosure: See next page

\* DL:LB#2 EHylton 10/30/84 \*DL:LB#2 ABournia 10/30/84 \*DL:LB#2
ASchwencer

\*See previous concurrence

11/9/84

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Mr. Dennis L. Farrar Director of Nuclear Licensing Commonwealth Edison Company P.O. Box 767 Chicago, Illinois 60690

cc: Philip P. Steptoe, Esquire Suite 4200 One First National Plaza Chicago, Illinois 60603

> Assistant Attorney General 188 West Randolph Street Suite 2315 Chicago, Illinois 60601

Michael J. Jordan, Resident Inspector La Salle, NPS, U.S.N.R.C. P.O. Box 224 Marseilles, Illinois 61364

Chairman La Salle County Board of Supervisors La Salle County Courthouse Ottawa, Illinois 61350

Attorney General 500 South 2nd Street Springfield, Illinois 62701

Department of Public Health 535 West Jefferson Springfield, Illinois 62761 ATTN: Chief, Division of Nuclear Safety

The Honorable Tom Corcoran
United States House of Representatives
Washington, D.C. 20515

Chairman
Illinois Commerce Commission
Leland Building
527 East Capitol Avenue
Springfield, Illinois 62706

Mr. Gary N. Wright, Manager Nuclear Facility Safety Illinois Department of Nuclear Safety 1035 Outer Park Drive, 5th Floor Springfield, Illinois 62704

## Issuance of Amendment No. 19 to Facility Operating Licensing No. NPF-11 La Salle County Station, Unit 2

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### UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

#### COMMONWEALTH EDISON COMPANY

#### DOCKET NO. 50-373

#### LA SALLE COUNTY STATION, UNIT 1

#### AMENDMENT TO FACILITY OPERATING LICENSE

Amendment 19 License No. NPF-11

- 1. The Nuclear Regulatory Commission (the Commission or the NRC) having found that:
  - A. The application for amendment filed by the Commonwealth Edison Company, dated July 25, 1984, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
  - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
- 2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of the Facility Operating License No. NPF-11 hereby amended to read as follows:
  - (2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 19, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This amendment is effective as of date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

A. Schwencer, Chief Licensing Branch No. 2 Division of Licensing

Enclosure: Changes to the Technical Specifications

Date of Issuance: NOV 09 1984

DL:UB#2 EHNYIton DL:LB#2 ABournia:pob 10/3°/84

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DL:ADIL TMNOVak 10/9/84

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# ENCLOSURE TO LICENSE AMENDMENT NO. 19 FACILITY OPERATING LICENSE NO. NPF-11 DOCKET NO. 50-373

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain a vertical line indicating the area of change.

REMOVE	INSERT		
3/4 3-69 3/4 3-70	3/4 3-69 3/4 3-70 3/4 3-70(a)		

#### INSTRUMENTATION

#### ACCIDENT MONITORING INSTRUMENTATION

#### LIMITING CONDITION FOR OPERATION

3.3.7.5 The accident monitoring instrumentation channels shown in Table 3.3.7.5-1 shall be OPERABLE.

APPLICABILITY: OPERATIONAL CONDITIONS 1 and 2.

#### ACTION:

With one or more accident monitoring instrumentation channels inoperable, take the ACTION required by Table 3.3.7.5-1.

#### SURVEILLANCE REQUIREMENTS

4.3.7.5 Each of the above required accident monitoring instrumentation channels shall be demonstrated OPERABLE by performance of the CHANNEL CHECK and CHANNEL CALIBRATION operations at the frequencies shown in Table 4.3.7.5-1.

TABLE 3.3.7.5-1

ACCIDENT MONITORING INSTRUMENTATION

	REQUIRED NUMBER OF CHANNELS	MINIMUM CHANNELS OPERABLE	ACTION
Reactor Vessel Pressure	2	1	80
Reactor Vessel Water Level	2	1	80
Suppression Chamber Water Level	2	1	80
Suppression Chamber Water Temperature	7, 1/well	7, 1/well	80
Suppression Chamber Air Temperature	2	1	80
Drywell Pressure	2	1	80
Drywell Air Temperature	2	1	80
Drywell Oxygen Concentration*	2	1	80
Drywell Hydrogen Concentration Analyzer* and Monitor	2	1	82
Primary Containment Gross Gamma Radiation	2	1	81
	1/valve	1/valve	80
	1	1	81
	1	1	81
	Reactor Vessel Water Level  Suppression Chamber Water Level  Suppression Chamber Water Temperature  Suppression Chamber Air Temperature  Drywell Pressure  Drywell Air Temperature  Drywell Oxygen Concentration*  Drywell Hydrogen Concentration Analyzer* and Monitor  Primary Containment Gross Gamma Radiation  Safety/Relief Valve Position Indicators  Noble Gas Monitor, Main Stack	Reactor Vessel Pressure  Reactor Vessel Water Level  Suppression Chamber Water Level  Suppression Chamber Water Temperature  Suppression Chamber Air Temperature  Drywell Pressure  Drywell Air Temperature  Drywell Air Temperature  Drywell Oxygen Concentration*  Drywell Hydrogen Concentration Analyzer* and Monitor  Primary Containment Gross Gamma Radiation  Safety/Relief Valve Position Indicators  Noble Gas Monitor, Main Stack  2  Reactor Vessel Pressure  2  2  2  1/valve	Reactor Vessel Pressure  Reactor Vessel Water Level  Suppression Chamber Water Level  Suppression Chamber Water Temperature  Suppression Chamber Air Temperature  Drywell Pressure  Drywell Air Temperature  Drywell Oxygen Concentration*  Drywell Hydrogen Concentration Analyzer* and Monitor  Primary Containment Gross Gamma Radiation  Safety/Relief Valve Position Indicators  Noble Gas Monitor, Main Stack  Reactor Vessel ChanNELS OPERABLE  1  1  1  CHANNELS OPERABLE  1  1  1  1  1  1  1  1  1  1  1  1  1

<sup>\*</sup>Actuated after LOCA.

#### <u>Table 3.3.7.5-1</u> (Continued)

### ACCIDENT MONITORING INSTRUMENTATION ACTION STATEMENTS

#### ACTION 80 -

- a. With the number of OPERABLE accident monitoring instrumentation channels less than the Required Number of Channels shown in Table 3.3.7.5-1, restore the inoperable channel(s) to OPERABLE status within 7 days or be in at least HOT SHUTDOWN within the next 12 hours.
- b. With the number of OPERABLE accident monitoring instrumentation channels less than the Minimum Channels OPERABLE requirements of Table 3.3.7.5-1, restore the inoperable channel(s) to OPERABLE status within 48 hours or be in at least HOT SHUTDOWN within the next 12 hours.

#### ACTION 81 -

With the number of OPERABLE channels less than required by the minimum channels OPERABLE requirements, initiate the preplanned alternate method of monitoring the appropriate parameter(s) within 72 hours, and:

- 1) either restore the inoperable channel(s) to OPERABLE status within 7 days of the event, or
- 2) prepare and submit a Special Report to the Commission pursuant to Specification 6.6.C within 14 days following the event outlining the action taken, the cause of the inoperability and the plans and schedule for restoring the system to OPERABLE status.

#### ACTION 82 -

- a. With the number of OPERABLE channels one less than the required number of channels shown in Table 3.3.7.5-1, restore the inoperable channel to OPERABLE status within 30 days or be in at least HOT SHUTDOWN within the next 12 hours.
- b. With the number of OPERABLE channels less than the minimum channels OPERABLE requirements of Table 3.3.7.5-1, restore at least one channel to OPERABLE status within 7 days or be in at least HOT SHUTDOWN within the next 12 hours.



## UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

#### COMMONWEALTH EDISON COMPANY

#### DOCKET NO. 50-374

#### LA SALLE COUNTY STATION, UNIT 2

#### AMENDMENT TO FACILITY OPERATING LICENSE

Amendment 5 License No. NPF-18

- The Nuclear Regulatory Commission (the Commission or the NRC) having found that:
  - A. The application for amendment filed by the Commonwealth Edison Company, dated July 25, 1984, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
  - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
- 2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of the Facility Operating License No. NPF-18 hereby amended to read as follows:
  - (2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 5, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This amendment is effective as of date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

A. Schwencer, Chief Licensing Branch No. 2 Division of Licensing

Enclosure: Changes to the Technical Specifications

Date of Issuance: NOV 09 1984

#### ENCLOSURE TO LICENSE AMENDMENT NO. 5 FACILITY OPERATING LICENSE NO. NPF-18 DOCKET NO. 50-374

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain a vertical line indicating the area of change.

REMOVE	INSERT		
3/4 3-69 3/4 3-70	3/4 3-69 3/4 3-70		
	3/4 3-70(a)		

#### INSTRUMENTATION

#### ACCIDENT MONITORING INSTRUMENTATION

#### LIMITING CONDITION FOR OPERATION

3.3.7.5 The accident monitoring instrumentation channels shown in Table 3.3.7.5-1 shall be OPERABLE.

APPLICABILITY: OPERATIONAL CONDITIONS 1 and 2.

#### ACTION:

a. With one or more accident monitoring instrumentation channels inoperable, take the ACTION required by Table 3.3.7.5-1.

#### SURVEILLANCE REQUIREMENTS

4.3.7.5 Each of the above required accident monitoring instrumentation channels shall be demonstrated OPERABLE by performance of the CHANNEL CHECK and CHANNEL CALIBRATION operations at the frequencies shown in Table 4.3.7.5-1.

TABLE 3.3.7.5-1

#### ACCIDENT MONITORING INSTRUMENTATION

ALLE - L			REQUIRED NUMBER OF CHANNELS	MINIMUM CHANNELS OPERABLE	ACTION
TINU	1.	Reactor Vessel Pressure	2	- 1	80
2	2.	Reactor Vessel Water Level	2	1	80
	3.	Suppression Chamber Water Level	2	1	80
	4.	Suppression Chamber Water Temperature	7, 1/well	7, 1/well	80
	5.	Suppression Chamber Air Temperature	2	1	80
	6.	Drywell Pressure	2	1	80
3/4	7.	Drywell Air Temperature	2	1	80
3-70	8.	Drywell Oxygen Concentration*	2	1	80
Amendn	9.	Drywell Hydrogen Concentration Analyzer* and Monitor	2	1	82
	10.	Primary Containment Gross Gamma Radiation	2	1	81
	11.	Safety/Relief Valve Position Indicators	1/valve	1/valve	80
	12.	Noble Gas Monitor, Main Stack	1	1	81
	13.	Noble Gas Monitor, Standby Gas Treatment System Stack	1	1	81

<sup>\*</sup>Actuated after LOCA.

#### Table 3.3.7.5-1 (Continued)

### ACCIDENT MONITORING INSTRUMENTATION ACTION STATEMENTS

#### ACTION 80 -

- a. With the number of OPERABLE accident monitoring instrumentation channels less than the Required Number of Channels shown in Table 3.3.7.5-1, restore the inoperable channel(s) to OPERABLE status within 7 days or be in at least HOT SHUTDOWN within the next 12 hours.
- b. With the number of OPERABLE accident monitoring instrumentation channels less than the Minimum Channels OPERABLE requirements of Table 3.3.7.5-1, restore the inoperable channel(s) to OPERABLE status within 48 hours or be in at least HOT SHUTDOWN within the next 12 hours.

#### ACTION 81 -

With the number of OPERABLE channels less than the required by the minimum channels OPERABLE requirements, initiate the preplanned alternate method of monitoring the appropriate parameter(s) within 72 hours, and:

- 1) either restore the inoperable channel(s) to OPERABLE status within 7 days of the event, or
- 2) prepare and submit a Special Report to the Commission pursuant to Specification 6.6.c within 14 days following the event outlining the action taken, the cause of the inoperability and the plans and schedule for restoring the system to OPERABLE status.

#### ACTION 82 -

- a. With the number of OPERABLE channels one less than the required number of channels shown in Table 3.3.7.5-1, restore the inoperable channel to OPERABLE status within 30 days or be in at least HOT SHUTDOWN within the next 12 hours.
- b. With the number of OPERABLE channels less then the minimum channels OPERABLE requirements of Table 3.3.7.5-1, restore at least one channel to OPERABLE status within 7 days or be in at least HOT SHUTDOWN within the next 12 hours.



## UNITED STATES N' CLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

#### SAFETY EVALUATION

#### AMENDMENT NO. 19 TO NPF-11 AND

AMENDMENT NO. 2 TO NPF-18

#### LA SALLE COUNTY STATION, UNITS 1 AND 2

DOCKET NOS. 50-373 AND 50-374

#### Introduction

By letter dated July 25, 1984, Commonwealth Edison Company (the licensee) proposed amendments that would revise the La Salle Units 1 and 2 Technical Specifications to modify limits on monitors in accordance with the guidance specified in Generic letter No. 83-36, "NUREG-0737 Technical Specifications."

#### **Evaluation**

The action statements for the accident monitoring instrumentation for (1) Drywell Hydrogen Concentration Monitor, (2) Primary Containment Gross Gamma Radiation, and (3) Noble Gas Monitors are modified in the La Salle Units 1 and 2 Technical Specifications. These changes were requested by the NRC staff in Generic Letter No. 83-36.

These changes modify the Technical Specifications requiring shutdown where plant safety is not in question. The results of the changes were reviewed and found to clearly fall within all acceptance criteria with respect to the system or component specified in the standard review plan. The staff has has reviewed the licensee's responses to the staff request and finds them acceptable.

#### Environmental Consideration

This amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

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#### Conclusion

The Commission made a proposed determination that the amendment involves no significant hazards consideration which was published in the Federal Register (49 FR 38390) on September 28, 1984. No public comments were received.

We have concluded, based on the considerations discussed above, that:
(1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: NOV 9 8 1984