UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

Docket No. 50-373/374

JAN 0 8 1985

Mr. Dennis L. Farrar
Director of Nuclear Licensing
Commonwealth Edison Company
P.O. Box 767
Chicago, Illinois 60690

Dear Mr. Farrar:

Subject: Issuance Amendment No. 20 to Facility Operating License No. NPF-11 and Amendment No. 7 to Facility Operating License No. NPF-18-La Salle County Station, Units 1 and 2

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 20 to Facility Operating License No. NPF-11 and Amendment No. 7 to Facility Operating License No. NPF-18 for the La Salle County Station, Units 1 and 2. These amendments are in response to your letter dated September 19, 1984 and as modified by your letter of October 5, 1984. The amendments change the Technical Specifications for Units 1 and 2 by eliminating the reactor water cleanup pump room ambient and differential temperature monitoring requirements.

A copy of the related safety evaluation supporting Amendment 20 to Facility Operating License NPF-11 and Amendment 7 to Facility Operating License NPF-18 is enclosed.

Sincerely,

A. Schwencer

A. Schwencer, Chief
Licensing Branch No. 2
Division of Licensing

Enclosures:
1. Amendment No. 20 to NPF-11
2. Amendment No. 7 to NPF-18
3. Safety Evaluation

cc: w/ enclosures
See next page

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Sincerely,

Original signed by:
A. Schwencer, Chief
Licensing Branch No. 2
Division of Licensing

Enclosures:
1. Amendment No. 20 to NPF-11
2. Amendment No. 7 to NPF-18
3. Safety Evaluation

cc: w/ enclosures
See next page
*See previous concurrence

DL:LB#2/PM	DL:LB#2/LA	DL:LB#2/BC
ABournia:bm	EHylton	ASchwencer
12/21/84	12/21/84	~~12/21/84~~ 1/8/85

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A copy of the related safety evaluation supporting Amendment 20 to Facility Operating License NPF-11 and Amendment 7 to Facility Operating License NPF-18 is enclosed.

Sincerely,

A. Schwencer, Chief
Licensing Branch No. 2
Division of Licensing

Enclosures:
1. Amendment No. 20 to NPF-11
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3. Safety Evaluation

cc: w/ enclosures
See next page

DL:LB#2/PM	DL:LB#2/LA	DL:LB#2/BC
ABournia:bm	EHylton	ASchwencer
12/21/84	12/21/84	12/21/84

La Salle

Mr. Dennis L. Farrar
Director of Nuclear Licensing
Commonwealth Edison Company
P.O. Box 767
Chicago, Illinois 60690

cc: Philip P. Steptoe, Esquire
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Assistant Attorney General
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Suite 2315
Chicago, Illinois 60601

Michael J. Jordan, Resident Inspector
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P.O. Box 224
Marseilles, Illinois 61364

Chairman
La Salle County Board of Supervisors
La Salle County Courthouse
Ottawa, Illinois 61350

Attorney General
500 South 2nd Street
Springfield, Illinois 62701

Department of Public Health
535 West Jefferson
Springfield, Illinois 62761
ATTN: Chief, Division of Nuclear Safety

The Honorable Tom Corcoran
United States House of Representatives
Washington, D.C. 20515

Chairman
Illinois Commerce Commission
Leland Building
527 East Capitol Avenue
Springfield, Illinois 62706

Mr. Gary N. Wright, Manager
Nuclear Facility Safety
Illinois Department of Nuclear Safety
1035 Outer Park Drive, 5th Floor
Springfield, Illinois 62704

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY
DOCKET NO. 50-373
LA SALLE COUNTY STATION, UNIT 1
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment 20
License No. NPF-11

1. The Nuclear Regulatory Commission (the Commission or the NRC) having found that:

   A. The application for amendment filed by the Commonwealth Edison Company, dated September 19, 1984, as supplemented by letter dated October 5, 1984, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;

   C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of the Facility Operating License No. NPF-11 is hereby amended to read as follows:

   (2) Technical Specifications and Environmental Protection Plan

   The Technical Specifications contained in Appendix A, as revised through Amendment No. 20, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This amendment is effective as of date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

A. Schwencer

A. Schwencer, Chief
Licensing Branch No. 2
Division of Licensing

Enclosure:
Changes to the Technical
Specifications

Date of Issuance: JAN 08 1985

3. This amendment is effective as of date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Original signed by:

A. Schwencer, Chief
Licensing Branch No. 2
Division of Licensing

Enclosure:
Changes to the Technical
Specifications

Date of Issuance: JAN 08 1985

DL:LB#2/PM	DL:LB#2/LA	DL:LB#2/BC	OELD	DL:AD/L
ABournia:bdm	EHylton	ASchwencer	CWoodhead	TNovak
12/21/84	12/21/84	~~12/27/84~~ 1/8/85	12/28/84	~~12/5/84~~

ENCLOSURE TO LICENSE AMENDMENT NO. 20
FACILITY OPERATING LICENSE NO. NPF-11
DOCKET NO. 50-373

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain a vertical line indicating the area of change.

REMOVE	INSERT
3/4 3-12	3/4 3-12
3/4 3-15	3/4 3-15
3/4 3-18	3/4 3-18
3/4 3-20	3/4 3-20

TABLE 3.3.2-1 (Continued)

ISOLATION ACTUATION INSTRUMENTATION

TRIP FUNCTION	VALVE GROUPS OPERATED BY SIGNAL (a)	MINIMUM OPERABLE CHANNELS PER TRIP SYSTEM (b)	APPLICABLE OPERATIONAL CONDITION	ACTION
3. REACTOR WATER CLEANUP SYSTEM ISOLATION				
a. Δ Flow - High	5	1	1, 2, 3	22
b. Heat Exchanger Area Temperature - High	5	1	1, 2, 3	22
c. Heat Exchanger Area Ventilation ΔT - High	5	1	1, 2, 3	22
d. SLCS Initiation	5(f)	NA	1, 2, 3	22
e. Reactor Vessel Water Level - Low Low, Level 2	5	2	1, 2, 3	22
4. REACTOR CORE ISOLATION COOLING SYSTEM ISOLATION				
a. RCIC Steam Line Flow - High	8	1	1, 2, 3	22
b. RCIC Steam Supply Pressure - Low	8, 9(g)	2	1, 2, 3	22
c. RCIC Turbine Exhaust Diaphragm Pressure - High	8	2	1, 2, 3	22
d. RCIC Equipment Room Temperature - High	8	1	1, 2, 3	22
e. RCIC Steam Line Tunnel Temperature - High	8	1	1, 2, 3	22
f. RCIC Steam Line Tunnel Δ Temperature - High	8	1	1, 2, 3	22
g. Drywell Pressure - High	9(g)	2	1, 2, 3	22

TABLE 3.3.2-2

ISOLATION ACTUATION INSTRUMENTATION SETPOINTS

TRIP FUNCTION	TRIP SETPOINT	ALLOWABLE VALUE
A. AUTOMATIC INITIATION		
1. PRIMARY CONTAINMENT ISOLATION		
a. Reactor Vessel Water Level		
1) Low, Level 3	$\geq$ 12.5 inches*	$\geq$ 11.0 inches*
2) Low Low, Level 2	$\geq$ -50 inches*	$\geq$ -57 inches*
b. Drywell Pressure - High	$\leq$ 1.69 psig	$\leq$ 1.89 psig
c. Main Steam Line		
1) Radiation - High	$\leq$ 3.0 x full power background	$\leq$ 3.6 x full background
2) Pressure - Low	$\geq$ 854 psig	$\geq$ 834 psig
3) Flow - High	$\leq$ 111 psid	$\leq$ 116 psid
d. Main Steam Line Tunnel Temperature - High	$\leq$ 140°F	$\leq$ 146°F
e. Main Steam Line Tunnel Δ Temperature - High	$\leq$ 36°F	$\leq$ 42°F
f. Condenser Vacuum - Low	$\geq$ 7 inches Hg vacuum	$\geq$ 5.5 inches Hg vacuum
2. SECONDARY CONTAINMENT ISOLATION		
a. Reactor Building Vent Exhaust Plenum Radiation - High	$\leq$ 10 mr/hr	$\leq$ 15 mr/hr
b. Drywell Pressure - High	$\leq$ 1.69 psig	$\leq$ 1.89 psig
c. Reactor Vessel Water Level - Low Low, Level 2	$\geq$ -50 inches*	$\geq$ -57 inches*
d. Fuel Pool Vent Exhaust Radiation - High	$\leq$ 10 mr/hr	$\leq$ 15 mr/hr
3. REACTOR WATER CLEANUP SYSTEM ISOLATION		
a. ΔFlow - High	$\leq$ 70 gpm	$\leq$ 87.5 gpm
b. Heat Exchanger Area Temperature - High	$\leq$ 181°F	$\leq$ 187°F
c. Heat Exchanger Area Ventilation ΔT - High	$\leq$ 85°F	$\leq$ 91°F
d. SLCS Initiation	NA	NA
e. Reactor Vessel Water Level - Low Low, Level 2	$\geq$ -50 inches*	$\geq$ -57 inches*

TABLE 3.3.2-3

ISOLATION SYSTEM INSTRUMENTATION RESPONSE TIME

TRIP FUNCTION	RESPONSE TIME (Seconds)#
A. AUTOMATIC INITIATION	
1. PRIMARY CONTAINMENT ISOLATION	
a. Reactor Vessel Water Level	
1) Low, Level 3	NA
2) Low Low, Level 2	$\leq 1.0^{*}/\leq 13^{(a)**}$
b. Drywell Pressure - High	$\leq 13^{(a)}$
c. Main Steam Line	
1) Radiation - High[(b)]	$\leq 1.0^{*}/\leq 13^{(a)**}$
2) Pressure - Low	$\leq 1.0^{*}/\leq 13^{(a)**}$
3) Flow - High	$\leq 0.5^{*}/\leq 13^{(a)**}$
d. Main Steam Line Tunnel Temperature - High	NA
e. Condenser Vacuum - Low	NA
f. Main Steam Line Tunnel Δ Temperature - High	NA
2. SECONDARY CONTAINMENT ISOLATION	
a. Reactor Building Vent Exhaust Plenum Radiation - High[(b)]	$\leq 13^{(a)}$
b. Drywell Pressure - High	$\leq 13^{(a)}$
c. Reactor Vessel Water Level - Low, Level 2	$\leq 13^{(a)}$
d. Fuel Pool Vent Exhaust Radiation - High[(b)]	$\leq 13^{(a)}$
3. REACTOR WATER CLEANUP SYSTEM ISOLATION	
a. Δ Flow - High	$\leq 13^{(a)\#\#}$
b. Heat Exchanger Area Temperature - High	NA
c. Heat Exchanger Area Ventilation ΔT-High	NA
d. SLCS Initiation	NA
e. Reactor Vessel Water Level - Low Low, Level 2	$\leq 13^{(a)}$
4. REACTOR CORE ISOLATION COOLING SYSTEM ISOLATION	
a. RCIC Steam Line Flow - High	$\leq 13^{(a)\#\#\#}$
b. RCIC Steam Supply Pressure - Low	$\leq 13^{(a)}$
c. RCIC Turbine Exhaust Diaphragm Pressure - High	NA
d. RCIC Equipment Room Temperature - High	NA
e. RCIC Steam Line Tunnel Temperature - High	NA
f. RCIC Steam Line Tunnel Δ Temperature - High	NA
g. Drywell Pressure - High	NA
5. RHR SYSTEM STEAM CONDENSING MODE ISOLATION	
a. RHR Equipment Area Δ Temperature - High	NA
b. RHR Area Cooler Temperature - High	NA
c. RHR Heat Exchanger Steam Supply Flow High	NA

TABLE 4.3.2.1-1

ISOLATION ACTUATION INSTRUMENTATION SURVEILLANCE REQUIREMENTS

TRIP FUNCTION	CHANNEL CHECK	CHANNEL FUNCTIONAL TEST	CHANNEL CALIBRATION	OPERATIONAL CONDITIONS FOR WHICH SURVEILLANCE REQUIRED
A. AUTOMATIC INITIATION				
1. PRIMARY CONTAINMENT ISOLATION				
a. Reactor Vessel Water Level				
1) Low, Level 3	S	M	R	1, 2, 3
2) Low Low, Level 2	S	M	R	1, 2, 3
b. Drywell Pressure - High	NA	M	Q	1, 2, 3
c. Main Steam Line				
1) Radiation - High	S	M	R	1, 2, 3
2) Pressure - Low	NA	M	Q	1
3) Flow - High	S	M	R	1, 2, 3
d. Main Steam Line Tunnel Temperature - High	NA	M	R	1, 2, 3
e. Condenser Vacuum - Low	NA	M	Q	1, 2*, 3*
f. Main Steam Line Tunnel Δ Temperature - High	NA	M	R	1, 2, 3
2. SECONDARY CONTAINMENT ISOLATION				
a. Reactor Building Vent Exhaust Plenum Radiation - High	S	M	R	1, 2, 3 and **
b. Drywell Pressure - High	NA	M	Q	1, 2, 3
c. Reactor Vessel Water Level - Low Low, Level 2	S	M	R	1, 2, 3, and #
d. Fuel Pool Vent Exhaust Radiation - High	S	M	R	1, 2, 3 and **
3. REACTOR WATER CLEANUP SYSTEM ISOLATION				
a. Δ Flow - High	S	M	R	1, 2, 3
b. Heat Exchanger Area Temperature - High	NA	M	Q	1, 2, 3
c. Heat Exchanger Area Ventilation ΔT - High	NA	M	Q	1, 2, 3
d. SLCS Initiation	NA	R	NA	1, 2, 3
e. Reactor Vessel Water Level - Low Low, Level 2	S	M	R	1, 2, 3

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY
DOCKET NO. 50-374
LA SALLE COUNTY STATION, UNIT 2
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment 7
License No. NPF-18

1. The Nuclear Regulatory Commission (the Commission or the NRC) having found that:

   A. The application for amendment filed by the Commonwealth Edison Company, dated September 19, 1984, as supplemented by letter dated October 5, 1984, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;

   C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of the Facility Operating License No. NPF-18 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 7, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This amendment is effective as of date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

A. Schwencer, Chief
Licensing Branch No. 2
Division of Licensing

Enclosure:
Changes to the Technical
Specifications

Date of Issuance: JAN 0 8 1985

3. This amendment is effective as of date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Original signed by:

A. Schwencer, Chief
Licensing Branch No. 2
Division of Licensing

Enclosure:
Changes to the Technical
Specifications

Date of Issuance: JAN 0 8 1985

DL:LB#2/PM	DL:LB#2/LA	DL:LB#2/BC	OELD	DL:AD/L
ABournia:bdm	EHylton	ASchwencer	CWoodhead	INovak
12/21/84	12/21/84	~~12/24/84~~ 1/8/85	12/27/84	12/4/84

ENCLOSURE TO LICENSE AMENDMENT NO. 7
FACILITY OPERATING LICENSE NO. NPF-18
DOCKET NO. 50-374

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain a vertical line indicating the area of change.

REMOVE	INSERT
3/4 3-12	3/4 3-12
3/4 3-15	3/4 3-15
3/4 3-18	3/4 3-18
3/4 3-20	3/4 3-20

TABLE 3.3.2-1 (Continued)

ISOLATION ACTUATION INSTRUMENTATION

TRIP FUNCTION	VALVE GROUPS OPERATED BY SIGNAL (a)	MINIMUM OPERABLE CHANNELS PER TRIP SYSTEM (b)	APPLICABLE OPERATIONAL CONDITION	ACTION
3. REACTOR WATER CLEANUP SYSTEM ISOLATION				
a. Δ Flow - High	5	1	1, 2, 3	22
b. Heat Exchanger Area Temperature - High	5	1	1, 2, 3	22
c. Heat Exchanger Area Ventilation ΔT - High	5	1	1, 2, 3	22
d. SLCS Initiation	5(f)	NA	1, 2, 3	22
e. Reactor Vessel Water Level - Low Low, Level 2	5	2	1, 2, 3	22
4. REACTOR CORE ISOLATION COOLING SYSTEM ISOLATION				
a. RCIC Steam Line Flow - High	8	1	1, 2, 3	22
b. RCIC Steam Supply Pressure - Low	8, 9(g)	2	1, 2, 3	22
c. RCIC Turbine Exhaust Diaphragm Pressure - High	8	2	1, 2, 3	22
d. RCIC Equipment Room Temperature - High	8	1	1, 2, 3	22
e. RCIC Steam Line Tunnel Temperature - High	8	1	1, 2, 3	22
f. RCIC Steam Line Tunnel Δ Temperature - High	8	1	1, 2, 3	22
g. Drywell Pressure - High	9(g)	2	1, 2, 3	22

TABLE 3.3.2-2

ISOLATION ACTUATION INSTRUMENTATION SETPOINTS

TRIP FUNCTION	TRIP SETPOINT	ALLOWABLE VALUE
A. AUTOMATIC INITIATION		
1. PRIMARY CONTAINMENT ISOLATION		
a. Reactor Vessel Water Level		
1) Low, Level 3	≥ 12.5 inches*	≥ 11.0 inches*
2) Low Low, Level 2	≥ -50 inches*	≥ -57 inches*
b. Drywell Pressure - High	≤ 1.69 psig	≤ 1.89 psig
c. Main Steam Line		
1) Radiation - High	≤ 3.0 x full power background	≤ 3.6 x full background
2) Pressure - Low	≥ 854 psig	≥ 834 psig
3) Flow - High	≤ 111 psid	≤ 116 psid
d. Main Steam Line Tunnel Temperature - High	≤ 140°F	< 146°F
e. Main Steam Line Tunnel Δ Temperature - High	≤ 36°F	≤ 42°F
f. Condenser Vacuum - Low	≥ 7 inches Hg vacuum	≥ 5.5 inches Hg vacuum
2. SECONDARY CONTAINMENT ISOLATION		
a. Reactor Building Vent Exhaust Plenum Radiation - High	≤ 10 mr/h	≤ 15 mr/h
b. Drywell Pressure - High	≤ 1.69 psig	≤ 1.89 psig
c. Reactor Vessel Water Level - Low Low, Level 2	≥ -50 inches*	≥ -57 inches*
d. Fuel Pool Vent Exhaust Radiation - High	≤ 10 mr/h	≤ 15 mr/h
3. REACTOR WATER CLEANUP SYSTEM ISOLATION		
a. ΔFlow - High	≤ 70 gpm	≤ 87.5 gpm
b. Heat Exchanger Area Temperature - High	≤ 181°F	≤ 187°F
c. Heat Exchanger Area Ventilation ΔT - High	≤ 85°	≤ 91°F
d. SLCS Initiation	N.A.	N.A.
e. Reactor Vessel Water Level - Low Low, Level 2	≥ -50 inches*	≥ -57 inches*

TABLE 3.3.2-3

ISOLATION SYSTEM INSTRUMENTATION RESPONSE TIME

TRIP FUNCTION	RESPONSE TIME (Seconds)#
A. AUTOMATIC INITIATION	
1. PRIMARY CONTAINMENT ISOLATION	
a. Reactor Vessel Water Level	
1) Low, Level 3	N.A.
2) Low Low, Level 2	$\leq 1.0^{*}/\leq 13^{(a)**}$
b. Drywell Pressure - High	$\leq 13^{(a)}$
c. Main Steam Line	
1) Radiation - High[(b)]	$\leq 1.0^{*}/\leq 13^{(a)**}$
2) Pressure - Low	$\leq 1.0^{*}/\leq 13^{(a)**}$
3) Flow - High	$\leq 0.5^{*}/\leq 13^{(a)**}$
d. Main Steam Line Tunnel Temperature - High	N.A.
e. Condenser Vacuum - Low	N.A.
f. Main Steam Line Tunnel Δ Temperature - High	N.A.
2. SECONDARY CONTAINMENT ISOLATION	
a. Reactor Building Vent Exhaust Plenum Radiation - High[(b)]	$\leq 13^{(a)}$
b. Drywell Pressure - High	$\leq 13^{(a)}$
c. Reactor Vessel Water Level - Low, Level 2[(b)]	$\leq 13^{(a)}$
d. Fuel Pool Vent Exhaust Radiation - High[(b)]	$\leq 13^{(a)}$
3. REACTOR WATER CLEANUP SYSTEM ISOLATION	
a. Δ Flow - High	$\leq 13^{(a)\#\#}$
b. Heat Exchanger Area Temperature - High	N.A.
c. Heat Exchanger Area Ventilation ΔT-High	N.A.
d. SLCS Initiation	N.A.
e. Reactor Vessel Water Level - Low Low, Level 2	$\leq 13^{(a)}$
4. REACTOR CORE ISOLATION COOLING SYSTEM ISOLATION	
a. RCIC Steam Line Flow - High	$\leq 13^{(a)\#\#\#}$
b. RCIC Steam Supply Pressure - Low	$\leq 13^{(a)}$
c. RCIC Turbine Exhaust Diaphragm Pressure - High	N.A.
d. RCIC Equipment Room Temperature - High	N.A.
e. RCIC Steam Line Tunnel Temperature - High	N.A.
f. RCIC Steam Line Tunnel ΔTemperature - High	N.A.
g. Drywell Pressure - High	N.A.
5. RHR SYSTEM STEAM CONDENSING MODE ISOLATION	
a. RHR Equipment Area ΔTemperature - High	N.A.
b. RHR Area Cooler Temperature - High	N.A.
c. RHR Heat Exchanger Steam Supply Flow High	N.A.

TABLE 4.3.2.1-1

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a. Reactor Vessel Water Level				
1) Low, Level 3	S	M	R	1, 2, 3
2) Low Low, Level 2	S	M	R	1, 2, 3
b. Drywell Pressure - High	NA	M	Q	1, 2, 3
c. Main Steam Line				
1) Radiation - High	S	M	R	1, 2, 3
2) Pressure - Low	NA	M	Q	1
3) Flow - High	S	M	R	1, 2, 3
d. Main Steam Line Tunnel Temperature - High	NA	M	R	1, 2, 3
e. Condenser Vacuum - Low	NA	M	Q	1, 2*, 3*
f. Main Steam Line Tunnel Δ Temperature - High	NA	M	R	1, 2, 3
2. SECONDARY CONTAINMENT ISOLATION				
a. Reactor Building Vent Exhaust Plenum Radiation - High	S	M	R	1, 2, 3 and **
b. Drywell Pressure - High	NA	M	Q	1, 2, 3
c. Reactor Vessel Water Level - Low Low, Level 2	S	M	R	1, 2, 3, and #
d. Fuel Pool Vent Exhaust Radiation - High	S	M	R	1, 2, 3 and **
3. REACTOR WATER CLEANUP SYSTEM ISOLATION				
a. Δ Flow - High	S	M	R	1, 2, 3
b. Heat Exchanger Area Temperature - High	NA	M	Q	1, 2, 3
c. Heat Exchanger Area Ventilation ΔT - High	NA	M	Q	1, 2, 3
d. SLCS Initiation	NA	R	NA	1, 2, 3
e. Reactor Vessel Water Level - Low Low, Level 2	S	M	R	1, 2, 3

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION
AMENDMENT NO. 20 TO NPF-11 AND
AMENDMENT NO. 7 TO NPF-18
LA SALLE COUNTY STATION, UNITS 1 & 2
DOCKET NOS. 50-373 AND 50-374

Introduction

By letter dated September 19, 1984 and as modified by letter dated October 5, 1984, Commonwealth Edison Company (the licensee) proposed amendments requesting changes to the La Salle Units 1 and 2 Technical Specifications consistent with a design change in the location of the reactor water cleanup (RWCU) pumps to a point in the system containing lower water temperature. The Technical Specifications changes would eliminate the requirement to specify limits on the ambient and differential temperature measurements in the RWCU pumps in Tables 3.3.2-1, 3.3.2-2, 3.3.2-3, and 4.3.2-1.

Evaluation

In the original RWCU system design, the RWCU recirculation pumps utilized water at full operating temperature and pressure. For better pump operation based upon experience at other plants, the licensee changed the system design to locate these pumps downstream of the system heat exchangers so that pumps would utilize lower temperature water. In the original RWCU system design, the pump room leak detection system was designed for hot water which included the following instrumentation: (1) system high differential flow; (2) RWCU pump rooms (three rooms) high ambient and high differential temperature; and (3) reactor building sump high level. Items (1) and (2) provide automatic isolation signals to the RWCU system, and item (3) provides a remote alarm to the control room to allow the operator to manually isolate the RWCU system. After the RWCU system configuration change, the RWCU pumps pump low temperature water and the ambient and differential temperature monitors are no longer effective. The setpoints for the instruments are adjusted to detect the equivalent leakage limit and are very near the normal operating conditions due to the lower temperature. However, the automatic isolation provisions based on temperature were not removed from the design and were installed in the plants and included in the plants' Technical Specifications. Consequently, the licensee indicates that this has caused unnecessary spurious isolation, when no leaks were present. Sufficient diversity remains in the differential flow and reactor low water level automatic isolation to ensure that actual RWCU leakage in the pump rooms is monitored and will be promptly isolated. In addition, a ruptured RWCU system can be detected and isolated manually if area sump pumps indicate leakage in the system. Therefore, the deletion from the Technical Specification of high ambient and differential temperature isolation in the RWCU pump rooms do not involve an unreviewed safety matter because the RWCU system will still have adequate isolation capability and the leakage from the RWCU system has already been evaluated. It also does not involve a significant reduction in the margin of safety because the RWCU system will still be monitored for leakage and will still isolate if the leakage exceeds the required limits. This change deletes only temperature monitoring of the colder portion of the RWCU system where this type of monitoring is not very effective and causes unnecessary isolations.

Based on the above, the NRC staff concludes that the proposed change for deleting the pump room high ambient and high differential temperature isolation from the La Salle Unit 1 and Unit 2 Technical Specifications will not increase the consequence of previously evaluated accidents or decrease the margin of safety. The proposed changes reflect the as built design of the RWCU system; and are, therefore, acceptable.

Environmental Consideration

This amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR Sec 51.22(c)(9). Pursuant to 10 CFR 51.2(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

Conclusion

The Commission made a proposed determination that the amendment involves no significant hazards consideration which was published in the Federal Register (49 FR45946) on November 21, 1984. No public comments were received.

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: JAN 0 8 1985