

OCT 1 1985

Docket Nos. 50-373/374

Mr. Dennis L. Farrar  
Director of Nuclear Licensing  
Commonwealth Edison Company  
Post Office Box 767  
Chicago, Illinois 60690

Dear Mr. Farrar:

SUBJECT: ISSUANCE OF AMENDMENT 25 TO FACILITY OPERATING LICENSE NO. NPF-11  
AND AMENDMENT 13 TO FACILITY OPERATING LICENSE NO. NPF-18 LA SALLE  
COUNTY STATION, UNITS 1 AND 2

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 25 to Facility Operating License No. NPF-11 and Amendment No. 13 to Facility Operating License No. NPF-18. These amendments are in response to your letter dated September 23, 1985. These amendments were authorized by telephone on September 26, 1985, and was confirmed by letter dated September 26, 1985. The amendments would revise the Specification 4.5.1.c for La Salle, Unit 1 Low Pressure Coolant Injection and Specification 4.8.1.1.2.d for La Salle, Unit 2 Diesel Generator 1A, which is shared with Unit 1, to extend on a one-time-only waiver the 18-month interval surveillance requirement.

A copy of the related safety evaluation supporting Amendment No. 25 to Facility Operating License NPF-11 and Amendment No. 13 to Facility Operating License NPF-18 is enclosed.

Sincerely,

Walter R. Butler, Chief  
Licensing Branch No. 2  
Division of Licensing

Enclosures:

1. Amendment No. 25 to NPF-11
2. Amendment No. 13 to NPF-18
3. Safety Evaluation

cc w/enclosures:  
See next page

DISTRIBUTION  
See next page

*as*  
LB#2/DL/PM  
ABourna:lb  
09/26/85

*DL*  
LB#2/DL/LA  
EM/Don  
09/29/85

*Wally Hange*  
*Woodhead*  
OELD  
09/21/85

*WB*  
LB#2/DL/BC  
WButler  
09/26/85

8510080126 851001  
PDR ADOCK 05000373  
P PDR

DISTRIBUTION

Docket File  
NRC PDR  
Local PDR  
PRC System  
NSIC  
LB#2 Reading  
EHylton (2)  
ABourria (4)  
TNovak  
JSaltzman, SAB  
OELD, Woodhead  
CMiles  
HDenton  
JRutberg  
AToalston  
WMiller, LFMB  
JPartlow  
BGrimes  
EJordan  
LHarmon  
TBarnhart (8)



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

Docket Nos. 50-373/374

OCT 1 1985

Mr. Dennis L. Farrar  
Director of Nuclear Licensing  
Commonwealth Edison Company  
Post Office Box 767  
Chicago, Illinois 60690

Dear Mr. Farrar:

SUBJECT: ISSUANCE OF AMENDMENT 25 TO FACILITY OPERATING LICENSE NO. NPF-11  
AND AMENDMENT 13 TO FACILITY OPERATING LICENSE NO. NPF-18 LA SALLE  
COUNTY STATION, UNITS 1 AND 2

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 25 to Facility Operating License No. NPF-11 and Amendment No. 13 to Facility Operating License No. NPF-18. These amendments are in response to your letter dated September 23, 1985. These amendments were authorized by telephone on September 26, 1985, and was confirmed by letter dated September 26, 1985. The amendments would revise the Specification 4.5.1.c for La Salle, Unit 1 Low Pressure Coolant Injection and Specification 4.8.1.1.2.d for La Salle, Unit 2 Diesel Generator 1A, which is shared with Unit 1, to extend on a one-time-only waiver the 18-month interval surveillance requirement.

A copy of the related safety evaluation supporting Amendment No. 25 to Facility Operating License NPF-11 and Amendment No. 13 to Facility Operating License NPF-18 is enclosed.

Sincerely,

Walter R. Butler, Chief  
Licensing Branch No. 2  
Division of Licensing

Enclosures:

1. Amendment No. 25 to NPF-11
2. Amendment No. 13 to NPF-18
3. Safety Evaluation

cc w/enclosures:  
See next page

Mr. Dennis L. Farrar  
Commonwealth Edison Company

La Salle County Nuclear Power Station  
Units 1 & 2

cc:

Philip P. Steptoe, Esquire  
Suite 4200  
One First National Plaza  
Chicago, Illinois 60603

John W. McCaffrey  
Chief, Public Utilities Division  
160 North La Salle Street, Room 900  
Chicago, Illinois 60601

Assistant Attorney General  
188 West Randolph Street  
Suite 2315  
Chicago, Illinois 60601

Resident Inspector/LaSalle, NPS  
U.S. Nuclear Regulatory Commission  
Rural Route No. 1  
Post Office Box 224  
Marseilles, Illinois 61341

Chairman  
La Salle County Board of Supervisors  
La Salle County Courthouse  
Ottawa, Illinois 61350

Attorney General  
500 South 2nd Street  
Springfield, Illinois 62701

Chairman  
Illinois Commerce Commission  
Leland Building  
527 East Capitol Avenue  
Springfield, Illinois 62706

Mr. Gary N. Wright, Manager  
Nuclear Facility Safety  
Illinois Department of Nuclear Safety  
1035 Outer Park Drive, 5th Floor  
Springfield, Illinois 62704

Regional Administrator, Region III  
U. S. Nuclear Regulatory Commission  
799 Roosevelt Road  
Glen Ellyn, Illinois 60137



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-373

LA SALLE COUNTY STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 25  
License No. NPF-11

1. The Nuclear Regulatory Commission (the Commission or the NRC) having found that:
  - A. The application for amendment filed by the Commonwealth Edison Company, dated September 23, 1985, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
  - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. NPF-11 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 25 and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

8510080133 851001  
PDR ADDCK 05000373  
P PDR

3. This amendment is effective as of September 26, 1985.

FOR THE NUCLEAR REGULATORY COMMISSION



Walter R. Butler, Chief  
Licensing Branch No. 2  
Division of Licensing

Enclosure:  
Changes to the Technical  
Specifications

Date of Issuance: October 1, 1985

ENCLOSURE TO LICENSE AMENDMENT NO. 25  
FACILITY OPERATING LICENSE NO. NPF-11  
DOCKET NO. 50-373

Replace the following page of the Appendix "A" Technical Specifications with the enclosed page. The revised page is identified by Amendment number and contain a vertical line indicating the area of change.

REMOVE

3/4 5-4

INSERT

3/4 5-4

## EMERGENCY CORE COOLING SYSTEMS

### SURVEILLANCE REQUIREMENTS

---

4.5.1 ECCS divisions 1, 2 and 3 shall be demonstrated OPERABLE by:

- a. At least once per 31 days for the LPCS, LPCI and HPCS systems:
  1. Verifying by venting at the high point vents that the system piping from the pump discharge valve to the system isolation valve is filled with water.
  2. Performance of a CHANNEL FUNCTIONAL TEST of the:
    - a) Discharge line "keep filled" pressure alarm instrumentation, and
    - b) Header delta P instrumentation.
  3. Verifying that each valve, manual, power operated or automatic, in the flow path that is not locked, sealed, or otherwise secured in position, is in its correct position.
  4. Verifying that each ECCS corner room watertight door is closed, except during normal entry and exit from the room.
- b. Verifying that, when tested pursuant to Specification 4.0.5, each:
  1. LPCS pump develops a flow of at least 6350 gpm against a test line pressure greater than or equal to 290 psig.
  2. LPCI pump develops a flow of at least 7200 gpm against a test line pressure greater than or equal to 130 psig.
  3. HPCS pump develops a flow of at least 6250 gpm against a test line pressure greater than or equal to 370 psig.
- c. For the LPCS, LPCI\* and HPCS systems, at least once per 18 months:
  1. Performing a system functional test which includes simulated automatic actuation of the system throughout its emergency operating sequence and verifying that each automatic valve in the flow path actuates to its correct position. Actual injection of coolant into the reactor vessel may be excluded from this test.
  2. Performing a CHANNEL CALIBRATION of the:
    - a) Discharge line "keep filled" pressure alarm instrumentation and verifying the:
      - 1) High pressure setpoint and the low pressure setpoint of the:

\*The specified 18 month interval may be waived for Cycle 1 provided the surveillance is performed during Refuel 1.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-374

LA SALLE COUNTY STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 13  
License No. NPF-18

1. The Nuclear Regulatory Commission (the Commission or the NRC) having found that:
  - A. The application for amendment filed by the Commonwealth Edison Company, September 23, 1985, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
  - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. 18 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 13, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This amendment is effective as of September 26, 1985.

FOR THE NUCLEAR REGULATORY COMMISSION

*Walter R. Butler*

Walter R. Butler, Chief  
Licensing Branch No. 2  
Division of Licensing

Enclosure:  
Changes to the Technical  
Specifications

Date of Issuance: October 1, 1985

ENCLOSURE TO LICENSE AMENDMENT NO. 13  
FACILITY OPERATING LICENSE NO. NPF-18  
DOCKET NO. 50-374

Replace the following page of the Appendix "A" Technical Specifications with the enclosed page. The revised page is identified by Amendment number and contain a vertical line indicating the area of change.

REMOVE

3/4 8-4

INSERT

3/4 8-4

## ELECTRICAL POWER SYSTEMS

### SURVEILLANCE REQUIREMENTS (Continued)

---

- d. At least once per 18 months\* during shutdown by:
1. Subjecting the diesel to an inspection in accordance with procedures prepared in conjunction with its manufacturer's recommendations for this class of standby service.
  2. Verifying the diesel generator capability to reject a load of greater than or equal to 1190 kW for diesel generator 0, greater than or equal to 638 kW for diesel generators 1A and 2A, and greater than or equal to 2381 kW for diesel generator 2B while maintaining engine speed less than or equal to 75% of the difference between nominal speed and the overspeed trip setpoint or 15% above nominal, whichever is less.
  3. Verifying the diesel generator capability to reject a load of 2600 kW without tripping. The generator voltage shall not exceed 5000 volts during and following the load rejection.
  4. Simulating a loss-of-offsite power by itself, and:
    - a) For Divisions 1 and 2 and for Unit 1 Division 2:
      - 1) Verifying deenergization of the emergency busses and load shedding from the emergency busses.
      - 2) Verifying the diesel generator starts on the auto-start signal, energizes the emergency busses with permanently connected loads within 13 seconds, energizes the auto-connected loads and operates for greater than or equal to 5 minutes while its generator is so loaded. After energization, the steady-state voltage and frequency of the emergency busses shall be maintained at  $4160 \pm 150$  volts and  $60 \pm 1.2$  Hz during this test.
    - b) For Division 3:
      - 1) Verifying deenergization of the emergency bus.
      - 2) Verifying the diesel generator starts on the auto-start signal, energizes the emergency bus with its loads within 13 seconds and operates for greater than or equal to 5 minutes while its generator is so loaded. After energization, the steady-state voltage and frequency of the emergency bus shall be maintained at  $4160 \pm 150$  volts and  $60 \pm 1.2$  Hz during this test.
  5. Verifying that on an ECCS actuation test signal, without loss-of-offsite power, diesel generators 0, 2A, and 2B start on the auto-start signal and operate on standby for greater than or equal to 5 minutes. The generator voltage and frequency shall be  $4160 + 416, -150$  volts and  $60 + 3.0, -1.2$  Hz within 13 seconds after the auto-start signal; the steady-state generator voltage and frequency shall be maintained within these limits during this test.

\*The specified 18 month interval may be waived for Unit 1, Cycle 1, provided the surveillance is performed during Unit 1, Refuel 1.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION

AMENDMENT NO. 25 TO NPF-11

AND

AMENDMENT NO. 13 TO NPF-18

LA SALLE COUNTY STATION, UNITS 1 AND 2

DOCKET NO. 50-373

Introduction

By letter dated September 23, 1985, Commonwealth Edison Company (the licensee) requested emergency changes to the La Salle, Units 1 and 2 Technical Specifications. The proposed changes would revise the Technical Specifications. The proposed changes would revise the Technical Specifications to temporarily extend additional Specifications which were inadvertently omitted from the original submittal dated July 15, 1985 and supplemented by letters dated August 9 and 12, 1985. In the original submittal, the licensee requested a one-time-only extension of a limited number of 18-month interval surveillance requirements which required a unit shutdown for La Salle, Unit 1 for first refuel cycle. This would extend the 18-month surveillances 36 days beyond the maximum authorized 25 percent extension and would permit the licensee to delay performing this testing until the refueling outage, currently scheduled to commence on or before October 27, 1985. In conjunction with its efforts to prepare for the upcoming outage, the licensee determined that while all the surveillances which needed to be deferred were included in the original submittals two of the corresponding Technical Specification references were inadvertently omitted. This was discovered on September 18, 1985 by the licensee and discussed in person with the staff on September 20, 1985. Although the license amendment associated with the original request was issued on September 20, 1985, it was determined at that time that the two additional changes in the instant request did not qualify as "administrative" changes and could not be included with the original amendment issued on September 20, 1985.

Evaluation

In the original submittal, the request was made (and approved by Amendment 24 to La Salle, Unit 1) that Specification 4.3.3.2 pertaining to testing of the low pressure coolant injection system logic would be waived for 36 days. However, Specification 4.5.1.c was not included because it was inadvertently omitted. The surveillance requirement of Specification 4.3.3.2 is conducted along with Specification 4.5.1.c for the low pressure coolant injection system function. The logic system functional test for the low pressure coolant injection system is performed by surveillance procedure LES-RH-01 which satisfies both Specifications 4.3.3.2 and 4.5.1.c. Therefore, delaying performance of Specification 4.5.1.c has no additional significance for plant safety beyond that evaluated in Amendment 24.

8510080138 851001  
PDR ADOCK 05000373  
P PDR

Technical Specification 4.8.1.1.2.d requires testing at 18 month intervals to demonstrate the ability of the diesel generator system to supply sufficient emergency power to the unit to which it is assigned. The testing requirement of diesel generator 1A was deferred for Unit 1 by Amendment 24 to La Salle, Unit 1. However, diesel generator 1A 18 month testing is also required by the Unit 2 Technical Specifications because diesel 1A is shared by the two units. The Unit 2 Specification 4.8.1.1.2.d was inadvertently omitted from the original submittal requesting a waiver of the 18 month surveillance. Therefore, since the diesel generator will be operable for Unit 1 operation during the extension period granted, it should be operable for Unit 2 operation for the same period since the Unit 2 surveillance requirements are identical to those of Unit 1. Therefore, since the same diesel generator is affected, no additional impact will occur.

### Emergency Circumstances

The proposed changes fall into the category of emergency changes since La Salle, Unit 1 will have to be shutdown unless the NRC takes the action to approve the proposed changes. Without the proposed amendments the following will occur:

- (1) La Salle, Unit 1 cannot continue operation past September 27, 1985, due to surveillance requirement 4.5.1.c for the low pressure coolant injection system, which requires testing during a unit shutdown;
- (2) La Salle, Unit 1 will be forced to shutdown no later than September 27, 1985, to perform the surveillance tests required by specification 4.8.1.1.2.d for the shared diesel generator 1A.

We have reviewed the facts concerning this request and conclude that the licensee has made a timely submittal, that reactor shutdown will occur without NRC action and that although the licensee was remiss in that he filed an incomplete request in the initial submittal, that was inadvertent and not for the purpose of avoiding the normal notice requirements.

### Final No Significant Hazards Consideration Determination

The Commission has provided certain examples (48 FR 14870) of actions likely to involve no significant hazards considerations. Based on the review of the licensee's submittal as described herein, the staff has made a final determination that the licensee's amendment requests do not involve a significant hazards consideration since the operation of La Salle County Station, Units 1 and 2 with the requested changes would not (1) involve a significant increase in the probability or consequences of an accident previously evaluated because the operability of the equipment is still maintained by other Technical Specifications and based on the type of surveillances extended, no significant increase in the probability of equipment failure is postulated; (2) create the possibility of a

new or different kind of accident from any accident previously evaluated because these amendments do not remove or add any equipment nor do they eliminate tests required at refueling outages; and (3) involve a significant reduction in the margin of safety because the increased surveillance interval (30 days) does not significantly increase the possibility that an undetected failure will occur in any of the related equipment covered by these Technical Specifications. Accordingly, the staff has made a final determination that these license amendments involve no significant hazards consideration.

#### Staff Consultation

In accordance with the Commission's regulation, consultation was held with the State of Illinois by telephone on September 23, 1985. The State had no comments on this action.

#### Environmental Consideration

These amendments extend the interval for some surveillance requirements on a one-time-only basis. We have determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has made a final determination that these amendments involve no significant hazards considerations. Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

#### Conclusion

The staff has concluded, based on the considerations discussed above, that: (1) these license amendments involve no significant hazards considerations; (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner; and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Dated: October 1, 1985

DISTRIBUTION

Docket File

NRC PDR

Local PDR

PRC System

NSIC

LB#2 Reading

EHylton (2)

ABournia (4)

TNovak

JSaltzman, SAB

OELD, Woodhead

CMiles

HDenton

JRutberg

AToalston

WMiller, LFMB

JPartlow

BGrimes

EJordan

LHarmon

TBarnhart (8)

3. This amendment is effective as of September 26, 1985.

FOR THE NUCLEAR REGULATORY COMMISSION

Walter R. Butler, Chief  
Licensing Branch No. 2  
Division of Licensing

Enclosure:  
Changes to the Technical  
Specifications

Date of Issuance: October 1, 1985

LB#2/DL/LA  
Eaton  
09/26/85

LB#2/DL/PM  
ABournia:lb  
09/21/85

CEL  
CWoodhead  
09/27/85

LB#2/DL/BC  
WRButler  
09/26/85

AD/DL  
TMNovak  
09/26/85

3. This amendment is effective as of September 26, 1985.

FOR THE NUCLEAR REGULATORY COMMISSION

Walter R. Butler, Chief  
Licensing Branch No. 2  
Division of Licensing

Enclosure:  
Changes to the Technical  
Specifications

Date of Issuance: October 1, 1985

*[Signature]*  
LB#2/DL/LA  
Elyton  
09/26/85

*AB*  
LB#2/DL/PM  
ABournia:lb  
09/26/85

*w/SE change*  
*Woodhead*  
OELD  
CWoodhead  
09/27/85

*WB*  
LB#2/DL/BC  
WRButler  
09/26/85

*[Signature]*  
AD/DL  
TMNovak  
09/26/85