



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

Docket Nos. 50-373/374

9861 18 100

Mr. Dennis L. Farrar
Director of Licensing
Commonwealth Edison Company
P.O. Box 767
Chicago, Illinois 60690

Dear Mr. Farrar:

SUBJECT: ISSUANCE OF AMENDMENT NO. 28 TO FACILITY OPERATING LICENSE
NO. NPF-11 AND AMENDMENT NO. 15 TO FACILITY OPERATING
LICENSE NO. NPF 18 - LA SALLE COUNTY STATION, UNITS 1 AND 2

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 28 to Facility Operating License No. NPF-11 and Amendment No. 15 to Facility Operating License NPF-18 for the La Salle County Station, Units 1 and 2. These amendments are in response to your letter dated August 19, 1985, as supplemented by letter dated September 5, 1985. The amendments revise the setpoints and setpoint tolerances for the 18 safety-relief valves in the Technical Specifications for each of La Salle, Units 1 and 2.

A copy of the related safety evaluation supporting Amendment No. 28 to Facility Operating License NPF-11 and Amendment No. 15 to Facility Operating License No. NPF-18 is enclosed.

Sincerely,

Walter R. Butler, Chief
Licensing Branch No. 2
Division of Licensing

Enclosures:

1. Amendment No. 28 to NPF-11
2. Amendment No. 15 to NPF-18
3. Safety Evaluation

cc w/enclosure:
See next page

8511070387 851031
PDR ADDCK 05000373
P PDR

Mr. Dennis L. Farrar
Commonwealth Edison Company

La Salle County Nuclear Power Station
Units 1 & 2

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Regional Administrator, Region III
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Glen Ellyn, Illinois 60137

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Docket Nos. 50-373/374

OCT 31 1985

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Commonwealth Edison Company
P.O. Box 767
Chicago, Illinois 60690

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Division of Licensing

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3. Safety Evaluation

cc w/enclosure:
See next page

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LB#2/DL/LA
EHL:lon
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053
LB#2/DL/PM
ABournia:lb
10/21/85

With changes noted
in check w/ SECY
for NPF-18
OELD:JPW
Woodhead
10/23/85

LB#2/DL/BC
WRButler
10/22/85

WB



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-373

LA SALLE COUNTY STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 28
License No. NPF-11

1. The Nuclear Regulatory Commission (the Commission or the NRC) having found that:
 - A. The application for amendment filed by the Commonwealth Edison Company, dated August 19, 1985 as supplemented by letter dated September 5, 1985, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. NPF-11 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 28 and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

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PDR ADOCK 05000373
P PDR

3. This amendment is effective as of date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

A handwritten signature in cursive script, reading "Walter R. Butler".

Walter R. Butler, Chief
Licensing Branch No. 2
Division of Licensing

Enclosure:
Changes to the Technical
Specifications

Date of Issuance: October 31, 1985

3. This amendment is effective as of date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Walter R. Butler, Chief
Licensing Branch No. 2
Division of Licensing

Enclosure:
Changes to the Technical
Specifications

Date of Issuance: October 31, 1985

LB#2/DL/LA
EMyton
10/2/85

LB#2/DL/PM
ABournia:lb
10/24/85

W/changes noted to
SE + FRN;
check w/
SECT for
hearing
requests
OELD
CWoodhead
10/23/85

LB
LB#2/DL/BC
WRButler
10/22/85

for LB
AD/L/DL
TMNovak
10/31/85

ENCLOSURE TO LICENSE AMENDMENT NO. 28
FACILITY OPERATING LICENSE NO. NPF-11
DOCKET NO. 50-373

Replace the following page of the Appendix "A" Technical Specifications with the enclosed page. The revised page is identified by Amendment number and contains a vertical line indicating the area of change.

REMOVE

3/4 4-5

INSERT

3/4 4-5

REACTOR COOLANT SYSTEM

3/4.4.2 SAFETY/RELIEF VALVES

LIMITING CONDITION FOR OPERATION

3.4.2 The safety valve function of eighteen reactor coolant system safety/relief valves shall be OPERABLE with the specified code safety valve function lift settings.*#

- a. 4 safety/relief valves @ 1205 psig +1%, -3%
- b. 4 safety/relief valves @ 1195 psig +1%, -3%
- c. 4 safety/relief valves @ 1185 psig +1%, -3%
- d. 4 safety/relief valves @ 1175 psig +1%, -3%
- e. 2 safety/relief valves @ 1150 psig +1%, -3%

APPLICABILITY: OPERATIONAL CONDITIONS 1, 2 and 3.

ACTION:

- a. With the safety valve function of one or more of the above required safety/relief valves inoperable, be in at least HOT SHUTDOWN within 12 hours and in COLD SHUTDOWN within the next 24 hours.
- b. With one or more safety/relief valves stuck open, provided that suppression pool average water temperature is less than 110°F, close the stuck open relief valve(s); if unable to close the open valve(s) within 2 minutes or if suppression pool average water temperature is 110°F or greater, place the reactor mode switch in the Shutdown position.
- c. With one or more safety/relief valve stem position indicators inoperable, restore the inoperable stem position indicators to OPERABLE status within 7 days or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.

SURVEILLANCE REQUIREMENTS

4.4.2.1 The safety/relief valve stem position indicators of each safety/relief valve shall be demonstrated OPERABLE by performance of a:

- a. CHANNEL CHECK at least once per 31 days, and a
- b. CHANNEL CALIBRATION at least once per 18 months.**

4.4.2.2 The low low set function shall be demonstrated not to interfere with the OPERABILITY of the safety relief valves or the ADS by performance of a CHANNEL CALIBRATION at least once per 18 months.

*The lift setting pressure shall correspond to ambient conditions of the valves at nominal operating temperatures and pressures.

#Up to two inoperable valves may be replaced with spare OPERABLE valves with lower setpoints until the next refueling outage.

**The provisions of Specification 4.0.4 are not applicable provided the surveillance is performed within 12 hours after reactor steam pressure is adequate to perform the test.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-374

LA SALLE COUNTY STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 15
License No. NPF-18

1. The Nuclear Regulatory Commission (the Commission or the NRC) having found that:
 - A. The application for amendment filed by the Commonwealth Edison Company, dated August 19, 1985 as supplemented by letter dated September 5, 1985, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. NPF 18 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 15 and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This amendment is effective as of date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

A handwritten signature in cursive script, reading "Walter R. Butler".

Walter R. Butler, Chief
Licensing Branch No. 2
Division of Licensing

Enclosure:
Changes to the Technical
Specifications

Date of Issuance: October 31, 1985

3. This amendment is effective as of date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Walter R. Butler, Chief
Licensing Branch No. 2
Division of Licensing

Enclosure:
Changes to the Technical
Specifications

Date of Issuance: October 31, 1985

LB#2/DL/LA
EJW/ton
10/31/85

LB#2/DL/PM
ABournia:lb
10/24/85

W/changes noted
to SE + FR
OELD
CWoodhead
10/23/85

LB#2/DL/BC
WRButler
10/22/85

AD/L/DL
TMNovak
10/31/85

ENCLOSURE TO LICENSE AMENDMENT NO. 15
FACILITY OPERATING LICENSE NO. NPF-18
DOCKET NO. 50-374

Replace the following page of the Appendix "A" Technical Specifications with the enclosed page. The revised page is identified by Amendment number and contains a vertical line indicating the area of change.

REMOVE

3/4 4-6

INSERT

3/4 4-6

REACTOR COOLANT SYSTEM

3/4.4.2 SAFETY/RELIEF VALVES

LIMITING CONDITION FOR OPERATION

3.4.2 The safety valve function of 18 reactor coolant system safety/relief valves shall be OPERABLE with the specified code safety valve function lift settings.* #

- a. 4 safety/relief valves @ 1205 psig + 1%, -3%
- b. 4 safety/relief valves @ 1195 psig + 1%, -3%
- c. 4 safety/relief valves @ 1185 psig + 1%, -3%
- d. 4 safety/relief valves @ 1175 psig + 1%, -3%
- e. 2 safety/relief valves @ 1150 psig + 1%, -3%

APPLICABILITY: OPERATIONAL CONDITIONS 1, 2, and 3.

ACTION:

- a. With the safety valve function of one or more of the above required safety/relief valves inoperable, be in at least HOT SHUTDOWN within 12 hours and in COLD SHUTDOWN within the next 24 hours.
- b. With one or more safety/relief valves stuck open, provided that suppression pool average water temperature is less than 110°F, close the stuck open relief valve(s); if unable to close the open valve(s) within 2 minutes or if suppression pool average water temperature is 110°F or greater, place the reactor mode switch in the Shutdown position.
- c. With one or more safety/relief valve stem position indicators inoperable, restore the inoperable stem position indicators to OPERABLE status within 7 days or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.

SURVEILLANCE REQUIREMENTS

4.4.2.1 The safety/relief valve stem position indicators of each safety/relief valve shall be demonstrated OPERABLE by performance of a:

- a. CHANNEL CHECK at least once per 31 days, and a
- b. CHANNEL CALIBRATION at least once per 18 months.**

4.4.2.2 The low low set function shall be demonstrated not to interfere with the OPERABILITY of the safety/relief valves or the ADS by performance of a CHANNEL CALIBRATION at least once per 18 months.

*The lift setting pressure shall correspond to ambient conditions of the valves at nominal operating temperatures and pressures.

#Up to two inoperable valves may be replaced with spare OPERABLE valves with lower setpoints until the next refueling outage.

**The provisions of Specification 4.0.4 are not applicable provided the surveillance is performed within 12 hours after reactor steam pressure is adequate to perform the test.



UNITED STATES
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WASHINGTON, D. C. 20555

SAFETY EVALUATION

AMENDMENT NO. 28 TO NPF-11

AND

AMENDMENT NO. 15 TO NPF-18

LA SALLE COUNTY STATION, UNITS 1 AND 2

DOCKET NOS. 50-373 AND 50-374

Introduction

By letter dated August 19, 1985 as supplemented by letter dated September 5, 1985, Commonwealth Edison Company (the licensee) requested amendments to the La Salle, Units 1 and 2 Technical Specifications. The proposed changes are to revise the setpoints and setpoint tolerances for the 18 safety-relief valves (SRV) in the La Salle Units 1 and 2 Technical Specifications.

Evaluation

The La Salle County Station, Units 1 and 2 each uses 18 Crosby safety relief valves for their respective overpressure protection. The Crosby safety relief valves have a setpoint opening scatter band range of 4 percent which was measured during the life cycle qualification testing. The present setpoint tolerance in the Technical Specifications is ± 1 percent. The licensee indicates this limit is unduly restrictive and requires unnecessary testing of the valves. Consequently, the licensee is requesting that the Technical Specifications be revised to use the scatter band of the qualification test for the setpoint tolerances of +1 percent and -3 percent.

The licensee is also proposing that the setpoints for the last 2 safety relief valves be changed from 1146 to 1150 psig to be consistent with the General Electric design documents. Presently, the setpoints for these safety relief valves are as follows: 4 at 1205 psig, 4 at 1195 psig, 4 at 1185 psig, 4 at 1175 psig, and 2 at 1146 psig.

Overpressure protection evaluation considers only the maximum or high value of the opening pressures, which will be maintained by the proposed upper tolerance limit of 1 percent. The plant operating pressure is 1020 psig. The lower tolerance limit of -3 percent would actuate a safety relief valve for the minimum setting at about 1115 psig during the plant operation. This lower tolerance provides adequate simmer margin which will reduce the potential for spurious openings. This simmer margin is the difference between the safety relief valve set pressure and the reactor pressure vessel operating pressure.

The General Electric (the vendor) specification on blowdown requires that the blowdown be within the range of 2 to 11 percent of the nameplate permitted set pressure popping range. The proposed tolerance limit would be within the blowdown limit range.

The proposed changes do not affect overprotection analyses or plant performance; and, therefore, the staff finds the proposed changes acceptable.

Environmental Consideration

These amendments involve a change in use of facility components located within the restricted area as defined in 10 CFR Part 20 and changes surveillance requirements. The staff has determined that these amendments involve no significant increase in the amounts, and no significant changes in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that these amendments involve no significant hazards consideration and there has been no public comment on such finding. Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

Conclusion

The Commission made a proposed determination that these amendments involve no significant hazards consideration which were published in the Federal Register (50 FR 38912) on September 25, 1985. No public comments were received.

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Dated: October 31, 1985