UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SEP 20 1985

Docket No. 50-373

Mr. Dennis L. Farrar
Director of Licensing
Commonwealth Edison Company
P.O. Box 767
Chicago, Illinois 60690

Dear Mr. Farrar:

SUBJECT: ISSUANCE OF AMENDMENT NO. 24 TO FACILITY OPERATING LICENSE NO. NPF-11 - LA SALLE COUNTY STATION, UNIT 1

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 24 to Facility Operating License No. NPF-11 for the La Salle County Station, Unit 1. This amendment is in response to your letter dated July 15, 1985, as supplemented by letters dated August 9 and 12, 1985. The amendment would extend on a one-time-only basis a limited number of the surveillance requirements in the Technical Specifications which must be performed every 18 months and which can only be done when the plant is shutdown. Your reason for this extension is that La Salle has been through an extended startup program and has been shutdown for various reasons over the past months, and the core has not been fully utilized. Therefore, you have deferred your refueling outage from September 22, 1985 to October 27, 1985.

A copy of the related safety evaluation supporting Amendment No. 24 to Facility Operating License NPF-11 is enclosed.

Sincerely,

Walter R. Butler

Walter R. Butler, Chief
Licensing Branch No. 2
Division of Licensing

Enclosures:
1. Amendment No. 24 to NPF-11
2. Safety Evaluation

cc w/enclosures:
See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-373

LA SALLE COUNTY STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 24
License No. NPF-11

1. The Nuclear Regulatory Commission (the Commission or the NRC) having found that:

   A. The application for amendment filed by the Commonwealth Edison Company, dated July 15, 1985 and supplemented by letters dated August 9 and 12, 1985, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;

   C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. NPF-11 is hereby amended to read as follows:

   (2) Technical Specifications and Environmental Protection Plan

   The Technical Specifications contained in Appendix A, as revised through Amendment No. 24 and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Walter R. Butler

Walter R. Butler, Chief
Licensing Branch No. 2
Division of Licensing

Enclosure:
Changes to the Technical
Specifications

Date of Issuance: SEP 20 1985

ENCLOSURE TO LICENSE AMENDMENT NO. 24
FACILITY OPERATING LICENSE NO. NPF-11
DOCKET NO. 50-373

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain a vertical line indicating the area of change.

REMOVE	INSERT
3/4 3-1	3/4 3-1
3/4 3-8	3/4 3-8
3/4 3-23	3/4 3-23
3/4 3-32	3/4 3-32
3/4 3-33	3/4 3-33
3/4 3-34	3/4 3-34
3/4 3-35	3/4 3-35
3/4 3-89	3/4 3-89
3/4 3-90	3/4 3-90
3/4 3-92	3/4 3-92
3/4 4-6	3/4 4-6
3/4 4-9	3/4 4-9
3/4 6-7	3/4 6-7
3/4 6-34	3/4 6-34
3/4 8-4	3/4 8-4
3/4 8-17	3/4 8-17
3/4 8-28	3/4 8-28
3/4 8-29	3/4 8-29

3/4.3 INSTRUMENTATION

3/4.3.1 REACTOR PROTECTION SYSTEM INSTRUMENTATION

LIMITING CONDITION FOR OPERATION

3.3.1 As a minimum, the reactor protection system instrumentation channels shown in Table 3.3.1-1 shall be OPERABLE with the REACTOR PROTECTION SYSTEM RESPONSE TIME as shown in Table 3.3.1-2.

APPLICABILITY: As shown in Table 3.3.1-1.

ACTION:

a. With the number of OPERABLE channels less than required by the Minimum OPERABLE Channels per Trip System requirement for one trip system, place that trip system in the tripped condition* within 1 hour. The provisions of Specification 3.0.4 are not applicable.

b. With the the number of OPERABLE channels less than required by the Minimum OPERABLE Channels per Trip System requirement for both trip systems, place at least one trip system** in the tripped condition within 1 hour and take the ACTION required by Table 3.3.1-1.

SURVEILLANCE REQUIREMENTS

4.3.1.1 Each reactor protection system instrumentation channel shall be demonstrated OPERABLE by the performance of the CHANNEL CHECK, CHANNEL FUNCTIONAL TEST and CHANNEL CALIBRATION operations for the OPERATIONAL CONDITIONS and at the frequencies shown in Table 4.3.1.1-1.

4.3.1.2 LOGIC SYSTEM FUNCTIONAL TESTS and simulated automatic operation of all channels shall be performed at least once per 18 months.***

4.3.1.3 The REACTOR PROTECTION SYSTEM RESPONSE TIME of each reactor trip functional unit shown in Table 3.3.1-2 shall be demonstrated to be within its limit at least once per 18 months. Each test shall include at least one channel per trip system such that all channels are tested at least once every N times 18 months where N is the total number of redundant channels in a specific reactor trip system.

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* With a design providing only one channel per trip system, an inoperable channel need not be placed in the tripped condition where this would cause the Trip Function to occur. In these cases, the inoperable channel shall be restored to OPERABLE status within 2 hours or the ACTION required by Table 3.3.1-1 for that Trip Function shall be taken.

** If more channels are inoperable in one trip system than in the other, select that trip system to place in the tripped condition, except when this would cause the Trip Function to occur.

***The specified 18-month interval may be waived for Cycle 1 provided the surveillance is performed during Refuel 1.

TABLE 4.3.1.1-1 (Continued)

REACTOR PROTECTION SYSTEM INSTRUMENTATION SURVEILLANCE REQUIREMENTS

FUNCTIONAL UNIT	CHANNEL CHECK	CHANNEL FUNCTIONAL TEST	CHANNEL CALIBRATION	OPERATIONAL CONDITIONS FOR WHICH SURVEILLANCE REQUIRED
8. Scram Discharge Volume Water Level - High	NA	M	R	1, 2, 5
9. Turbine Stop Valve - Closure	NA	M	R	1
10. Turbine Control Valve Fast Closure Valve Trip System Oil Pressure - Low	NA	M	R*	1
11. Reactor Mode Switch Shutdown Position	NA	R	NA	1, 2, 3, 4, 5
12. Manual Scram	NA	M	NA	1, 2, 3, 4, 5

(a) Neutron detectors may be excluded from CHANNEL CALIBRATION.
(b) The IRM, and SRM channels shall be determined to overlap for at least 1/2 decades during each startup and the IRM and APRM channels shall be determined to overlap for at least 1/2 decades during each controlled shutdown, if not performed within the previous 7 days.
(c) Within 24 hours prior to startup, if not performed within the previous 7 days.
(d) This calibration shall consist of the adjustment of the APRM channel to conform to the power values calculated by a heat balance during OPERATIONAL CONDITION 1 when THERMAL POWER $\geq$ 25% of RATED THERMAL POWER. Adjust the APRM channel if the absolute difference is greater than 2%. Any APRM channel gain adjustment made in compliance with Specification 3.2.2 shall not be included in determining the absolute difference.
(e) This calibration shall consist of the adjustment of the APRM flow biased channel to conform to a calibrated flow signal.
(f) The LPRMs shall be calibrated at least once per 1000 effective full power hours (EFPH) using the TIP system.
(g) Measure and compare core flow to rated core flow.
(h) This calibration shall consist of verifying the 6 ± 1 second simulated thermal power time constant.

*The specified 18-month interval may be waived for Cycle 1 provided the surveillance is performed during Refuel 1.

3/4.3.3 EMERGENCY CORE COOLING SYSTEM ACTUATION INSTRUMENTATION

LIMITING CONDITION FOR OPERATION

3.3.3 The emergency core cooling system (ECCS) actuation instrumentation channels shown in Table 3.3.3-1 shall be OPERABLE with their trip setpoints set consistent with the values shown in the Trip Setpoint column of Table 3.3.3-2 and with EMERGENCY CORE COOLING SYSTEM RESPONSE TIME as shown in Table 3.3.3-3.

APPLICABILITY: As shown in Table 3.3.3-1.

ACTION:

a. With an ECCS actuation instrumentation channel trip setpoint less conservative than the value shown in the Allowable Values column of Table 3.3.3-2, declare the channel inoperable until the channel is restored to OPERABLE status with its trip setpoint adjusted consistent with the Trip Setpoint value.

b. With one or more ECCS actuation instrumentation channels inoperable, take the ACTION required by Table 3.3.3-1.

c. With either ADS trip system "A" or "B" inoperable, restore the inoperable trip system to OPERABLE status within:

   1. 7 days, provided that the HPCS and RCIC systems are OPERABLE.

   2. 72 hours.

   Otherwise, be in at least HOT SHUTDOWN within the next 12 hours and reduce reactor steam dome pressure to less than or equal to 122 psig within the following 24 hours.

SURVEILLANCE REQUIREMENTS

4.3.3.1 Each ECCS actuation instrumentation channel shall be demonstrated OPERABLE by the performance of the CHANNEL CHECK, CHANNEL FUNCTIONAL TEST and CHANNEL CALIBRATION operations for the OPERATIONAL CONDITIONS and at the frequencies shown in Table 4.3.3.1-1.

4.3.3.2 LOGIC SYSTEM FUNCTIONAL TESTS and simulated automatic operation of all channels shall be performed at least once per 18 months.*

4.3.3.3 The ECCS RESPONSE TIME of each ECCS trip function shown in Table 3.3.3-3 shall be demonstrated to be within the limit at least once per 18 months. Each test shall include at least one channel per trip system such that all channels are tested at least once every N times 18 months where N is the total number of redundant channels in a specific ECCS trip system.

*The specified 18-month interval may be waived for Cycle 1 provided the surveillance is performed during Refuel 1, for LPCI A, B, and C.

TABLE 4.3.3.1-1

EMERGENCY CORE COOLING SYSTEM ACTUATION INSTRUMENTATION SURVEILLANCE REQUIREMENTS

TRIP FUNCTION	CHANNEL CHECK	CHANNEL FUNCTIONAL TEST	CHANNEL CALIBRATION	OPERATIONAL CONDITIONS FOR WHICH SURVEILLANCE REQUIRED
A. DIVISION I TRIP SYSTEM				
1. RHR-A (LPCI MODE) AND LPCS SYSTEM				
a. Reactor Vessel Water Level - Low Low Low, Level 1	NA	M	R	1, 2, 3, 4*, 5*
b. Drywell Pressure - High	NA	M	Q	1, 2, 3
c. LPCS Pump Discharge Flow-Low	NA	M	Q	1, 2, 3, 4*, 5*
d. LPCS and LPCI A Injection Valve Injection Line Pressure Low Interlock	NA	M	R	1, 2, 3, 4*, 5*
e. LPCS and LPCI A Injection Valve Reactor Pressure Low Interlock	NA	M	R	1, 2, 3, 4*, 5*
f. LPCI Pump A Start Time Delay Relay	NA	M	Q	1, 2, 3, 4*, 5*
g. LPCI Pump A Flow-Low	NA	M	Q	1, 2, 3, 4*, 5*
h. Manual Initiation	NA	R***	NA	1, 2, 3, 4*, 5*
2. AUTOMATIC DEPRESSURIZATION SYSTEM TRIP SYSTEM "A"#				
a. Reactor Vessel Water Level - Low Low Low, Level 1	NA	M	R	1, 2, 3
b. Drywell Pressure-High	NA	M	Q	1, 2, 3
c. ADS Timer	NA	M	Q	1, 2, 3
d. Reactor Vessel Water Level - Low, Level 3	NA	M	R	1, 2, 3
e. LPCS Pump Discharge PRessure-High	NA	M	Q	1, 2, 3
f. LPCI Pump A Discharge Pressure-High	NA	M	Q	1, 2, 3
g. Manual Initiation	NA	R	NA	1, 2, 3

TABLE 4.3.3.1-1 (Continued)

EMERGENCY CORE COOLING SYSTEM ACTUATION INSTRUMENTATION SURVEILLANCE REQUIREMENTS

TRIP FUNCTION	CHANNEL CHECK	CHANNEL FUNCTIONAL TEST	CHANNEL CALIBRATION	OPERATIONAL CONDITIONS FOR WHICH SURVEILLANCE REQUIRED
B. DIVISION 2 TRIP SYSTEM				
1. RHR B AND C (LPCI MODE)				
a. Reactor Vessel Water Level - Low Low Low, Level 1	NA	M	R	1, 2, 3, 4*, 5*
b. Drywell Pressure - High	NA	M	Q	1, 2, 3
c. LPCI B and C Injection Valve Injection Line Pressure Low Interlock	NA	M	R	1, 2, 3, 4*, 5*
d. LPCI Pump B Start Time Delay Relay	NA	M	Q	1, 2, 3, 4*, 5*
e. LPCI Pump Discharge Flow-Low	NA	M	Q	1, 2, 3, 4*, 5*
f. Manual Initiation	NA	R***	NA	1, 2, 3, 4*, 5*
g. LPCI B and C Injection Valve Reactor Pressure Low Interlock	NA	M	R	1, 2, 3, 4*, 5*
2. AUTOMATIC DEPRESSURIZATION SYSTEM TRIP SYSTEM "B"#				
a. Reactor Vessel Water Level - Low Low Low, Level 1	NA	M	R	1, 2, 3
b. Drywell Pressure-High	NA	M	Q	1, 2, 3
c. ADS Timer	NA	M	Q	1, 2, 3
d. Reactor Vessel Water Level - Low, Level 3	NA	M	R	1, 2, 3
e. LPCS Pump B and C Discharge Pressure-High	NA	M	Q	1, 2, 3
f. Manual Initiation	NA	R	NA	1, 2, 3

TABLE 4.3.3.1-1 (Continued)

EMERGENCY CORE COOLING SYSTEM ACTUATION INSTRUMENTATION SURVEILLANCE REQUIREMENTS

TRIP FUNCTION	CHANNEL CHECK	CHANNEL FUNCTIONAL TEST	CHANNEL CALIBRATION	OPERATIONAL CONDITIONS FOR WHICH SURVEILLANCE REQUIRED
C. DIVISION 3 TRIP SYSTEM				
1. HPCS SYSTEM				
a. Reactor Vessel Water Level - Low Low, Level 2	NA	M	R	1, 2, 3, 4*, 5*
b. Drywell Pressure-High	NA	M	Q	1, 2, 3
c. Reactor Vessel Water Level-High Level 8	NA	M	R	1, 2, 3, 4*, 5*
d. Condensate Storage Tank Level - Low	NA	M	Q	1, 2, 3, 4*, 5*
e. Suppression Pool Water Level - High	NA	M	Q	1, 2, 3, 4*, 5*
f. Pump Discharge Pressure-High	NA	M	Q	1, 2, 3, 4*, 5*
g. HPCS System Flow Rate-Low	NA	M	Q	1, 2, 3, 4*, 5*
h. Manual Initiation	NA	R	NA	1, 2, 3, 4*, 5*
D. LOSS OF POWER				
1. 4.16 kv Emergency Bus Under-voltage (Loss of Voltage)	NA	NA	R	1, 2, 3, 4**, 5**

#Not required to be OPERABLE when reactor steam dome pressure is less than or equal to 122 psig.
*When the system is required to be OPERABLE after being manually realigned, as applicable, per Specification 3.5.2.
**Required when ESF equipment is required to be OPERABLE.
***The specified 18-month interval may be waived for Cycle 1 provided the surveillance is performed during Refuel 1.

3/4.3.4 RECIRCULATION PUMP TRIP ACTUATION INSTRUMENTATION

ATWS RECIRCULATION PUMP TRIP SYSTEM INSTRUMENTATION

LIMITING CONDITION FOR OPERATION

3.3.4.1 The anticipated transient without scram recirculation pump trip (ATWS-RPT) system instrumentation channels shown in Table 3.3.4.1-1 shall be OPERABLE with their trip setpoints set consistent with the values shown in the Trip Setpoint column of Table 3.3.4.1-2.

APPLICABILITY: OPERATIONAL CONDITION 1.

ACTION:

a. With an ATWS recirculation pump trip system instrumentation channel trip setpoint less conservative than the value shown in the Allowable Values column of Table 3.3.4.1-2, declare the channel inoperable until the channel is restored to OPERABLE status with its trip setpoint adjusted consistent with the Trip Setpoint value.

b. With the number of OPERABLE channels one less than required by the Minimum OPERABLE Channels requirement for one trip function in one trip system, restore the inoperable channel to OPERABLE status within 14 days or be in at least STARTUP within the next 8 hours.

SURVEILLANCE REQUIREMENTS

4.3.4.1.1 Each ATWS recirculation pump trip system instrumentation channel shall be demonstrated OPERABLE by the performance of the CHANNEL CHECK, CHANNEL FUNCTIONAL TEST and CHANNEL CALIBRATION operations at the frequencies shown in Table 4.3.4.1-1.

4.3.4.1.2 LOGIC SYSTEM FUNCTIONAL TESTS and simulated automatic operation of all channels shall be performed at least once per 18 months.*

*The specified 18-month interval may be waived for Cycle 1 provided the surveillance is performed during Refuel 1.

TABLE 4.3.7.11-1

RADIOACTIVE GASEOUS EFFLUENT MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

INSTRUMENT	CHANNEL CHECK	SOURCE CHECK	CHANNEL FUNCTIONAL TEST	CHANNEL CALIBRATION	OPERATIONAL CONDITIONS FOR WHICH SURVEILLANCE REQUIRED
1. MAIN CONDENSER OFFGAS TREATMENT SYSTEM EFFLUENT MONITORING SYSTEM					
a. Noble Gas Activity Monitor - Providing Alarm and Automatic Termination of Release	D	D	Q(1)	R(3)***	*
2. MAIN CONDENSER OFFGAS TREATMENT SYSTEM EXPLOSIVE GAS MONITORING SYSTEM					
a. Hydrogen Monitor	D	N.A.	M	Q(4)	**
3. MAIN STACK MONITORING SYSTEM					
a. Noble Gas Activity Monitor	D	M	Q(5)	R(3)	*
b. Iodine Sampler	W	N.A.	N.A.	N.A.	*
c. Particulate Sampler	W	N.A.	N.A.	N.A.	*
d. Effluent System Flow Rate Monitor	D	N.A.	Q	R	*
e. Sampler Flow Rate Monitor	D	N.A.	Q	R	*
4. CONDENSER AIR EJECTOR RADIOACTIVITY MONITOR					
a. Noble Gas Activity Monitor	D	M	Q(2)	R(3)	#
5. SBGTS MONITORING SYSTEM					
a. Noble Gas Activity Monitor	D	M	Q(5)	R(3)	##
b. Iodine Sampler	W	N.A.	N.A.	N.A.	##
c. Particulate Sampler	W	N.A.	N.A.	N.A.	##
d. Effluent System Flow Rate Monitor	D	N.A.	Q	R	##
e. Sampler Flow Rate Monitor	D	N.A.	Q	R	##

TABLE 4.3.7.11-1 (Continued)

TABLE NOTATION

* At all times.

** During main condenser offgas treatment system operation.

*** The specified 18-month interval may be waived for Cycle 1 provided the surveillance is performed during Refuel 1.

\# During operation of the main condenser air ejector.

\#\# During operation of the SBGTS.

(1) The CHANNEL FUNCTIONAL TEST shall also demonstrate the automatic isolation capability of this pathway, and that control room alarm annunciation occurs if any of the following conditions exists: (each channel will be tested independently so as not to initiate automatic isolation during operation).

1. Instrument indicates measured levels above the alarm/trip setpoint.
2. Loss of power.
3. Instrument alarms on downscale failure.
4. Instrument controls not set in Operate or High Voltage mode. (Automatic isolation shall be demonstrated during the CHANNEL CALIBRATION.)

(2) The CHANNEL FUNCTIONAL TEST for the log scale monitor shall also demonstrate that control room alarm annunciation occurs if any of the following conditions exists:

1. Instrument indicates measured levels above the alarm setpoint.
2. Loss of power.
3. Instrument alarms on downscale failure.
4. Instrument controls not set in Operate or High Voltage mode.

(3) The initial CHANNEL CALIBRATION shall be performed using one or more of the reference radioactive standards certified by the National Bureau of Standards (NBS) or using standards that have been obtained from suppliers that participate in measurement assurance activities with NBS. These standards shall permit calibrating the system over its intended range of energy and measurement range. For subsequent CHANNEL CALIBRATION, the initial reference radioactive standards or radioactive sources that have been related to the initial calibration shall be used.

(4) The CHANNEL CALIBRATION shall include the use of standard gas samples containing a nominal:

1. One volume percent hydrogen, balance nitrogen, and
2. Four volume percent hydrogen, balance nitrogen.

(5) The CHANNEL FUNCTIONAL TEST shall also demonstrate that control room alarm annunciation occurs if any of the following conditions exists:

1. Instrument indicates measured levels above the alarm setpoint.
2. Circuit failure.
3. Instrument controls not set in the Operate mode.

INSTRUMENTATION

3/4.3.8 FEEDWATER/MAIN TURBINE TRIP SYSTEM ACTUATION INSTRUMENTATION

LIMITING CONDITION FOR OPERATION

3.3.8 The feedwater/main turbine trip system actuation instrumentation channels shown in Table 3.3.8-1 shall be OPERABLE with their trip setpoints set consistent with the values shown in the Trip Setpoint column of Table 3.3.8-2.

APPLICABILITY: OPERATIONAL CONDITION 1.

ACTION:

a. With a feedwater/main turbine trip system actuation instrumentation channel trip setpoint less conservative than the value shown in the Allowable Values column of Table 3.3.8-2, declare the channel inoperable and either place the inoperable channel in the tripped condition until the channel is restored to OPERABLE status with its trip setpoint adjusted consistent with the Trip Setpoint value, or declare the associated system inoperable.

b. With the number of OPERABLE channels one less than required by the Minimum OPERABLE Channels per Trip System requirement, restore the inoperable channel to OPERABLE status within 7 days or be in at least STARTUP within the next 6 hours.

c. With the number of OPERABLE channels two less than required by the Minimum OPERABLE Channels per Trip System requirement, restore at least one of the inoperable channels to OPERABLE status within 72 hours or be in at least STARTUP within the next 6 hours.

SURVEILLANCE REQUIREMENTS

4.3.8.1 Each feedwater/main turbine trip system actuation instrumentation channel shall be demonstrated OPERABLE by the performance of the CHANNEL CHECK, CHANNEL FUNCTIONAL TEST and CHANNEL CALIBRATION operations at the frequencies shown in Table 4.3.8.1-1.

4.3.8.2 LOGIC SYSTEM FUNCTIONAL TESTS and simulated automatic operation of all channels shall be performed at least once per 18 months.*

*The specified 18 month interval may be waived for Cycle 1 provided the surveillance is performed during Refuel 1.

3/4.4.3 REACTOR COOLANT SYSTEM LEAKAGE
LEAKAGE DETECTION SYSTEMS

LIMITING CONDITION FOR OPERATION

3.4.3.1 The following reactor coolant system leakage detection systems shall be OPERABLE:

a. The primary containment atmosphere particulate radioactivity monitoring system,

b. The primary containment sump flow monitoring system, and

c. Either the primary containment air coolers condensate flow rate monitoring system or the primary containment atmosphere gaseous radioactivity monitoring system.

APPLICABILITY: OPERATIONAL CONDITIONS 1, 2 and 3.

ACTION:

With only two of the above required leakage detection systems OPERABLE, operation may continue for up to 30 days provided grab samples of the containment atmosphere are obtained and analyzed at least once per 24 hours when the required gaseous and/or particulate radioactive monitoring system is inoperable; otherwise, be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.

SURVEILLANCE REQUIREMENTS

4.4.3.1 The reactor coolant system detection systems shall be demonstrated OPERABLE by:

a. Primary containment atmosphere particulate and gaseous monitoring systems-performance of a CHANNEL CHECK at least once per 12 hours, a CHANNEL FUNCTIONAL TEST at least once per 31 days and a CHANNEL CALIBRATION at least once per 18 months.

b. Primary containment sump flow monitoring system-performance of a CHANNEL FUNCTIONAL TEST at least once per 31 days and a CHANNEL CALIBRATION TEST at least once per 18 months.*

c. Primary containment air coolers condensate flow rate monitoring system-performance of a CHANNEL FUNCTIONAL TEST at least once per 31 days and a CHANNEL CALIBRATION at least once per 18 months.

*The specified 18 month interval may be waived for Cycle 1 provided the surveillance is performed during Refuel 1.

TABLE 3.4.3.2-1

REACTOR COOLANT SYSTEM PRESSURE ISOLATION VALVES

SYSTEM	VALVE NUMBER	FUNCTION
a. LPCS	E21-F006	LPCS Injection
	E-21-F005*	LPCS Injection
b. HPCS	E22-F005	HPCS Injection
	E22-F004	HPCS Injection
c. RHR	E12-F041A	LPCI Injection
	E12-F041B	LPCI Injection
	E12-F041C	LPCI Injection
	E12-F042A	LPCI Injection
	E12-F042B*	LPCI Injection
	E12-F042C*	LPCI Injection
	E12-F050A	Shutdown Cooling Return
	E12-F050B	Shutdown Cooling Return
	E12-F053A	Shutdown Cooling Return
	E12-F053B*	Shutdown Cooling Return
	E12-F009	Shutdown Cooling Suction
	E12-F008	Shutdown Cooling Suction
d. RDIC	E51-F066	RCIC Head Spray
	E51-F065	RCIC Head Spray

*The specified 18 month interval may be waived for Cycle 1 provided the surveillance is performed during Refuel 1.

CONTAINMENT SYSTEMS

MSIV LEAKAGE CONTROL SYSTEM

LIMITING CONDITION FOR OPERATION

3.6.1.4 Two independent MSIV leakage control systems (LCS) shall be OPERABLE.

APPLICABILITY: OPERATIONAL CONDITIONS 1, 2, and 3.

ACTION:

With one MSIV leakage control system inoperable, restore the inoperable system to OPERABLE status within 30 days or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.

SURVEILLANCE REQUIREMENTS

4.6.1.4 Each MSIV leakage control system shall be demonstrated OPERABLE:

a. At least once per 31 days by:

1. Starting the blower(s) from the control room and operating the blower(s) for at least 15 minutes,

2. Energizing the heaters and verifying a current of 8.28 amperes ± 10% per phase for each heater.

b. During each COLD SHUTDOWN in accordance with Specification 4.0.5.

c. At least once per 18 months* by:

1. Performance of a functional test which includes simulated actuation of the system throughout its operating sequence, and verifying that each automatic valve actuates to its correct position and the blower starts.

2. Verifying that the blower develops at least the below required vacuum at the rated capacity:

a) Inboard valves, 15" H_2O at 100 scfm.

b) Outboard valves, 60" H_2O at 200 scfm.

d. By verifying the flow, pressure and, temperature operating instrumentation to be OPERABLE by performance of a:

1. CHANNEL FUNCTIONAL TEST at least once per 31 days, and
2. CHANNEL CALIBRATION at least once per 18 months.

*The specified 18 month interval may be waived for Cycle 1 provided the surveillance is performed during Refuel 1.

TABLE 3.6.3-1 (Continued)

PRIMARY CONTAINMENT ISOLATION VALVES

VALVE FUNCTION AND NUMBER

Other Isolation Valves (Continued)

7. Post LOCA Hydrogen Control

1HG001A, B
1HG002A, B
1HG005A, B
1HG006A, B

8. Standby Liquid Control System

1C41-F004A, B
1C41-F007

9. Reactor Recirculation Seal Injection***

1B33-F013A, B(j)
1B33-F017A, B(j)

10. Drywell Pneumatic System

1IN018

* But ≥ 3 seconds.
(a) See Specification 3.3.2, Table 3.3.2-1, for isolation signal(s) that operates each valve group.
(b) Not included in total sum of Type B and C tests.
(c) May be opened on an intermittent basis under administrative control.
(d) Not closed by SLCS actuation.
(e) Not closed by Trip Functions 5a, b or c, Specification 3.3.2, Table 3.3.2-1.
(f) Not closed by Trip Functions 4a, c, d, e or f of Specification 3.3.2, Table 3.3.2-1.
(g) Not subject to Type C leakage test.
(h) Opens on an isolation signal. Valves will be open during Type A test. No Type C test required.
(i) Also closed by drywell pressure-high signal.
(j) Hydraulic leak test at 43.6 psig.
(k) Not subject to Type C leakage test - leakage rate tested per Specification 4.4.3.2.2.
(l) These penetrations are provided with removable spools outboard of the outboard isolation valve. During operation, these lines will be blind flanged using a double O-ring and a type B leak test. In addition, the packing of these isolation valves will be soap-bubble tested to ensure insignificant or no leakage at the containment test pressure each refueling outage.
** These valves shall have a maximum isolation time of 40 seconds until STARTUP following the first refueling outage.
*** The specified 18-month interval may be waived for Cycle 1 provided the surveillance is performed during Refuel 1.

SURVEILLANCE REQUIREMENTS (Continued)

d. At least once per 18 months* during shutdown by:

1. Subjecting the diesel to an inspection in accordance with procedures prepared in conjunction with its manufacturer's recommendations for this class of standby service.

2. Verifying the diesel generator capability to reject a load of greater than or equal to 1190 kw for diesel generator 0, greater than or equal to 638 kw for diesel generators 1A and 2A, and greater than or equal to 2381 kw for diesel generator 1B while maintaining engine speed less than or equal to 75% of the difference between nominal speed and the overspeed trip setpoint or 15% above nominal, whichever is less.

3. Verifying the diesel generator capability to reject a load of 2600 kw without tripping. The generator voltage shall not exceed 5000 volts during and following the load rejection.

4. Simulating a loss of offsite power by itself, and:

a) For Divisions 1 and 2 and for Unit 2 Division 2:

1) Verifying de-energization of the emergency busses and load shedding from the emergency busses.

2) Verifying the diesel generator starts on the auto-start signal, energizes the emergency busses with permanently connected loads within 13 seconds, energizes the auto-connected loads and operates for greater than or equal to 5 minutes while its generator is so loaded. After energization, the steady state voltage and frequency of the emergency busses shall be maintained at 4160 ± 150 volts and 60 ± 1.2 Hz during this test.

b) For Division 3:

1) Verifying de-energization of the emergency bus.

2) Verifying the diesel generator starts on the auto-start signal, energizes the emergency bus with its loads within 13 seconds and operates for greater than or equal to 5 minutes while its generator is so loaded. After energization, the steady state voltage and frequency of the emergency bus shall be maintained at 4160 ± 150 volts and 60 ± 1.2 Hz during this test.

5. Verifying that on an ECCS actuation test signal, without loss of offsite power, diesel generators 0, 1A and 1B start on the auto-start signal and operate on standby for greater than or equal to 5 minutes. The generator voltage and frequency shall be 4160 + 416, -150 volts and 60 + 3.0, -1.2 Hz within 13 seconds after the auto-start signal; the steady state generator voltage and frequency shall be maintained within these limits during this test.

*The specified 18 month interval may be waived for Cycle 1 provided the surveillance is performed during Refuel 1.

SURVEILLANCE REQUIREMENTS (Continued)

b) Division 2, greater than or equal to:

1) 488.5 amperes for the first 60 seconds,
2) 237.6 amperes for the next 14 minutes,
3) 177.6 amperes for the next 15 minutes, and
4) 141.6 amperes for the next 30 minutes, and
5) 54.4 amperes for the last 180 minutes.

c) Division 3, greater than or equal to*:

1) 58.4 amperes for the first 60 seconds,
2) 11.1 amperes for the next 239 minutes.

d) Unit 2 Division 2, greater than or equal to:

1) 488.5 amperes for the first 60 seconds,
2) 237.6 amperes for the next 14 minutes,
3) 177.6 amperes for the next 15 minutes,
4) 141.6 amperes for the next 30 minutes, and
5) 54.4 amperes for the last 180 minutes.

e. At least once per 60 months, during shutdown, by verifying that the battery capacity is at least 80% of the manufacturers rating when subjected to a performance discharge test. Once per 60 month interval, this performance discharge test may be performed in lieu of the battery service test.

f. Annual performance discharge tests of battery capacity shall be given to any battery that shows signs of degradation or has reached 85% of the service life expected for the application. Degradation is indicated when the battery capacity drops more than 10% of rated capacity from its average on previous performance tests, or is below 90% of the manufacturer's rating.

*The specified 18 month interval may be waived for Cycle 1 provided the surveillance is performed during Refuel 1.

TABLE 3.8.3.31 (Continued)
MOTOR OPERATED VALVES THERMAL OVERLOAD
PROTECTION AND/OR BYPASS DEVICES

	VALVE NUMBER	BYPASS DEVICE (Continuous)(Accident Conditions)	SYSTEM(S) AFFECTED
e.	1B21 - F020	Continuous	Main steam system
	1B21 - F068	Continuous	
	1B21 - F070	Continuous	
	1B21 - F069	Continuous	
	1B21 - F071	Continuous	
	1B21 - F072	Continuous	
	1B21 - F073	Continuous	
f.	1B21 - F065A	Continuous	Main feedwater system
	1B21 - F065B	Continuous	
g.	1E21 - F001	Continuous	LPCS system
	1E21 - F005	Accident Conditions	
	1E21 - F011	Accident Conditions	
	1E21 - F012	Accident Conditions	
h.	1C41 - F001A	Accident Conditions	SBLCS
	1C41 - F001B	Accident Conditions	
i.	1G33 - F001	Accident Conditions	RWCU
	1G33 - F004	Accident Conditions	
j.	1E12 - F052A*	Accident Conditions	RHR system
	1E12 - F064A*	Accident Conditions	
	1E12 - F087A*	Accident Conditions	
	1E12 - F004A	Continuous	
	1E12 - F047A	Continuous	
	1E12 - F048A*	Accident Conditions	
	1E12 - F003A	Continuous	
	1E12 - F026A*	Accident Conditions	
	1E12 - F068A	Continuous	
	1E12 - F073A	Continuous	
	1E12 - F074A	Continuous	
	1E12 - F011A*	Accident Conditions	
	1E12 - F024A*	Accident Conditions	
	1E12 - F016A*	Accident Conditions	
	1E12 - F017A*	Accident Conditions	
	1E12 - F027A*	Accident Conditions	
	1E12 - F004B	Continuous	
	1E12 - F047B	Continuous	
	1E12 - F048B*	Accident Conditions	
	1E12 - F003B	Continuous	
	1E12 - F068B	Continuous	
	1E12 - F073B	Continuous	
	1E12 - F074B	Continuous	
	1E12 - F026B*	Accident Conditions	
	1E12 - F011B*	Accident Conditions	

*The specified 18 month interval may be waived for Cycle 1 provided the surveillance is performed during Refuel 1.

TABLE 3.8.3.31 (Continued)
MOTOR OPERATED VALVES THERMAL OVERLOAD
PROTECTION AND/OR BYPASS DEVICES

	VALVE NUMBER	BYPASS DEVICE (Continuous)(Accident Conditions)	SYSTEM(S) AFFECTED
j.	1E12 - F024B*	Accident Conditions	RHR system
	1E12 - F006B	Continuous	
	1E12 - F016B*	Accident Conditions	
	1E12 - F017B*	Accident Conditions	
	1E12 - F042B*	Accident Conditions	
	1E12 - F064B*	Accident Conditions	
	1E12 - F093	Continuous	
	1E12 - F021*	Accident Conditions	
	1E12 - F004C	Continuous	
	1E12 - F052B*	Accident Conditions	
	1E12 - F087B*	Accident Conditions	
	1E12 - F099B*	Accident Conditions	
	1E12 - F099A*	Accident Conditions	
	1E12 - F008	Accident Conditions	
	1E12 - F009	Accident Conditions	
	1E12 - F040A*	Accident Conditions	
	1E12 - F040B*	Accident Conditions	
	1E12 - F049A*	Accident Conditions	
	1E12 - F049B*	Accident Conditions	
	1E12 - F053A	Accident Conditions	
	1E12 - F053B	Accident Conditions	
	1E12 - F006A	Continuous	
	1E12 - F023	Accident Conditions	
	1E12 - F027B*	Accident Conditions	
	1E12 - F042A*	Accident Conditions	
	1E12 - F042C*	Accident Conditions	
	1E12 - F064C*	Accident Conditions	
	1E12 - F094	Continuous	
k.	1E51 - F086	Accident Conditions	RCIC system
	1E51 - F022	Accident Conditions	
	1E51 - F068	Continuous	
	1E51 - F069	Continuous	
	1E51 - F080	Accident Conditions	
	1E51 - F046	Accident Conditions	
	1E51 - F059	Accident Conditions	
	1E51 - F063	Accident Conditions	
	1E51 - F019	Accident Conditions	
	1E51 - F031	Continuous	
	1E51 - F045	Accident Conditions	
	1E51 - F008	Accident Conditions	
	1E51 - F010	Accident Conditions	
	1E51 - F013	Accident Conditions	
	1E51 - F064	Accident Conditions	
	1E51 - F076	Accident Conditions	

*The specified 18 month interval may be waived for Cycle 1 provided the surveillance is performed during Refuel 1.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION

AMENDMENT NO. 24 TO NPF-11

LA SALLE COUNTY STATION, UNIT 1

DOCKET NO. 50-373

Introduction

By letter dated July 15, 1985, as supplemented by letters dated August 9 and 12, 1985, Commonwealth Edison Company (the licensee) requested a one-time-only approval to temporarily extend a limited number of the surveillance requirements in the Technical Specifications, which must be performed every 18 months and which can only be done when the plant is shutdown. The extension would extend the 18 month surveillances 36 days beyond the maximum 25 percent extension. This would permit the licensee to delay performing this testing until the refueling outage, currently scheduled to commence on or before October 27, 1985.

Evaluation

The Technical Specifications (TS) require a number of surveillance tests to be performed about once every 18 months during a refueling outage. Since the La Salle Unit 1 has been through an extended startup program and has been shutdown several times for equipment failure, feedwater check valve problems, and environmental qualification equipment upgrade, the core has not been fully utilized. Therefore, the licensee rescheduled the refueling outage from September 22, 1985 to October 27, 1985 in order to operate the plant to extend useful core life. The result of this action is that some surveillance testing, normally performed at least once every 18 months during a refueling outage, becomes due before the October 27, 1985 date.

The 18-month surveillance interval was selected to be consistent with the maximum anticipated interval between refueling outages. However, Specification 4.0.2 of the TS does allow the time interval between surveillance testing to be extended by 25 percent in order to accommodate operations scheduling. The end of the most limiting surveillance interval, including the allowable 25 percent extension, for La Salle Unit 1 is September 22, 1985. Therefore, the temporary TS change would actually extend the permissible time to perform this test only about 36 days.

The requirements of the Technical Specifications for testing every 18 months for which deferrals are proposed and the reason these surveillances can only be performed while the reactor is shutdown are as follows:

1. Logic/Functional Testing

   Specification 4.3.1.2 (Reactor Protection System), Table 4.3.3.1-1 and 4.3.3.2 (Low Pressure Coolant Inspection), 4.3.4.1.2 (ATWS-RPT), Table 4.3.7.11-1 (Off-gas Post Treatment Monitor), 4.3.8.2 (Feedwater/

Main Turbine High Level Trip), 4.6.1.4C (MSIV Leakage Control), and 4.8.3.3.1 Thermal Overload Bypass-RHR).

All of the above systems have had functional tests and/or calibrations within their TS surveillance frequency. These functional or calibration tests verify operability of the instrumentation and/or components of which this logic system is a part. The testing performed to meet the above specifications generally entails verification that all portions work together. The result of this testing, however, requires actuation of systems in a mode which is not possible during normal power operation. Since the parts of the systems which are more likely to fail (valves, instruments, etc.) are verified to be operable by current surveillances during the extension period, no impact on plant safety will occur.

2. Pressure Isolation Valves

Specification 4.4.3.2-Table 3.4.3.2-1 (Valve Number 1E12-F042B, 1E12-F042C, 1E12-F053B, 1E21-F005.)

A redundant valve in each line listed above will remain within the Tech Spec surveillance interval. No valves listed are check valves. Gate and globe valves have had a good history of meeting the leakage rate requirements. Alarms monitor the low pressure piping to ensure that any leakage is detected (the alarm function is tested every 31 days). All valves were last left with zero leakage. These valves cannot be tested with the reactor vessel at normal operating pressure. The test also requires access to the drywell which is inerted.

3. Calibrations

Specifications (a) Table 4.3.1.1-1-10 (Turbine Control Valve Pressure Switches) and (b) 4.4.3.1.b (Primary Containment Floor Drain Sump Flow Monitoring)

The items listed above receive periodic functional testing to ensure the ability of the system to operate if required. All devices were found within acceptable limits at the last surveillance. Item (a) requires that the turbine control oil system be shutdown and requires access to the main turbine control valves. The turbine control oil system cannot be secured if the turbine is operating or if the bypass valves are passing steam. Therefore, the unit would have to be shutdown with the Main Steam Isolation Valves shut to perform this test. To perform Item (b) calibration would require access to the drywell.

4. Electric Power Source

Specifications (a) 4.8.1.1.2.d (Diesel Tests) and (b) 4.8.2.3.2.d.2.c (Division III Battery Tests)

The testing required by item (a) is normally done during refueling.

These diesel tests are included in the testing program to periodically ensure that certain functions have not degraded. These tests include logic testing and preventive maintenance. The diesel generators are verified to be operable while in operations by performing several surveillances required by specifications 4.8.1.1.2.a, b and c. This ensures that the diesel will start, will accept load and has available such auxiliaries as necessary. This applies to Divisions II and III only. All Division I testing will remain within the required interval. The testing required by item (b) is for the Division III battery, and this test only verifies that the battery still has sufficient capacity by actual testing. However, verification that battery specific gravities and voltages are proper assures that the battery will be available if required. This slight delay does not affect battery availability. The Technical Specifications require the plant to shutdown to perform this surveillance.

5. Others

Specification Table 3.6.3-1 note (j) (Valves 1B33-F013A, B and 1B33-F017A, B)

The testing for this Specification is a water leak test to verify that these check valves are able to close. These lines are small (3/4 in.) and provide seal injection water to the recirculation pumps from the Control Rod Drive System. The leak test of the 1B33-F013A/B and 1B33-F017A/B valves cannot be performed during normal operation for the following reasons: (1) The line must be isolated inside the drywell which is inerted, and (2) The line normally has flow from the CRD system to the reactor recirculation pump seals.

The 3/4 inch recirculation pump seal water line extends from the recirculation pump seal through the drywell and connects to the CRD supply line outside the drywell. The consequences of leakage through this small line are minimal since the line is always full of water. It has been evaluated that even if failure did occur, it would be bounded by the analysis for a failure of an instrument line. These four valves all had zero leakage during the last test.

Based on the above, we conclude that extension of the interval for the surveillance testing by 36 days on a one-time-only basis is acceptable because the increased surveillance interval of 36 days does not significantly increase the possibility that an undetected failure will occur in any of the related equipment covered by these Technical Specifications.

ENVIRONMENTAL CONSIDERATION

This amendment changes some surveillance requirements on a one-time-only basis. We have determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or

cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: SEP 20 1985

Docket No. 50-373

SEP 20 1985

Mr. Dennis L. Farrar
Director of Licensing
Commonwealth Edison Company
P.O. Box 767
Chicago, Illinois 60690

Dear Mr. Farrar:

SUBJECT: ISSUANCE OF AMENDMENT NO. 24 TO FACILITY OPERATING LICENSE NO. NPF-11 - LA SALLE COUNTY STATION, UNIT 1

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 24 to Facility Operating License No. NPF-11 for the La Salle County Station, Unit 1. This amendment is in response to your letter dated July 15, 1985, as supplemented by letters dated August 9 and 12, 1985. The amendment would extend on a one-time-only basis a limited number of the surveillance requirements in the Technical Specifications which must be performed every 18 months and which can only be done when the plant is shutdown. Your reason for this extension is that La Salle has been through an extended startup program and has been shutdown for various reasons over the past months, and the core has not been fully utilized. Therefore, you have deferred your refueling outage from September 22, 1985 to October 27, 1985.

A copy of the related safety evaluation supporting Amendment No. 24 to Facility Operating License NPF-11 is enclosed.

Sincerely,

Original signed by:

Walter R. Butler, Chief
Licensing Branch No. 2
Division of Licensing

Enclosures:
1. Amendment No. 24 to NPF-11
2. Safety Evaluation

cc w/enclosures:
See next page

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LB#2/DL/LA	LB#2/DL/PM	OELD	LB#2/DL/BC
EHylton	ABournia:lb	Woodhead	WRButler
08/20/85	08/21/85	08/30/85	08/29/85

No issuance until check with Secy after notice expires

3. This amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Walter R. Butler, Chief
Licensing Branch No. 2
Division of Licensing

Enclosure:
Changes to the Technical
Specifications

Date of Issuance: SEP 20 1985

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