

(7590-01)

UNITED STATES NUCLEAR REGULATORY COMMISSION

COMMONWEALTH EDISON COMPANY

DOCKET NO. 373

NOTICE OF CONSIDERATION OF ISSUANCE OF AMENDMENT TO
FACILITY OPERATING LICENSE AND PROPOSED NO SIGNIFICANT HAZARDS
CONSIDERATION DETERMINATION AND OPPORTUNITY FOR HEARING

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of an amendment to Facility Operating License No. NPF-11, issued to Commonwealth Edison Company (the licensee), for operation of the La Salle County Station, Unit 1 located in La Salle County, Illinois.

The amendment would revise the Technical Specifications (TS) to allow a one-time-only extension of time to satisfy a limited number of Technical Specification testing requirements which must be performed every 18 months and require shutdown. Under the amendment, the surveillances would be performed during the first refueling outage, which will occur a maximum of 36 days beyond the time otherwise designated by TS. The purpose of this amendment is to allow more use of the core since the reactor has been shutdown for various reasons over the past months. The Technical Specification requirements for testing every 18 months which are requested to be extended are as follows:

1. Logic/Functional Testing

Specification 4.3.1.2 (Reactor Protection System), Table 4.3.3.1-1

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and 4.3.3.2 (Low Pressure Coolant Inspection), 4.3.4.1.2 (ATWS-RPT), Table 4.3.7.11-1 (Off-gas Post Treatment Monitor), 4.3.8.2 (Feedwater/Main Turbine High Level Trip), 4.6.1.4C (MSIV Leakage Control), and 4.8.3.3.1 Thermal Overload Bypass-RHR).

All of the above systems have had functional tests and/or calibrations within their TS surveillance frequency. These functional or calibration tests verify operability of the instrumentation and/or components of which this logic system is a part. In many cases these tests cover the majority of the logic system. (For example, all channels of the reactor protection system are half scram tested monthly except the mode switch in shutdown position scram). The testing performed to meet the above specifications generally entails verification that all portions work together. The result of this testing, however, requires actuation of systems in a mode which is not possible during normal power operation. Since the parts of the systems which are more likely to fail (valves, instruments, etc) are verified to be operable by current surveillances during the extension period, no impact on plant safety will occur.

2. Pressure Isolation Valves

Specification 4.4.3.2-Table 3.4.3.2-1 (Valve Number 1E12-F042B, 1E12-F042C, 1E12-F053B, 1E21-F005.)

A redundant valve in each line listed above will remain within the Tech Spec surveillance interval. No valves listed are check valves. Gate and globe valves have had a good history of meeting the leakage

rate requirements. Alarms monitor the low pressure piping to ensure that any leakage is detected (the alarm function is tested every 31 days). All valves were last left with zero leakage. These valves cannot be tested with the reactor vessel at normal operating pressure. The test also requires access to the drywell which is inerted.

3. Calibrations

Specifications (a) Table 4.3.1.1-1-10 (Turbine Control Valve Pressure Switches) and (b) 4.4.3.1.b (Primary Containment Floor Drain Sump Flow Monitoring)

The items listed above receive periodic functional testing to ensure the ability of the system to operate if required. All devices were found within acceptable limits at the last surveillance. Item (a) requires that the turbine control oil system be shutdown and requires access to the main turbine control valves. The turbine control oil system cannot be secured if the turbine is operating or if the bypass valves are passing steam. Therefore, the unit would have to be shutdown with the Main Steam Isolation Valves shut to perform this test. To perform Item (b) calibration would require access to the drywell.

4. Electric Power Source

Specifications (a) 4.8.1.1.2.d (Diesel Tests) and (b) 4.8.2.3.2.d.2.c (Division III Battery Tests)

The testing required by item (a) is normally done during refueling. These diesel tests are included in the testing program to periodically

ensure that certain functions have not degraded. These tests include logic testing and preventive maintenance. The diesel generators are verified to be operable while in operations by performing several surveillances required by specifications 4.8.1.1.2.a, b and c. This ensures that the diesel will start, will accept load and has available such auxiliaries as necessary. This applies to Divisions II and III only. All Division I testing will remain within the required interval.

The testing required by item (b) is for the Division III battery, and this test only verifies that the battery still has sufficient capacity by actual testing. However, verification that battery specific gravities and voltages are proper assures that the battery will be available if required. This slight delay does not affect battery availability. The Technical Specifications require the plant to shutdown to perform this surveillance.

5. Others

Specification Table 3.6.3-1 note (j) (Valves 1B33-F013A, B and 1B33-F017A, B)

The testing for this Specification is a water leak test to verify that these check valves are able to close. These lines are small (3/4 in.) and provide seal injection water to the recirculation pumps from the Control Rod Drive System.

The leak test of the 1B33-F013A/B and 1B33-F017A/B valves cannot be performed during normal operation for the following reasons (1) The line must be isolated inside the drywell which is inerted, (2) The line normally has flow from the CRD system to the reactor recirculation pump seals.

The 3/4 inch recirculation pump seal water line extends from the recirculation pump seal through the drywell and connects to the CRD supply line outside the drywell. The consequences of leakage through this small line are minimal since the line is always full of water. It has been evaluated that even if failure did occur, it would be bounded by the analysis for a failure of an instrument line. These four valves all had zero leakage during the last test.

The 18 month surveillance interval was selected to be consistent with the maximum anticipated interval between refueling outages. However, the Technical Specifications allow an extension of 25% to this 18 month frequency to accommodate operations scheduling. Therefore, the end of the most limiting surveillance interval, including the allowable 25% extension, for La Salle Unit 1 is September 22, 1985. The refueling is currently expected to start on or before October 27, 1985. This is a consequence of La Salle Unit 1 having gone through an extended startup program and having been shutdown several times because of equipment failures, feed-water check valve problems, and environmental qualification upgrades. The period of plant operation during the requested extensions, therefore, is a maximum of 36 days. This amendment is in accordance with the licensee's application for amendment dated July 15, 1985, as supplemented by letters dated August 9, 1985 and August 12, 1985.

Before issuance of the proposed license amendment, the Commission will have made findings required by the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations.

The Commission has made a proposed determination that the amendment request involves no significant hazards consideration. Under the Commission's regulations in 10 CFR 50.92, this means that operation of the facility in accordance with the proposed amendment would not (1) involve a significant increase in the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in a margin of safety.

The licensee, in its letter of July 15, 1985, has determined and the NRC staff agrees that the proposed amendment will not: 1) Involve a significant increase in the probability or consequences of an accident previously evaluated because the operability of the equipment is tested and maintained by other TS requirements and based on the type of surveillances extended, no significant increase in the probability of equipment failure is postulated, 2) Create the possibility of a new or different kind of accident from any accident previously evaluated because this amendment neither removes or adds any equipment nor does it eliminate tests required at refueling outage; and 3) Involve a significant reduction in the margin of safety because the increased surveillance interval (36-days) does not significantly increase the possibility that an undetected failure will occur in any of the related equipment covered by these Technical Specifications. Accordingly, the

Commission proposes to determine that the proposed changes to the Technical Specifications involve no significant hazards considerations.

The Commission is seeking public comments on this proposed determination. Any comments received within 30 days after the date of publication of this notice will be considered in making any final determination. The Commission will not normally make a final determination unless it receives a request for a hearing.

By _____, the licensee may file a request for a hearing with respect to issuance of the amendment to the subject facility operating license and any person whose interest may be affected by this proceeding and who wishes to participate as a party in the proceeding must file a written petition for leave to intervene. Request for a hearing and petitions for a hearing and petitions for leave to intervene shall be filed in accordance with the Commission's "Rules of Practice for Domestic Licensing Proceedings" in 10 CFR Part 2. If a request for a hearing or petition for leave to intervene is filed by the above date, the Commission or an Atomic Safety and Licensing Board, designated by the Commission or by the Chairman of the Atomic Safety and Licensing Board Panel, will rule on the request and/or petition and the Secretary or the designated Atomic Safety and Licensing Board will issue a notice of hearing or an appropriate order.

As required by 10 CFR §2.714, a petition for leave to intervene shall set forth with particularity the interest of the petitioner in the proceeding, and how that interest may be affected by the results of the proceeding. The petition should specifically explain the reasons why intervention should be

permitted with particular reference to the following factors: (1) the nature of the petitioner's right under the Act to be made a party to the proceeding; (2) the nature and extent of the petitioner's property, financial, or other interest in the proceeding; and (3) the possible effect of any order which may be entered in the proceeding on the petitioner's interest. The petition should also identify the specific aspect(s) of the subject matter of the proceeding as to which petitioner wishes to intervene. Any person who has filed a petition for leave to intervene or who has been admitted as a party may amend the petition without requesting leave of the Board up to fifteen (15) days prior to the first prehearing conference scheduled in the proceeding, but such an amended petition must satisfy the specificity requirements described above.

Not later than fifteen (15) days prior to the first prehearing conference scheduled in the proceeding, a petitioner shall file a supplement to the petition to intervene which must include a list of the contentions which are sought to be litigated in the matter, and the bases for each contention set for the reasonable specificity. Contentions shall be limited to matters within the scope of the amendment under consideration. A petitioner who fails to file such a supplement which satisfies these requirements with respect to at least one contention will not be permitted to participate as a party.

Those permitted to intervene become parties to the proceeding subject to any limitations in the order granting leave to intervene, and have the opportunity to participate fully in the conduct of the hearing, including the opportunity to present evidence and cross-examine witnesses.

If a hearing is requested, the Commission will make a final determination on the issue of no significant hazards consideration. The final determination will serve to decide when the hearing is held.

If the final determination is that the amendment request involves no significant hazards consideration, the Commission may issue the amendment and make it effective, notwithstanding the request for a hearing. Any hearing held would take place after issuance of the amendment.

Normally, the Commission will not issue the amendment until the expiration of the 30-day notice period. However, should circumstances change during the notice period such that failure to act in a timely way would result, for example, in derating or shutdown of the facility, the Commission may issue the license amendment before the expiration of the 30-day notice period, provided that its final determination is that the amendment involves no significant hazards consideration. The final determination will consider all public and State comments received. Should the Commission take this action, it will publish a notice of issuance and provide for opportunity for a hearing after issuance. The Commission expects that the need to take this action will occur very infrequently.

A request for a hearing or a petition for leave to intervene must be filed with the Secretary of the Commission, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Docketing and Service Branch, or may be delivered to the Commission's Public Document Room, 1717 H Street, N.W.,

Washington, D.C., by the above date. Where petitions are filed during the last ten (10) days of the notice period, it is requested that the petitioner promptly so inform the Commission by a toll-free telephone call to Western Union at (800) 325-6000 (in Missouri (800) 342-6700). The Western Union operator should be given Datagram Identification Number 3737 and the following message addressed to Walter R. Butler: petitioner's name and telephone number; date petition was mailed; plant name; and publication date and page number of the FEDERAL REGISTER notice. A copy of the petition should also be sent to the Executive Legal Director, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, and to Isham, Lincoln, and Burke, Suite 840, 1120 Connecticut Avenue, N.W., Washington, D.C. 20036, attorney for the licensee.

Nontimely filings of petitions for leave to intervene, amended petitions, supplemental petitions and/or requests for hearing will not be entertained absent as a determination by the Commission, the presiding officer or the Atomic Safety and Licensing Board designated to rule on the petition and/or request, that the petitioner has made a substantial showing of good cause for the granting of a late petition and/or request. That determination will be based upon a balancing of the factors specified in 10 CFR 2.714(a)(1)(i)-(v) and 2.714(a).

For further details with respect to this action, see the application for amendment which is available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C., and at the Public Library of Illinois Valley Community College, Rural Route No. 1, Ogelsby, Illinois 61348.

Dated at Bethesda, Maryland, this 1st day of August 1985.

FOR THE NUCLEAR REGULATORY COMMISSION

Walter R. Butler
Walter K. Butler, Chief
Licensing Branch No. 2
Division of Licensing

DISTRIBUTION: *w/enc.*
Docket File (50-373)
 LB#2 Reading
 ABournia
 EGHylton
 NRC PDR
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 NSIC
 PRC System

August 19, 1985

DOCKET NO(S). 50-373

Mr. Dennis L. Farrar
 Director of Licensing
 Commonwealth Edison Company
 P. O. Box 767
 Chicago, Illinois 60690

SUBJECT: NOTICE OF CONSIDERATION OF ISSUANCE OF AMENDMENT TO FACILITY OPERATING LICENSE
 AND PROPOSED NO SIGNIFICANT HAZARDS CONSIDERATION DETERMINATION AND
 OPPORTUNITY FOR HEARING - LA SALLE, UNIT 1

The following documents concerning our review of the subject facility are transmitted for your information.

- Notice of Receipt of Application, dated _____.
- Draft/Final Environmental Statment, dated _____.
- Notice of Availability of Draft/Final Environmental Statement, dated _____.
- Safety Evaluation Report, or Supplement No. _____, dated _____.
- Notice of Hearing on Application for Construction Permit, dated _____.
- Notice of Consideration of Issuance of Facility Operating License, dated _____.
- Monthly Notice; Applications and Amendments to Operating Licenses Involving no Significant Hazards Considerations, dated _____.
- Application and Safety Analysis Report, Volume _____.
- Amendment No. _____ to Application/SAR dated _____.
- Construction Permit No. CPPR- _____, Amendment No. _____ dated _____.
- Facility Operating License No. _____, Amendment No. _____, dated _____.
- Order Extending Construction Completion Date, dated _____.
- Other (Specify) **See subject above.**

Office of Nuclear Reactor Regulation

Enclosures:
 As stated

cc: See next page

OFFICE	DL:LB#2						
SURNAME	EGHylton						
DATE	8/19/85						



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

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Office of Nuclear Reactor Regulation

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As stated

cc: See next page

Mr. Dennis L. Farrar
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Units 1 & 2

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