

APR 17 1982

Docket Nos. 50-373

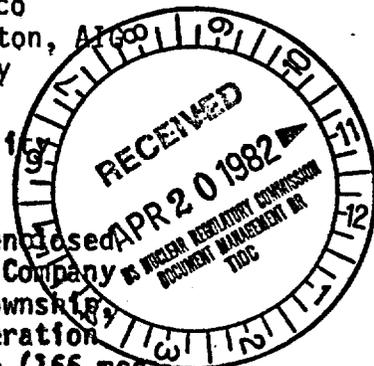
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Mr. Cordell Reed
 Vice President, Nuclear Operations
 Commonwealth Edison Company
 Post Office Box 767
 Chicago, Illinois 60690

Dear Mr. Reed:

Subject: La Salle County Station, Unit 1 - Issuance of Facility
 Operating License



The U.S. Nuclear Regulatory Commission (NRC) has issued the enclosed Facility Operating License No. NPF-11 to Commonwealth Edison Company for La Salle County Station, Unit 1, located in Brookfield Township, La Salle County, Illinois. License No. NPF-11 authorizes operation of the La Salle County Station, Unit 1, at five percent power (166 megawatts thermal). Authorization to operate beyond five percent is still under consideration by the NRC. The issuance of this license authorizing operation at five percent of full power is without prejudice to future consideration by the Commission with respect to operation at power levels in excess of five percent.

Also enclosed is a copy of a related Federal Register notice which has been forwarded to the Office of the Federal Register for publication.

Two signed copies of Amendment No. 3 to Indemnity Agreement No. B-84 which covers the activities authorized under License No. NPF-11 are also enclosed. Please sign and return one copy to this office.

Supplement No. 3 to the Safety Evaluation Report for La Salle County Station, Units 1 and 2 has been issued. Two copies are enclosed; 18 additional copies will be forwarded to you in about a week.

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Darrell G. Eisenhut, Director
 Division of Licensing
 Office of Nuclear Reactor Regulation

Enclosures:

1. Facility Operation License No. NPF-11
2. Federal Register Notice
3. Amendment No. 3 to Indemnity Agreement No. B-84
4. Supplement No. 3 to the SER (2)

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The Honorable Tom Corcoran
United States House of Representatives
Washington, D. C. 20515

Chairman
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-373

LA SALLE COUNTY STATION, UNIT 1

FACILITY OPERATING LICENSE

License No. NPF-11

1. The Nuclear Regulatory Commission (the Commission or the NRC) having found that:
 - A. The application for a license filed by the Commonwealth Edison Company complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I, and all required notifications to other agencies or bodies have been duly made;
 - B. Construction of the La Salle County Station, Unit 1 (the facility), has been substantially completed in conformity with Construction Permit No. CPPR-99 and the application, as amended, the provisions of the Act, and the regulations of the Commission;
 - C. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the regulations of the Commission;
 - D. There is reasonable assurance: (i) that the activities authorized by this operating license can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - E. The Commonwealth Edison Company is technically qualified to engage in the activities authorized by this operating license in accordance with the Commission's regulations set forth in 10 CFR Chapter I;
 - F. The Commonwealth Edison Company has satisfied the applicable provisions of 10 CFR Part 140, "Financial Protection Requirements and Indemnity Agreements," of the Commission's regulations;
 - G. The issuance of this license will not be inimical to the common defense and security or to the health and safety of the public;

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- H. After weighing the environmental, economic, technical, and other benefits of the facility against environmental and other costs and considering available alternatives, the issuance of Facility Operating License No. NPF-11, subject to the conditions for protection of the environment set forth herein, is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied; and
 - I. The receipt, possession, and use of source, by-product and special nuclear material as authorized by this license will be in accordance with the Commission's regulations in 10 CFR Parts 30, 40 and 70.
2. Based on the foregoing findings regarding this facility, Facility Operating License NPF-11 is hereby issued to the Commonwealth Edison Company (the licensee) to read as follows:
- A. This license applies to the La Salle County Station, Unit 1, a boiling water nuclear reactor and associated equipment, owned by the Commonwealth Edison Company. The facility is located in Brookfield Township, La Salle County, Illinois, and is described in the licensee's "Final Safety Analysis Report," as supplemented and amended, and in the licensee's Environmental Report, as supplemented and amended.
 - B. Subject to the conditions and requirements incorporated herein, the Commission hereby licenses:
 - (1) Commonwealth Edison Company, pursuant to Section 103 of the Act and 10 CFR Part 50, "Licensing of Production and Utilization Facilities," to possess, use, and operate the facility at the designated location in Brookfield Township, La Salle County, Illinois, in accordance with the procedures and limitations set forth in this license;
 - (2) Commonwealth Edison Company, pursuant to the Act and 10 CFR Part 70, to receive, possess and use at any time special nuclear material as reactor fuel, in accordance with the limitations for storage and amounts required for reactor operation, as described in the Final Safety Analysis Report, as supplemented and amended;
 - (3) Commonwealth Edison Company, pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess, and use at any time any byproduct, source and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;

- (4) Commonwealth Edison Company, pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess, and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and
- (5) Commonwealth Edison Company, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.

C. This license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

The licensee is authorized to operate the facility at reactor core power levels not in excess of 5 percent of full power (166 megawatts thermal) in accordance with the conditions specified herein and in Attachment 1 to this license. The preoperational tests, startup tests and other items identified in Attachment 1 to this license shall be completed as specified. Attachment 1 is an integral part of this license.

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B attached hereto are hereby incorporated in this license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

(3) Conduct of Work Activities During Fuel Load and Initial Startup

The licensee shall review by committee all Unit 1 Preoperational Testing and System Demonstration activities performed concurrently with Unit 1 initial fuel loading or with the Unit 1 Startup Test Program to assure that the activity will not affect the safe performance of the Unit 1 fuel loading or the portion of the Unit 1 Startup Program being performed. The review shall address, as a minimum, system interaction, span of control, staffing, security and health physics, with respect to performance of the activity concurrently with the Unit 1 fuel loading or the portion of the Unit 1 Startup Program being performed. The committee for the review shall be composed of a least three members, knowledgeable in the above areas, and who meet the qualifications for professional-technical personnel specified by

section 4.4 of ANSI N18.7-1971. At least one of these three shall be a senior member of the Assistant Superintendent of Operation's staff.

(4) Resolution of Rebar Damage and Adequacy of Off-gas Building Roof

The licensee shall complete its assessment of the rebar damaged due to drilling and coring in concrete and the structural adequacy of the off-gas building roof. The results shall be reported to the NRC staff for review and approval, prior to power operation following initial criticality and zero power physics testing.

(5) Snubbers

- (a) Prior to criticality, the licensee shall submit for NRC approval, a revised list of safety-related snubbers to be contained in Table 3.7.9-1 of the Technical Specifications to include such snubbers on lines 3 inches in diameter or less.
- (b) Prior to startup after the first refueling outage, the licensee shall provide, as necessary, a revision to the Technical Specifications to remove snubbers that are determined to be unnecessary and replace them with rigid strut and rod assemblies.

(6) Deferred Preoperational Deficiencies

The licensee shall satisfactorily resolve those deficiencies which were deferred from the preoperational testing program on a schedule that shall assure that the capability of a system required to be operable by Technical Specification is not degraded.

(7) Surveillance of Tendons (Section 3.8.1*, SSER #3)

Prior to full power, the licensee shall supply the predicted lift-off forces required to complete Tables 4.6.1.5-1 and 4.6.1.5-2 of the Technical Specifications.

(8) Masonry Wall (Section 3.8.3, SER, SSER #2)

Based on the findings of our preliminary review of the licensee's submittals and its commitments related to masonry wall evaluation, the following actions are required by the licensee:

- (a) The present fixes for modifications implemented shall not preclude the option of implementing additional modifications if directed by our future review of the licensee's design criteria.

*The parenthetical notation following the title of many license conditions denotes the section of the Safety Evaluation Report and/or its supplements wherein the license condition is discussed.

- (b) Prior to startup after the first refueling outage, the licensee shall resolve the differences between our interim criteria and the criteria used by the licensee to the satisfaction of the staff and shall implement the required wall fixes or modifications that might result from such a resolution.

(9) Inservice Testing of Pumps and Valves (Section 3.9.6, SER)

Pursuant to 10 CFR Part 50.55a, the relief that the licensee has requested from the pump and valve testing requirements of 10 CFR Part 50, Section 55.55 (g)(2) and (g)(4)(i) is granted for that portion of the initial 120-month period during which we complete our review.

(10) Dynamic Qualification (3.10, SER, SSER #1, SSER #2)

- (a) Prior to startup after the first refueling outage, the licensee shall complete any modifications or replacement of equipment as a result of the fatigue evaluation. In the interim, the licensee shall document the occurrence of every safety relief valve actuation into the suppression pool, the associated cumulative damage factors calculated for typical representative equipment and kept up-to-date, and report to NRC any malfunction of equipment that occurs due to any safety relief discharge.
- (b) Prior to startup after the first refueling outage, the licensee shall replace or modify the NSSS equipment (intermediate range monitor, C51-K-601A/H and two-inch air-operated globe valve, C11-F011) if the results of the requalification tests indicate either change is required.

(11) Environmental Qualifications (Section 3.11, SER, SSER #1, SSER #2)

- (a) No later than June 30, 1982, the licensee shall be in compliance with the provisions of NUREG-0588, "Interim Staff Position on Environmental Qualification of Safety-Related Electrical Equipment", for safety-related electrical equipment exposed to a harsh environment.
- (b) Complete and auditable records must be available and maintained at a central location which describe the environmental qualification methods used for all safety-related electrical equipment in sufficient detail to document the degree of compliance with NUREG-0588. Such records shall be updated and maintained current as equipment is replaced, further tested, or otherwise further qualified to document complete compliance no later than June 30, 1982.
- (c) The licensee shall complete the corrective actions stipulated in Appendix C to Supplement No. 2 of the Safety Evaluation Report by June 30, 1982.

(12) Seismic and Loss-of-Coolant Accident Loads (Section 4.2.3.4, SER, SSER #1, SSER #2)

- (a) By July 30, 1982, the licensee shall submit to NRC a complete description of the analytical methods along with all analytical results necessary to show that La Salle fully meets the criteria of Appendix A to the Standard Review Plan, Section 4.2 (NUREG-0800) with regard to fuel assembly liftoff.
- (b) Prior to startup after the first refueling outage, the review of the fuel assembly liftoff issue must be satisfactorily resolved to the satisfaction of the staff.

(13) Surveillance of Control Blade (Section 4.2.3.14, SER)

IE Bulletin No. 79-26, Revision 1, "Boron Loss from BWR Control Blades," describes certain actions to be taken by licensees to determine boron loss from BWR control blades. The licensee shall comply with items 1, 2 and 3 of this bulletin and submit a written response on item 3 within 30 days after plant startup following the first refueling outage.

(14) Scram Discharge Volume (Sections 4.6.2, SER and 6.3.2.3, SSER #2)

- (a) Prior to startup after the first refueling outage, the licensee shall incorporate the following additional modifications into the scram discharge volume system:
 - (i) Redundant vent and drain valves, and
 - (ii) Diverse and redundant scram instrumentation for each instrumented volume, including both delta pressure sensors and float sensors.
- (b) Prior to startup after the first refueling outage, the licensee shall complete system or procedural modifications, if required, as a result of the staff's completion of its review of the licensee's response to NUREG-0803.

(15) Low Pressure in Pump Discharge of the Control Rod Drive (Section 4.6.2, SSER #2)

Prior to startup after the first refueling outage, the licensee shall install instrumentation for an automatic scram that would shut down the reactor in the event of low control rod drive pump discharge pressure to be activated during startup and refueling modes only.

(16) Containment Long Term Program Load Specifications (6.2.1.1, SSER #2)

Prior to October 1, 1982, the licensee shall submit its confirmatory assessment of the containment design adequacy for pool dynamic loads (chugging, vent lateral and diaphragm reverse pressure) developed in conjunction with the Long Term Program and reported in NUREG-0808.

(17) Pressure Interlocks on Valves Interfacing at Low and High Pressure (Section 6.3.4, SSER #2)

Prior to startup after the first refueling outage, the licensee shall implement isolation protection in conformance to the requirements of Section 6.3 of the Standard Review Plan against overpressurization of the low pressure emergency core cooling systems (RHR/LPCI and LPCS) at the high and low pressure interface containing a check valve and a closed motor-operated valve.

(18) Compliance with Regulatory Guide 1.97 (Sections 7.5.2, SER)

By July 1, 1982, the licensee shall provide a plan for implementing modifications necessary to comply with Revision 2 of Regulatory Guide 1.97, "Instrumentation for Light Water Cooled Nuclear Power Plants to Assess Plant Conditions During and Following an Accident," dated December 1980.

(19) Additional Instrumentation and Control Concerns (Section 7.7.3.4, SSER #1)

The licensee shall resolve the following concerns to the NRC staff's satisfaction prior to startup after the first refueling outage:

- (a) whether common electrical power sources or sensor malfunctions may cause multiple control systems failures, and
- (b) whether high energy line breaks will result in unacceptable consequential control system failures.

(20) Low and/or Degraded Grid Voltage (Section 8.2.2.2, SER)

The licensee shall install a second level of undervoltage protection prior to startup after the first refueling outage.

(21) Reliability of Diesel-Generators (Sections 8.3.1.1, SER and 9.6.3.4, SER)

Prior to startup after the first refueling outage, the licensee shall implement the following design modifications with respect to diesel-generator reliability:

- (a) A heavy duty turbocharger gear drive assembly be installed on the diesel-generators.
- (b) A prelube pump, powered from a reliable direct current power supply, be installed in the system to operate in parallel with the engine-driven lube oil pump, or an alternative acceptable to the NRC shall be installed to preclude dry-starting of the diesel-engine.
- (c) Controls and monitoring instrumentation be removed from the engine and engine skid, except instruments qualified for this location. The non-qualified control and monitoring instruments shall be installed on a free standing floor mounted panel and located on a vibration free floor area. If the floor is not vibration free, the panel shall be equipped with vibration mounts.

(22) Direct Current Power Systems (Section 8.3.1.2, SER)

Prior to startup after the first refueling outage for the 125- and 250-volt direct current systems for Divisions 1 and 2 and the 125-volt Division 3 direct current system, the following additional instrumentation shall be provided in the control room: (1) Battery current (ammeter-charge/discharge), (2) Battery charger output voltage (voltmeter), (3) Battery charger output current (ammeter), (4) Battery high discharge rate alarm, and (5) Battery charger trouble alarm. In the interim, the licensee shall implement approved procedures to monitor battery current, battery charger output voltage, and battery charger output current at the local panels at least once per eight hour shift.

(23) Reactor Containment Electrical Penetrations (Section 8.4.1, SER)

Prior to startup after the first refueling outage, a redundant fault current device (circuit breakers or fuses) shall be provided on each penetrating circuit that would limit a fault current surge to be less than the surge for which the penetration is qualified except for low energy (milliamps) instrument systems.

(24) Separation of Class 1E and Non-Class 1E Cable Trays (Section 8.4.6.1, SER, SSER #1, SSER #2)

Prior to startup after the first refueling outage, the licensee shall provide adequate separation or barriers between Class 1E and adjacent non-Class 1E cable trays.

(25) Fire Protection Program (Section 9.5, SER, SSER #2, SSER #3)

- (a) The licensee shall maintain in effect and fully implement all provisions of the approved fire protection plan. In addition, the licensee shall maintain the fire protection program set forth in Appendix R to 10 CFR Part 50, except for the following deviations:**
 - (i) Hydrostatic hose tests in accordance with NEPA 1962-1979, and**
 - (ii) No automatic fire detection systems in areas 2K/3K and 5B4.**
- (b) Prior to initial criticality, the licensee shall install a 1-hour rated barrier on all four sides of a partially protected power cable pan and a general sprinkler system, both located in the diesel-generator corridor.**
- (c) Prior to startup after the first refueling outage, the licensee shall provide fire protection systems in fire areas 2C/3C, 4C3 and 6E.**
- (d) Prior to startup after the first refueling outage, the licensee with respect to fire doors shall implement one of the following:**
 - (i) Perform an engineering review of the manufacturer's certified doors and door frames by a nationally recognized laboratory to certify that the door and door frames provide the required fire resistance rating, or**
 - (ii) Test a replicate "as installed" door assembly by a nationally recognized laboratory to determine the door rating, or**
 - (iii) Replace manufacturer's labeled doors and door frames with UL rated items.**
- (e) Prior to startup after the first refueling outage, the licensee shall demonstrate the adequacy of its fire protection for record storage.**

(26) Radiation/Chemistry Technicians on the Backshift (Section 13.1, SSER#2)

- (a) All Radiation/Chemistry Technicians on the backshift shall be trained per the La Salle's Training Qualification Guide. All such Technicians shall also have satisfactorily completed the following emergency response training:
- (i) Tasks to be performed during the first 60 minutes of a serious emergency on the backshift;
 - (ii) Post-accident sampling and analysis for the first three hours of an emergency;
 - (iii) In-plant radiation surveys during an accident;
 - (iv) Use and interpretation of both portable and fixed area radiation monitoring equipment, such as the Eberline PING-3 and SAM-2;
 - (v) Interpretation of critical effluent monitoring data for assisting the Chief Engineer during the first hour of an accident (i.e., station vent monitor and standby gas treatment monitor);
 - (vi) First aid and bioassay techniques; and
 - (vii) Use of respiratory equipment during emergency situations.
- (b) By June 1, 1983, the licensee shall have Radiation/Chemistry Technicians onshift for 24 hours per day who meet ANSI N18.1-1971 or who are qualified in accordance with a NRC approved alternative program.

(27) Industrial Security (Section 13.6, SER, SSER#3)

The licensee shall maintain in effect and fully implement all provisions of the Commission's approved physical security plan, guard training and qualification plan, and contingency plan, including amendments made pursuant to the authority of 10 CFR 50.54(p). The approved plans which contain safeguard information are collectively entitled: "La Salle County Station Security Plan Units 1 and 2," Revision 11, dated December 24, 1981; "La Salle County Station Guard Training and Qualification Plan," submitted by their letter dated August 16, 1979, as revised in August, 1980; and "La Salle Nuclear Power Station Contingency Plan," dated March, 1980, as revised by pages dated June, 1980.

The licensee is exempt from the commitment to fully implement those portions of the Security Plan as described in Items 1 and 2 in the licensee's letter dated April 1, 1982, provided that the compensatory measures delineated in the above referenced letter are in place. Compensatory measures as described in Item 3 in the April 1, 1982 letter are approved with full implementation of the security plan commitments to be accomplished no later than July 1, 1982.

The licensee is exempted from the provisions of 10 CFR 73.55(d)(9), but shall meet all other commitments of the physical security plan and the following additional items.

- (a) Change all keys, locks, and combinations and related equipment used to control access to protected areas and vital areas at least every 12 months.
- (b) Issue keys, locks, combinations, and other access control devices to protected and vital areas only to those individuals who possess access authorization to those areas.
- (c) Change keys, locks, combinations, and related equipment to which an individual had access within 5 days and immediately for card keys after access authorization is withdrawn due to lack of trustworthiness, reliability, or inadequate work performance.

(28) Initial Test Program (Section 14, SER)

The licensee shall conduct the post-fuel-loading initial test program (set forth in Section 14 of the licensee's Final Safety Analysis Report, as amended) without making any major modifications of this program unless modifications have been identified and have received prior NRC approval. Major modifications are defined as:

- (a) Elimination of any test identified in Section 14 of the licensee's Final Safety Analysis Report, as amended as being essential;
- (b) Modification of test objectives, methods or acceptance criteria for any test identified in Section 14 of the licensee's Final Safety Analysis Report, as amended as being essential;
- (c) Performance of any test at a power level different from that described in the program; and
- (d) Failure to complete any tests included in the described program (planned or scheduled for power levels up to the authorized power level).

(29) Assurance of Proper Design and Construction (Section 17.4, SSER #2)

Prior to exceeding 5 percent of full power, the licensee shall have conducted an independent review of the mechanical and structural design of the loop C residual heat removal system, excluding all branch piping less than 3 inches, in the functioning mode of the low pressure injection system using loads resulting from

the actuation of the automatic depressurization system in conjunction with the operating basis earthquake to verify that this system has been designed and constructed in accordance with all pertinent NRC requirements. This verification review shall consider design, installation, inspection, testing, and any other aspects necessary to ensure conformance with the design. This review shall be performed independently of the licensee and its contractors who performed design and construction activities for the La Salle County Station, and it shall be completed to the satisfaction of NRC.

(30) NUREG-0737 Conditions (Section 22.2)

The licensee shall complete the following conditions to the satisfaction of the NRC. These conditions reference the appropriate items in Section 22.2, "TMI Action Plan Requirements for Applicants for Operating Licenses," in the Safety Evaluation Report and Supplements 1, 2 and 3, NUREG-0519.

(a) Shift Technical Advisor (I.A.1.1, SER, SSER #2)

The Shift Technical Advisor (STA) function shall be fulfilled by the Station Control Room Engineer (SCRE) who will be a designated SRO. However, if a SCRE is not available, the licensee shall provide a fully-trained on-shift technical advisor to the shift engineer (shift supervisor).

(b) Nuclear Steam Supply System Vendor Review of Procedures (I.C.7, SER)

Prior to beginning low-power testing, the licensee must assure that the General Electric review of the power-ascension test procedures has been completed and the General Electric recommendations have been incorporated.

(c) Independent Safety Engineering Group (I.B.1.2, SER)

The licensee shall have an on-site independent engineering group.

(d) Control Room Design Review (I.D.1, SER, SSER #2)

The licensee shall correct the design deficiencies identified in Appendix C of Supplement No. 1 to the Safety Evaluation Report, NUREG-0519 on the schedule prescribed therein.

(e) Training During Low-Power Testing (I.G.I, SER, SSER #2)

At least 4 weeks prior to performing the Special Test, Simulated Loss of Onsite and Offsite Alternating-Current Power Test, the licensee shall provide a safety analysis for the test and its procedures to NRC for review and approval.

(f) Post Accident Sampling (II.B.3, SSER #2)

Prior to criticality, the licensee shall install and test a high radiation sampling system for obtaining reactor coolant and containment atmosphere sampling under degraded core accident conditions without excessive exposure.

(g) Direct Indication of Safety/Relief Valve Position (II.D.3, SER, SSER #2)

Prior to startup after the first refueling outage, the licensee shall replace the safety/relief valve position indicator to a model that meets the IEEE Standards 323-1974 and 344-1975.

(h) Additional Accident-Monitoring Instrumentation (II.F.1, SER, SSER #2)

Attachment 1, Noble Gas Effluent Monitor

Prior to criticality, the licensee shall install and have procedures approved by the NRC for noble gas effluent monitoring system at plant effluent pathways.

Attachment 2, Sampling and Analysis of Plant Effluents

Prior to criticality, the licensee shall install and have procedures approved by the NRC for radioiodine and particulate sampling and analysis system at plant effluent pathways.

(i) Instrumentation for Detection of Inadequate Core Cooling (II.F.2, SER SSER #1, SSER #2)

By July 31, 1982, the licensee shall submit a report addressing the analysis performed by the BWR Owners Group regarding additional instrumentation relative to inadequate core cooling and that the licensee shall implement the staff's requirements after the completion of the staff's review of this report.

(j) Proper Functioning of Heat Removal Systems (II.K.1.22, SER, SSER #2, and II.K.3.13, SER, SSER #2)

The licensee shall implement the logic to restart automatically the core isolation cooling system prior to startup after the first refueling outage.

(k) Modify Break Detection Logic to Prevent Spurious Isolation of High Pressure Coolant Injection and Reactor Core Isolation Cooling System (II.K.3.15, SER, SSER #2)

Prior to startup after the first refueling outage, the licensee shall implement a circuit modification to assure that transients monitored by pressure instruments to sense flow in these two systems actually sense continuous high steam flow.

(l) Modification of Automatic Depressurization System Logic - Feasibility for Increased Diversity for Some Event Sequences (II.K.3.18, SER, SER #1, SSER #3)

(a) By October 1, 1982, the licensee shall evaluate the alternative design modifications of the BWR Owners Group relative to the logic for the automatic depressurization system, submit such evaluation, and propose modification to NRC for review and approval.

(b) Prior to startup after the first refueling outage, the licensee shall implement the approved alternative logic modification of the automatic depressurization system.

(m) Restart of Core Spray and Low Pressure Core Injection System (II.K.3.21, SER, SSER #2)

Prior to startup after the first refueling outage, the licensee shall provide an auto start for the high pressure core spray.

(n) Automatic Switchover of Reactor Core Isolation Cooling System Suction--Verify Procedures and Modify Design (II.K.3.22, SER)

Prior to startup after the first refueling outage, the licensee shall implement the automatic switchover of the reactor core isolation cooling system suction from the condensate storage tank to the suppression pool when the condensate storage tank level is low.

(o) Upgrade Emergency Support Facilities (III.A.1.2, SER, SSER #1)

The licensee shall complete its Emergency Response Facilities as follows:

- (i) Safety Parameter Display System October 1, 1982
- (ii) Emergency Operations Facility October 1, 1982
- (iii) Technical Support Center October 1, 1982

(p) Improving Licensee's Emergency Preparedness - Long Term (III.A.2, SER, SSER #1, SSER #2)

- (1) Prior to exceeding five percent power, the licensee shall complete a successful emergency exercise with the La Salle facility and La Salle County.
- (2) Prior to exceeding five percent power, a test shall be performed to demonstrate an adequate alerting/notification system.
- (3) Prior to exceeding five percent power, the licensee shall demonstrate that the state of offsite preparedness provides assurance that adequate protective measures can and will be taken in the event of a radiological emergency. The use of 10 CFR 50.54(s)(2) to specify a period within which corrective actions must be taken to assure an adequate state of emergency preparedness will include instances where NRC finds that the lack of progress in completion of the procedures in the Federal Emergency Management Agency's proposed rule set forth in 44 CFR Part 350 is an indication that major substantive problems exist in achieving or maintaining an adequate state of preparedness. Any corrective period specified will relate to substantive problems identified by the Federal Emergency Management Agency.
- (4) The licensee shall provide the interim meteorological improvement and shall provide the mechanism for long-term improvements as follows:
 - (i) Prior to exceeding five percent power, the licensee shall install a process computer with the capability to retrieve meteorological information that provides a redundant means for data access.
 - (ii) Prior to exceeding five percent power, the licensee shall propose a plan for meeting the meteorological and dose assessment capability guidance of Appendix 2, NUREG-0654, Revision 1 as follows:

- installation of hardware and software capability described above by July 1, 1982; and
- full operational capability described above by January 1, 1983.

(iii) Prior to exceeding five percent power, the licensee shall include a description of the dose calculational methodology with a Class A transport and diffusion module, and a description of an acceptable meteorological measurement preventative and corrective maintenance program in the radiological emergency plan.

- D. Exemptions from certain requirements of Appendices G, H, J, R and §50.55(a) to 10 CFR Part 50 and 10 CFR Part 73 are described in the Safety Evaluation Report and Supplement No. 1, No. 2 and No. 3 to the Safety Evaluation Report. In addition, an exemption was requested until the completion of the first refueling from the requirements of 10 CFR §70.24 and an exemption from 10 CFR Part 50, Appendix E from performing a full scale exercise within one year before issuance of an operating license, both exemptions are described in Supplement No. 2 of the Safety Evaluation Report. These exemptions are authorized by law and will not endanger life or property or the common defense and security and are otherwise in the public interest. Therefore, these exemptions are hereby granted. The facility will operate, to the extent authorized herein, in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission.
- E. This license is subject to the following additional condition for the protection of the environment:
- Before engaging in additional construction or operational activities which may result in a significant adverse environmental impact that was not evaluated or that is significantly greater than that evaluated in the Final Environmental Statement and its Addendum, the licensee shall provide a written notification to the Director of the Office of Nuclear Reactor Regulation and receive written approval from that office before proceeding with such activities.
- F. The licensee shall notify the Commission, as soon as possible but not later than one hour, of any accident at this facility which could result in an unplanned release of quantities of fission products in excess of allowable limits for normal operation established by the Commission.
- G. The licensee shall have and maintain financial protection of such type and in such amounts as the Commission shall require in accordance with Section 170 of the Atomic Energy Act of 1954, as amended, to cover public liability claims.

H. This license is effective as of the date of issuance and shall expire at midnight on April 17, 2022; provided however that should the Commission, in conjunction with its consideration of the petition for Rulemaking in Docket No. PRM-50-30 determine that the termination date for operating licenses should appropriately run from the date of the issuance of a licensee's construction permit, the expiration date of this license will be September 10, 2013, effective upon notice to the Licensee of the Commission's action in this regard.

FOR THE NUCLEAR REGULATORY COMMISSION

Harold R. Denton, Director
Office of Nuclear Reactor Regulation

Attachment:

- 1. Attachment 1
- 2. Appendix A - Technical Specifications (NUREG-0861)
- 3. Appendix B - Environmental Protection Plan

Date of Issuance:

*See previous concurrences on next page

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F. The licensee shall have and maintain financial protection of such type and in such amounts as the Commission shall require in accordance with Section 170 of the Atomic Energy Act of 1954, as amended, to cover public liability claims.

6. This license is effective as of the date of issuance and shall expire at midnight on April 16, 2022; provided however that should the Commission, in conjunction with its consideration of the petition for Rulemaking in Docket No. PRM-50-30 determine that the termination date for operating licenses should appropriately run from the date of the issuance of a licensee's construction permit, the expiration date of this license will be September 10, 2013, effective upon notice to the Licensee of the Commission's action in this regard.

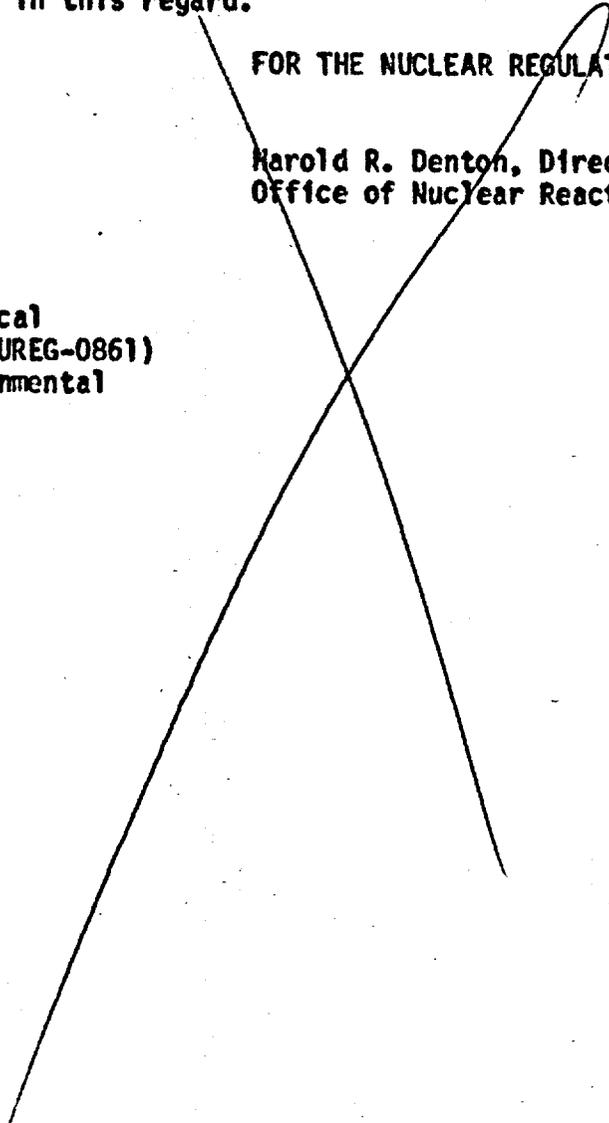
FOR THE NUCLEAR REGULATORY COMMISSION

Harold R. Denton, Director
Office of Nuclear Reactor Regulation

Attachment:

- 1. Attachment 1
- 2. Appendix A - Technical Specifications (NUREG-0861)
- 3. Appendix B - Environmental Protection Plan

Date of Issuance:



By *telecom*

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ATTACHMENT 1 TO LICENSE NPF-11

This attachment identifies certain preoperational tests, system demonstrations and other items which must be completed to the Commission's satisfaction prior to proceeding to Operational Mode 2 (initial criticality or 212°F as applicable). The licensee shall not proceed beyond this Operational Mode without written confirmation from NRC that the following items have been completed in accordance with the conditions set forth below.

1. The following Preoperational Tests shall be completed, including all reviews:
 - a. Containment Monitoring System (PT-CM-101)
 - b. Drywell Pneumatic System (PT-IN-101)
 - c. Traversing Incore Probe (PT-NR-102) (Prior to Entering Test Condition 1)
 - d. Off-Gas System (PT-OG-101)
 - e. Primary Containment ILRT (PT-PC-101)
 - f. Pipe Vibration Monitoring (PT-SI-102)
2. The following System Demonstrations shall be completed, including all reviews:
 - a. High Radiation Sampling System (SD-PS-102)
 - b. Dynamic Effects (SD-SI-101)
3. Commonwealth Edison Company must install and test the microwave voice channel communications system. (373/81-14-25)
4. Commonwealth Edison Company must install radiation measurement equipment in the EOF. (373/81-14-31)
5. Commonwealth Edison Company must assure the following commitments per TMI Action Plan Requirement III.D.3.3 are met:
 - a. Availability of 5 PING-3 (2A special) particulate, iodine, and noble gas air monitoring systems mounted on carts.
 - b. Availability of Eberline Instrument Corporation SAM-2 iodine monitors with silver zeolite cartridges.
 - c. Availability of a low background, low contamination area for analyzing iodine cartridges. (373/81-00-102)

6. Commonwealth Edison Company must assure that results from Preoperational Test PT-RP-101 for response time of the scram signals for the turbine stop valve and turbine control valve fast closure are added to the results obtained in Preoperational Test PT-RR-101 to obtain correct scram response times for these items. (373/80-15-12)
7. Commonwealth Edison Company must conduct a site assembly drill. (373/80-53-03)
8. Commonwealth Edison Company must include in Startup Test Procedure STP-17 requirements for measuring GAP clearances between the process pipe and the surrounding pipe whip restraint structural assemblies. (373/81-29-01)
9. Commonwealth Edison Company must develop procedures for estimating Noble Gas Radioiodine Release Rates required by Table II.F.1-2 per TMI Action Plan Requirement II.F.1. (373/81-00-104)
10. Commonwealth Edison Company must satisfactorily resolve those deficiencies affecting systems for which Preoperational Tests or System Demonstrations are required to be completed prior to initial criticality. (Category 4 deficiencies as per the licensee's procedure).
11. Commonwealth Edison Company must review the Cable Pan Loading Report to verify there are no outstanding discrepancies. (373/79-34-01)
12. Commonwealth Edison Company must calibrate high range station vent and standby gas treatment system monitors. (373/81-14-20)
13. Commonwealth Edison Company must complete modifications to the laboratory HVAC system to meet design criteria requirements of positive pressure between the counting room and surrounding areas. (373/81-43-03)
14. Commonwealth Edison Company must review the stop and control valves closure time acceptance criteria basis to determine it meets design specifications. (373/81-43-06)
15. Commonwealth Edison Company must conduct a 100% reinspection of high strength steel bolting. (373/81-48-06)
16. Commonwealth Edison Company must develop a written technical basis for the practice of installing high strength bolts without torquing requirements. (373/81-48-07)
17. Commonwealth Edison Company must complete a testing program for vibration monitoring of the Low Pressure Core Spray 1A and Residual Heat Removal System 1B pumps. (373/82-10-15 and 373/82-18-01)

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-373

COMMONWEALTH EDISON COMPANY

NOTICE OF ISSUANCE OF FACILITY OPERATION LICENSE

Notice is hereby given that the U.S. Nuclear Regulatory Commission (the Commission), has issued Facility Operating License No. NPF-11, to Commonwealth Edison Company (the licensee) which authorizes operation of the La Salle County Station, Unit 1 (the facility), by Commonwealth Edison Company at reactor core power levels not in excess of 166 megawatts thermal (5 percent power) in accordance with the provisions of the License, the Technical Specifications and the Environmental Protection Plan.

La Salle County Station, Unit 1, is a boiling water nuclear reactor located at the licensee's site in Brookfield Township, La Salle County, Illinois approximately 65 miles southwest of Chicago, Illinois. The license is effective as of the date of issuance.

The application for the license complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations. The Commission has made appropriate findings as required by the Act and the Commission's regulations in 10 CFR Chapter I, which are set forth in the license. Prior public notice of the overall action involving the proposed issuance of an operating license was published in the Federal Register on June 9, 1977 (42 F.R. 29576-29577).

The Commission has determined that the issuance of this license will not result in any environmental impacts other than those evaluated in the Final Environmental Statement and its Addendum since the activity authorized

by the license is encompassed by the overall action evaluated in the Final Environmental Statement and its Addendum.

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For further details with respect to this action, see (1) Facility Operating License No. NPF-11, complete with Technical Specifications and Environmental Protection Plan; (2) the report of the Advisory Committee on Reactor Safeguards dated April 16, 1981; (3) the Commission's Safety Evaluation Report dated March 1981, Supplement No. 1 dated June 1981, Supplement No. 2 dated February 1982, and Supplement No. 3 dated April 1982; (4) the Final Safety Analysis Report and amendments thereto; (5) the Environmental Report and Supplements thereto; (6) the Final Environmental Statement dated November 1978 and the Addendum to the Final Environmental Statement dated May 1981; and (7) NRC Flood Plain Review of La Salle Nuclear Plant Site dated January 29, 1981.

These items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C. 20555, and the Public Library of Illinois Valley Community College, Rural Route No. 1, Oglesby, Illinois 61348. A copy of Facility Operating License No. NPF-11 may be obtained upon request addressed to the U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Licensing. Copies of the Safety Evaluation Report and its Supplements 1, 2 and 3 (NUREG-0519) may be purchased at current rates from the National Technical Information Service, Department of Commerce, 5285 Port Royal Road, Springfield, Virginia 22161, and through the NRC GPO sales program by writing to the U.S. Nuclear Regulatory Commission.

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Attention: Sales Manager, Washington, D.C. 20555. GPO deposit account holders can call (301) 492-9530.

Dated at Bethesda, Maryland, this 17th day of April 1982.

FOR THE NUCLEAR REGULATORY COMMISSION

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A. Schwencer, Chief
Licensing Branch No. 2
Division of Licensing

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UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-373

COMMONWEALTH EDISON COMPANY

NOTICE OF ISSUANCE OF FACILITY OPERATION LICENSE

Notice is hereby given that the U.S. Nuclear Regulatory Commission (the Commission), has issued Facility Operating License No. NPF-11, to Commonwealth Edison Company (the licensee) which authorizes operation of the La Salle County Station, Unit 1 (the facility), by Commonwealth Edison Company at reactor core power levels not in excess of 166 megawatts thermal (5 percent power) in accordance with the provisions of the License, the Technical Specifications and the Environmental Protection Plan.

La Salle County Station, Unit 1, is a boiling water nuclear reactor located at the licensee's site in Brookfield Township, La Salle County, Illinois approximately 65 miles southwest of Chicago, Illinois. The license is effective as of the date of issuance and shall expire at midnight on April , 2022.

The application for the license complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations. The Commission has made appropriate findings as required by the Act and the Commission's regulations in 10 CFR Chapter I, which are set forth in the license. Prior public notice of the overall action involving the proposed issuance of an operating license was published in the Federal Register on June 9, 1977 (42 F.R. 29576-29577).

The Commission has determined that the issuance of this license will not result in any environmental impacts other than those evaluated in the Final Environmental Statement and its Addendum since the activity authorized by the license is encompassed by the overall action evaluated in the Final

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APPENDIX B

TO FACILITY LICENSE NO. NPF-11

LA SALLE COUNTY STATION

UNITS 1 AND 2

COMMONWEALTH EDISON COMPANY

DOCKET NOS. 50-373 AND 50-374

ENVIRONMENTAL PROTECTION PLAN

LA SALLE COUNTY STATION

UNITS 1 AND 2

ENVIRONMENTAL PROTECTION PLAN

(NON-RADIOLOGICAL)

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1.0 OBJECTIVES OF THE ENVIRONMENTAL PROTECTION PLAN

The Environmental Protection Plan (EPP) is to provide for protection of environmental values during construction and operation of the nuclear facility. The principal objectives of the EPP are as follows:

- (1) Verify that the Plant is operated in an environmentally acceptable manner, as established by the FES and other NRC environmental impact assessments.
- (2) Coordinate NRC requirements and maintain consistency with other Federal, State and local requirements for environmental protection.
- (3) Keep NRC informed of the environmental effects of facility construction and operation and of action taken to control those effects.

Environmental concerns identified in the FES which relate to water quality matters are regulated by way of the licensee's NPDES permit.

2.0 ENVIRONMENTAL PROTECTION ISSUES

In the FES-OL dated November 1978, the staff considered the environmental impacts associated with the operation of the two-unit La Salle County Station. Certain environmental issues were identified which required study or license conditions to resolve environmental concerns and to assure adequate protection of the environment.

2.1 Aquatic Issues

Specific aquatic issues raised by the staff in the FES-OL were:

- (1) The need for aquatic monitoring programs to confirm that thermal mixing results in compliance with State water quality standards as predicted, that chlorine releases are controlled within those discharge concentrations evaluated, and that effects on aquatic biota and water quality due to plant operation are no greater than predicted.
- (2) The need for special studies to document levels of intake entrainment and impingement.
- (3) The need for a special study to document the levels of indicator organisms in the cooling lake for the purpose of identifying and defining the presence of potential public health hazard.

(FES-OL: Summary and Conclusions and Sections 6.2 and 6.3)

Aquatic issues identified in items 1 and 2 above are addressed by the effluent limitations, monitoring requirements and the Section 316(b) demonstration requirement contained in the effective NPDES permit issued by the Illinois Environmental Protection Agency. The NRC will rely on this agency for regulation of these matters as they involve water quality and aquatic biota. The aquatic issue identified in item 3 above has been addressed in correspondence between the NRC and the State of Illinois. The State has been appraised of the intention of the NRC not to include this monitoring and mitigation requirement in this facility license. The NRC will rely on the State of Illinois for the establishment and conduct of this program. This action has been taken in accordance with Section 511(c)(2) of the Clean Water Act which places responsibility for establishment and enforcement of programs for the protection of the aquatic environment with the U.S. Environmental Protection Agency or state(s) granted authority for such programs under the Act.

2.2 Terrestrial Issues

- (1) Potential increase in fogging and icing associated with operation of the cooling system involving the cooling lake (FES-0L Section 5.4.1).
- (2) Potential erosion effects along the dike around the cooling lake and the banks of Armstrong Run.

NRC requirements with regard to the terrestrial issues are specified in Subsection 4.2 of this EPP.

3.0 CONSISTENCY REQUIREMENTS

3.1 Plant Design and Operation

The licensee may make changes in plant design or operation or perform tests or experiments affecting the environment provided such changes, tests or experiments do not involve an unreviewed environmental question, and do not involve a change in the Environmental Protection Plan.* Changes in plant design or operation and performance of tests or experiments which do not affect the environment are not subject to the requirements of this EPP. Activities governed by Section 3.3 are not subject to the requirements of this section.

Before engaging in additional construction or operational activities which may affect the environment, the licensee shall prepare and record an environmental evaluation of such activity.** When the evaluation indicates that such activity involves an unreviewed environmental question, the licensee shall provide a written evaluation of such activities and obtain prior approval from the Director, Office of Nuclear Reactor Regulation. When such activity involves a change in the Environmental Protection Plan, such activity and change to the Environmental Protection Plan may be implemented only in accordance with an appropriate license amendment as set forth in Section 5.3.

* This provision does not relieve the licensee of the requirements of 10 CFR 50.59.

**Activities are excluded from this requirement if all measurable environmental effects are confined to on-site areas previously disturbed during site preparation and plant construction.

A proposed change, test or experiment shall be deemed to involve an unreviewed environmental question if it concerns (1) a matter which may result in a significant increase in any adverse environmental impact previously evaluated in the final environmental statement (FES) as modified by staff's testimony to the Atomic Safety and Licensing Board, supplements to the FES, environmental impact appraisals, or in any decisions of the Atomic Safety and Licensing Board; or (2) a significant change in effluents or power level (in accordance with 10 CFR Part 51.5(b)(2)) or (3) a matter not previously reviewed and evaluated in the documents specified in (1) of this Subsection, which may have a significant adverse environmental impact.

The licensee shall maintain records of changes in plant design or operation and of tests and experiments carried out pursuant to this Subsection. These records shall include a written evaluation which provide bases for the determination that the change, test, or experiment does not involve an unreviewed environmental question nor constitute a decrease in the effectiveness of this EPP to meet the objectives specified in Section 1.0. The licensee shall include as part of his Annual Environmental Operating Report (per Subsection 5.4.1) brief descriptions, analyses, interpretations, and evaluations of such changes, tests and experiments.

3.2 Reporting Related to the NPDES Permits and State Certification

Violations of the NPDES permit or the State certification (pursuant to Section 401 of the Clean Water Act, respectively) shall be reported to the NRC by submittal of copies of the reports required by the NPDES permit or certification. The licensee shall also provide the NRC with copies of the results of

the special studies conducted in accordance with the Clean Water Act, at the same time they are submitted to the permitting agency, namely, the Demonstration Study pursuant to Section 316(b) of the Clean Water Act.

Changes and additions to the NPDES permit or the State certification shall be reported to the NRC within 30 days following the date the change is approved. If a permit or certification, in part or in its entirety, is appealed and stayed, the NRC shall be notified within 30 days following the date the stay is granted.

The NRC shall be notified of changes proposed by the licensee to the effective NPDES permit by providing NRC with a copy of the proposed change at the same time it is submitted to the permitting agency. The notification of a licensee-initiated change shall include a copy of the requested revision submitted to the permitting agency. The licensee shall provide the NRC with a copy of its application for renewal of the NPDES permit at the same time the application is submitted to the permitting agency.

3.3 Changes Required for Compliance with Other Environmental Regulations

Changes in plant design or operation and performance of tests or experiments required to achieve compliance with other Federal, State, or local environmental regulations are not subject to requirements of Section 3.1.

4.0 ENVIRONMENTAL CONDITIONS

4.1 Unusual or Important Environmental Events

Any occurrence of an unusual or important event that indicates or could result in significant environmental impact casually related to station operation shall be recorded and promptly reported to the NRC within 24 hours by telephone, telegraph or facsimile transmissions followed by a written report per Subsection 5.4.2. The following are examples: excessive bird impaction events, onsite plant or animal disease outbreaks, mortality or unusual occurrence of any species protected by the Endangered Species Act of 1973, fish kills, increase in nuisance organisms or conditions and unanticipated or emergency discharge of waste water or chemical substances.

No routine monitoring programs are required to implement this condition.

4.2 Environmental Monitoring

4.2.1 Vegetative Integrity on Cooling Pond Dike

A stable vegetative cover will inhibit erosion of the dike during intense rainstorms; therefore, a program of monitoring vegetative integrity and erosion with subsequent remedial actions will minimize erosion damage. Should significant and increasing damage occur, further remedial action may be required.

The monitoring program consists of a visual inspection of the vegetative integrity on the outer face of the peripheral cooling pond dike at the beginning of the spring planting season (May) and at the beginning of the fall planting season (August). If a failure of the vegetative cover is discovered by inspection, the affected area shall be reseeded during the succeeding planting season.

4.2.2 Monitoring of Fog and Ice Due to the Cooling Pond

The results of this monitoring program will be to determine (1) the frequency and density of cooling pond-induced fog on local roads; and (2) the extent of rime ice formation in the vicinity of the pond and to determine whether there is a need for action to mitigate fog or icing effects.

In connection with the training phase described below, locations will be established from and at which observations will be made for occurrence of fog and ice. Observation locations will be chosen to include appropriate sightings of the roadways near the periphery of the cooling pond, nearby residences, and other sensitive locations, if any. Specific roadways to be observed near the cooling pond are County Highway 6, State Route 170, the gravel road north of the cooling pond and County Highway 30.

The observations will be made by company personnel trained to observe and report steam fogging and rime ice conditions near the periphery of the cooling pond under the auspices of a Certified Consulting Meteorologist.

Monitoring shall commence as soon as practical after January 1, 1980, so as to obtain as much of one year's preoperational data as possible. Observations made during one-unit and two-unit operation will be in accordance with the schedule expressed in paragraph six of Subsection 6.2.1 of the LSCNPP Final Environmental Statement (FES). The monitoring data will be summarized and submitted to the NRC quarterly. These data shall be summarized, analyzed, interpreted and compared with the preoperational monitoring data in the annual environmental reports.

As an aid to observations, appropriate landmarks will be identified within sight of each observation location to facilitate the estimation of the extent of (distance covered by) fogging and rime icing and the density of (visibility impairment caused by) fogging. The distances between the landmarks and their associated observation locations will be measured and recorded.

(a) Fog

From each observation location, visual observations will be made and recorded by trained personnel twice each week, before and after the day shift, except weekends and holidays, indicating the presence, location and extent of fog and the estimated local visibility (range). These observations will not be made on days these roads are impassable due to snow or flood.

(b) Ice

From each observation location, visual observations will be made and recorded by trained personnel twice each day, before and after the day shift, except weekends and holidays, from October 1 through March 31, indicating the presence, location, extent and thickness of rime ice. These observations will not be made on days these roads are impassable due to snow or flood.

(c) Supporting Observations

The observations record shall include time, ambient air temperature (dry bulb and dew point), wind direction and speed, electrical load on plant and condensing water temperatures (intake to and discharge from the plant).

(d) Environmental Impact of Rime Icing

As soon as possible after the end of the observation season, but no later than May 1, the data on the occurrence of rime icing will be correlated to determine the locations where environmental impact surveys will be conducted. Such surveys will include observation of vegetation by a terrestrial biologist or botanist during the spring growing season.

5.0 ADMINISTRATIVE PROCEDURES

5.1 Review and Audit

The licensee shall provide for review and audit of compliance with the Environmental Protection Plan. The audits shall be conducted independently of the individual or groups responsible for performing the specific activity. A description of the organization structure utilized to achieve the independent review and audit function and results of the audit activities shall be maintained and made available for inspection.

5.2 Records Retention

Records and logs relative to the environmental aspects of plant operation shall be made and retained in a manner convenient for review and inspection. These records and logs shall be made available to NRC on request.

Records of modifications to plant structures, systems and components determined to potentially affect the continued protection of the environment shall be retained for the life of the plant. All other records, data and logs relating to this EPP shall be retained for five years or, where applicable, in accordance with the requirements of other agencies.

5.3 Changes in Environmental Protection Plan

Requests for changes in the Environmental Protection Plan shall include an assessment of the environmental impact of the proposed change and a supporting justification. Implementation of such changes in the EPP shall not commence prior to NRC approval of the proposed changes in the form of a licensee amendment incorporating the appropriate revision to the Environmental Protection Plan.

5.4 Station Reporting Requirements

5.4.1 Routine Reports

An Annual Environmental Operating Report describing implementation of this EPP for the previous year shall be submitted to the NRC prior to May 1 of each year. The initial report shall be submitted prior to May 1 of the year following issuance of the operating license. The period of the first report shall begin with the date of issuance of the operating license for the first operational unit.

The report shall include summaries and analyses of the results of the environmental protection activities required by Subsection 4.2 of this Environmental Protection Plan for the report period, including a comparison with preoperational studies, operational controls (as appropriate), and previous non-radiological environmental monitoring reports, and an assessment of the observed impacts of the station operation on the environment. If harmful effects or evidence of trends towards irreversible damage to the environment

are observed, the licensee shall provide a detailed analysis of the data and a proposed course of action to alleviate the problem.

The Annual Environmental Operating Report shall also include:

- (a) A list of EPP noncompliances and the corrective actions taken to remedy them.
- (b) A list of all changes in station design or operation, tests, and experiments made in accordance with Subsection 3.1 which involved a potentially significant unreviewed environmental issue.
- (c) A list of nonroutine reports submitted in accordance with Subsection 5.4.2.

In the event that some results are not available by the report due date, the report shall be submitted noting and explaining the missing results. The missing data shall be submitted as soon as possible in a supplementary report.

5.4.2 Nonroutine Reports

A written report shall be submitted to the NRC within 30 days of occurrence of a nonroutine event. The report shall (a) describe, analyze, and evaluate the event, including extent and magnitude of these impact and plant operating characteristics, (b) describe the probable cause of the event, (c) indicate the action taken to correct the reported event, (d) indicate the corrective action taken to preclude repetition of the event and to prevent similar

occurrences involving similar components or systems, and (e) indicate the agencies notified and their preliminary responses.

Events reportable under this subsection which also require reports to other Federal, State, or local agencies shall be reported in accordance with those reporting requirements in lieu of the requirements of this subsection. The NRC shall be provided with a copy of each report at the same time it is submitted to the other agency.