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JUL 9 1982

Docket No.: 50-373

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Mr. Louis O. DelGeorge
Director of Nuclear Licensing
Commonwealth Edison Company
P.O. Box 767
Chicago, Illinois 60690

Dear Mr. DelGeorge:

Subject: Amendment No. 2 to Facility Operating License No. NPF-11 - La Salle County Station, Unit 1

The U.S. Nuclear Regulatory Commission has issued the enclosed Amendment No. 2 to Facility Operating License No. NPF-11 for the La Salle County Station, Unit 1. This Amendment consists of changes to the Technical Specifications in response to your application dated June 14, 1982 and July 2, 1982.

The changes to the Technical Specifications were as follows:

(1) Reduce the count rate on the source range monitors from 3 cps to 0.7 cps for modes other than shutdown, with a minimum allowable value of 0.5 cps instead of 2 cps.

(2) Revise Technical Specification 4.4.3.2.2.6 alarm setpoint for the reactor core isolation cooling system from 60 psig to 90 psig.

By letter dated June 14, 1982, Commonwealth Edison has proposed a change to the Technical Specifications for La Salle County Station, Unit 1. The change would lower the setpoint for the source range monitor down-scale trip from three counts per second to 0.7 counts per second with a minimum allowable value of 0.5 counts per second and define the operability of the source range monitors in terms of 0.7 counts per second. The 0.7 counts per second would be coupled with a procedural requirement for a signal to noise ratio of two or greater. The request is occasioned by the fact that La Salle Unit 1 has an operating license with an upper limit of five percent of full power. At this power, the antimony-beryllium source cannot be regenerated sufficiently to support a three counts per second rate. If the five percent power restriction remains in effect for a sufficient time, the source strength will decay to the point that three counts per second can no longer be achieved.

The source range monitor circuitry is used to provide information on the sub-critical multiplication of the core during startups. It is important to "see" the neutron population in the core, i.e., to be counting neutrons rather than noise, but the count rate is less important. The requirement for a signal to noise ratio greater than two assures that neutrons are being counted. The value of 0.7 counts per second meets the guidance in Regulatory Guide 1.68 (Rev. 2), "Initial Test Programs for Water-Cooled Reactor Power Plants", with respect to minimum count rates for startup.

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The effect of a reduced neutron population ("core power") on the rod drop accident has been addressed by General Electric for the Cooper Station. The expected conclusion that the effect is insignificant was confirmed.

Based on the discussion presented above, we conclude that the proposed changes to the Technical Specifications for La Salle Unit 1 are acceptable.

In a letter dated July 2, 1982, Commonwealth Edison requested a change in the alarm setpoint for the reactor core isolation cooling system high/low pressure interface valve leakage pressure monitor (1E 51-N021) from 60 psig to 90 psig. The alarm is an indication of leakage past the isolation valves which protect the low pressure portions of the reactor core isolation cooling system from the high pressure reactor coolant system. The setpoint correction is necessary because, during reactor operation, the sensing line is normally pressurized above the current alarm level (60 psig) by operation of the reactor core isolation cooling system water leg pump. A revised setpoint of 90 psig is above the water leg pump discharge pressure but is still below the relief valve setpoint of 100 psig.

We have reviewed the process diagrams and the piping and instrumentation diagram for the reactor core isolation cooling system to determine the impact of the proposed change and found that the relief valve in the affected line is set at 100 psig. We therefore conclude that the function of the alarm, i.e., to advise the operator that there is leakage past the isolation valves, will not be affected by changing the alarm setpoint to 90 psig. The request to revise the setpoint to 90 psig is acceptable.

We have determined that the Amendment does not involve a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the Amendment involves an action which is insignificant from the standpoint of environmental impact and pursuant to 10 CFR Section 51.5(d)(4) that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this Amendment.

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the Amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this Amendment will not be inimical to the common defense and security or to the health and safety of the public.

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A copy of a related Notice of Issuance is also enclosed.

Sincerely,

Original signed by:

A. Schwencer, Chief
Licensing Branch No. 2
Division of Licensing

Enclosures:
1. Amendment No. 2 to NPF-11
2. Federal Register Notice

cc w/enclosures:
See next page

OFFICE	DL:LB#2/PM	DL:LB#2/BC	OELD	DL:LB#2/LA			
SURNAME	ABournia:kw	ASchwencer	CWoodhead	EHylton			
DATE	7/9/82	7/9/82	7/9/82	7/9/82			

La Salle

Mr. Louis O. DelGeorge
Director of Nuclear Licensing
Commonwealth Edison Company
P. O. Box 767
Chicago, Illinois 60690

cc: Philip P. Steptoe, Esquire
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Dean Hansell, Esquire
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P. O. Box 224
Marseilles, Illinois 61364

COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-373

LA SALLE COUNTY STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

License No. NPF-11
Amendment No. 2

1. The Nuclear Regulatory Commission (the Commission or the NRC) having found that:

   A. The application for amendment by the Commonwealth Edison Company, dated June 14, 1982 and July 2, 1982, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;

   C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of the Facility Operating License No. NPF-11 is hereby amended to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 2, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

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3. This amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Original signed by:

A. Schwencer, Chief
Licensing Branch No. 2
Division of Licensing

Attachments:
Changes to the Technical
Specifications

Date of Issuance: JUL 9 1982

OFFICE	DL:LB#2/LA	DL:LB#2/PM	DL:LB#2/BC	OELD			
SURNAME	EHylton:kw	ABournia	ASchwencer	CWoodhead			
DATE	7/9/82	7/9/82	7/9/82	7/9/82			

ATTACHMENT TO LICENSE AMENDMENT NO. 2

FACILITY OPERATING LICENSE NO. NPF-11

DOCKET NO. 50-373

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

REMOVE	INSERT
3/4 3-53	3/4 3-53
3/4 3-72	3/4 3-72
3/4 9-4	3/4 9-4
3/4 4-8	3/4 4-8

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TABLE 3.3.6-2

CONTROL ROD WITHDRAWAL BLOCK INSTRUMENTATION SETPOINTS

TRIP FUNCTION	TRIP SETPOINT	ALLOWABLE VALUE
1. ROD BLOCK MONITOR		
a. Upscale	$\leq$ 0.66 W + 40%	$\leq$ 0.66 W + 43%
b. Inoperative	NA	NA
c. Downscale	$\geq$ 5% of RATED THERMAL POWER	$\geq$ 3% of RATED THERMAL POWER
2. APRM		
a. Flow Biased Simulated Thermal Power-Upscale	$\leq$ 0.66 W + 42%*	$\leq$ 0.66 W + 45% *
b. Inoperative	NA	NA
c. Downscale	$\geq$ 5% of RATED THERMAL POWER	$\geq$ 3% of RATED THERMAL POWER
d. Neutron Flux-High	$\leq$ 12% of RATED THERMAL POWER	$\leq$ 14% of RATED THERMAL POWER
3. SOURCE RANGE MONITORS		
a. Detector not full in	NA	NA
b. Upscale	$\leq 2 \times 10^5$ cps	$\leq 5 \times 10^5$ cps
c. Inoperative	NA	NA
d. Downscale	$\geq$ 0.7 cps	$\geq$ 0.5 cps
4. INTERMEDIATE RANGE MONITORS		
a. Detector not full in	NA	NA
b. Upscale	$\leq$ 108/125 of full scale	$\leq$ 110/125 of full scale
c. Inoperative	NA	NA
d. Downscale	$\geq$ 5/125of full scale	$\geq$ 3/125 of full scale
5. SCRAM DISCHARGE VOLUME		
a. Water Level-High	$\leq$ 765' 5¼"	$\leq$ 765' 5¼"
b. Scram Discharge Volume Switch in Bypass	NA	NA
6. REACTOR COOLANT SYSTEM RECIRCULATION FLOW		
a. Upscale	$\leq$ 108/125 of full scale	$\leq$ 111/125 of full scale
b. Inoperative	NA	NA
c. Comparator	$\leq$ 10% flow deviation	$\leq$ 11% flow deviation

*The Average Power Range Monitor rod block function is varied as a function of recirculation loop flow (W). The trip setting of this function must be maintained in accordance with Specification 3.2.2.

SOURCE RANGE MONITORS

LIMITING CONDITION FOR OPERATION

3.3.7.6 At least three source range monitor channels shall be OPERABLE.

APPLICABILITY: OPERATIONAL CONDITIONS 2*, 3 and 4.

ACTION:

a. In OPERATIONAL CONDITION 2* with one of the above required source range monitor channels inoperable, restore at least 3 source range monitor channels to OPERABLE status within 4 hours or be in at least HOT SHUTDOWN within the next 12 hours.

b. In OPERATIONAL CONDITION 3 or 4 with two or more of the above required source range monitor channels inoperable, verify all insertable control rods to be inserted in the core and lock the reactor mode switch in the Shutdown position within one hour.

SURVEILLANCE REQUIREMENTS

4.3.7.6 Each of the above required source range monitor channels shall be demonstrated OPERABLE by:

a. Performance of a:

1. CHANNEL CHECK at least once per:

   a) 12 hours in CONDITION 2*, and

   b) 24 hours in CONDITION 3 or 4.

2. CHANNEL CALIBRATION** at least once per 18 months.

b. Performance of a CHANNEL FUNCTIONAL TEST:

1. Within 24 hours prior to moving the reactor mode switch from the Shutdown position, if not performed within the previous 7 days, and

2. At least once per 31 days.

c. Verifying, prior to withdrawal of control rods, that the SRM count rate is at least 0.7 cps with the detector fully inserted.

*With IRM's on range 2 or below.
**Neutron detectors may be excluded from CHANNEL CALIBRATION.

SURVEILLANCE REQUIREMENTS (Continued)

b. Performance of a CHANNEL FUNCTIONAL TEST:

   1. Within 24 hours prior to the start of CORE ALTERATIONS, and

   2. At least once per 7 days.

c. Verifying that the channel count rate is at least 0.7 cps:

   1. Prior to control rod withdrawal,

   2. Prior to and at least once per 12 hours during CORE ALTERATIONS, and

   3. At least once per 24 hours.

d. Verifying that the RPS circuitry "shorting links" have been removed within 8 hours prior to and at least once per 12 hours during:

   1. The time any control rod is withdrawn,## or

   2. Shutdown margin demonstrations.

##Not required for control rods removed per Specification 3.9.10.1 or 3.9.10.2.

SURVEILLANCE REQUIREMENTS (Continued)

4.4.3.2.2 Each reactor coolant system pressure isolation valve specified in Table 3.4.3.2-1 shall be demonstrated OPERABLE:

a. Pursuant to Specification 4.0.5, except that in lieu of any leakage testing required by Specification 4.0.5, each valve shall be demonstrated OPERABLE by verifying leakage to be within its limit:

   1. At least once per 18 months, and

   2. Prior to returning the valve to service following maintenance, repair or replacement work on the valve which could affect its leakage rate.

   In addition, until the LPCS system and the LPCI system injection valve differential pressure-low permissive is modified during or before the first refueling outage, the LPCS system check valve 1E21-F006 and the LPCI system check valves 1E12-F041 A, B, and C shall also be demonstrated OPERABLE by verifying leakage to be within its limit:

   1. Whenever the unit has been in COLD SHUTDOWN or REFUELING, after the last valve disturbance prior to reactor coolant system temperature exceeding 200°F

   2. Within 24 hours following valve disturbance except when in COLD SHUTDOWN or REFUELING.

   The provisions of Specification 4.0.4 are not applicable for entry into OPERATIONAL CONDITION 3.

b. By demonstrating OPERABILITY of the high/low pressure interface valve leakage pressure monitors by performance of a:

   1. CHANNEL FUNCTIONAL TEST at least once per 31 days, and

   2. CHANNEL CALIBRATION at least once per 18 months,

With the alarm setpoint for the:

   1. HPCS system $\leq$ 100 psig.

   2. LPCS system $\leq$ 500 psig.

   3. LPCI/shutdown cooling system $\leq$ 400 psig.

   4. RHR shutdown cooling $\leq$ 190 psig.

   5. RCIC $\leq$ 90 psig.

7590-01

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-373

COMMONWEALTH EDISON COMPANY

NOTICE OF ISSUANCE OF AMENDMENT OF FACILITY OPERATING LICENSE

The U.S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 2 to Facility Operating License No. NPF-11, issued to Commonwealth Edison Company, which revised Technical Specifications for operation of the La Salle County Station, Unit No. 1 (the facility) located in Brookfield Township, La Salle County, Illinois. The Amendment is effective as of the date of issuance.

The Amendment consists of changes to the Technical Specifications. The changes to the Technical Specifications were as follows: (1) reduce the count rate on the source range monitors from 3 cps to 0.7 cps, with a minimum allowable value of 0.5 cps instead of 2 cps, and (2) revise the alarm setpoint for the reactor core isolation cooling system from 60 psig to 90 psig.

The application for amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this Amendment was not required since the Amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this Amendment will not result in any significant environmental impact and that pursuant to 10 CFR Section 51.5(d)(4) an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this Amendment.

For further details with respect to this action, see (1) the application for amendment dated June 14, 1982 and July 2, 1982, (2) Amendment No. 2 to License NPF-11 dated July 8, 1982. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C. 20555, and the Public Library of Illinois Valley Community College, Rural Route No. 1, Ogelsby, Illinois. A copy of items (1) and (2) may be obtained upon request addressed to the U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 9th day of July 1982.

FOR THE NUCLEAR REGULATORY COMMISSION

JUL 9 1982

A. Schwencer, Chief
Licensing Branch No. 2
Division of Licensing

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SURNAME	EHylton:kw	ABournia	ASchwencer	CWoodhead			
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