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Docket No.: 50-373

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Mr. Louis O. DelGeorge
Director of Nuclear Licensing
Commonwealth Edison Company
P.O. Box 767
Chicago, Illinois 60690

Dear Mr. DelGeorge:

Subject: Amendment No. 5 to Facility Operating License No. NPF-11 -
La Salle County Station, Unit 1

The U.S. Nuclear Regulatory Commission has issued the enclosed Amendment No. 5 to Facility Operating License No. NPF-11 for the La Salle County Station, Unit 1. This Amendment which is effective as of August 13, 1982, consists of changes to the license in response to your application dated August 11, 1982. The changes to the Technical Specifications relate to Specification 4.7.3.C.2 and Bases 3/4.7.3 to account for flow differences between the test flow path and the normal flow path required for an 18 month surveillance on the reactor core isolation cooling (RCIC) system. These changes were approved on August 13, 1982, as indicated in our letter to you on August 16, 1982. In addition, a number of pages are being transmitted to be replaced in the Technical Specifications as a result of typo-corrections made to the snubber numbers in Table 3.7.9-2 and the Y2 term in Table 4.6.1.5-2.

By letter dated August 11, 1982, Commonwealth Edison proposed changes to the Technical Specifications for La Salle County Station, Unit 1 to require that the RCIC system be demonstrated operable at least every 18 months by verifying, using the test flow path, that the system is capable of delivering a flow greater than or equal to 600 gpm to the reactor vessel when steam is supplied to the turbine at a pressure of 150 ± 15 psig. A request was also included to modify Bases 3/4.7.3 to allow the use of initial startup test data for determining equivalent flows in the RCIC test line and vessel injection line. The present specification requires that the RCIC system be demonstrated operable at least every 18 months by verifying that the system will develop a flow greater than or equal to 600 gpm in the test flow path when steam is supplied to the turbine at a pressure of 150 ± 15 psig.

Commonwealth Edison requested these changes because excessive pressure losses in the test line prevent the system from achieving a flow of 600 gpm in the test path when steam pressure is 150 psig. (A flow of 590 gpm was achieved.) Commonwealth Edison calculated the driving pressure required to deliver 600 gpm to the vessel at the point where the test line and the vessel injection line split. Based upon this calculation and the pressure measured at this point during a test in which a test line flow of 590 gpm was achieved, it was shown that a margin of 150 psi is available above the pressure needed to achieve 600

OFFICE	gpm to the vessel. Supporting analyses have been provided which show that
SURNAME	pressure losses in the flow path to the reactor vessel are significantly lower
DATE	than those in the test line. In addition, as part of your startup test program,
	you will verify a flow rate to the reactor of at least 600 gpm when steam at
	150 ± 15 psig is supplied to the turbine.

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USGPO: 1981-335-960

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PDR

On August 13, 1982, we reviewed your request and the supporting analyses and concluded that the reliability of the RCIC system will not be adversely affected by this change. We further concluded that this action will not:

- (1) Involve a significant increase in the probability or consequences of an accident previously evaluated;
- (2) Create the possibility of an accident of a type different from any evaluated previously; or
- (3) Involve a significant reduction in a margin of safety.

The proposed Technical Specification changes were therefore, found to be acceptable.

We have determined that the Amendment does not involve a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the Amendment involves an action which is insignificant from the standpoint of environmental impact and pursuant to 10 CFR Section 51.5(d)(4) that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this Amendment.

We have concluded, based on the considerations discussed above, that:
 (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the Amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this Amendment will not be inimical to the common defense and security or to the health and safety of the public.

A copy of a related Notice of Issuance is also enclosed.

Sincerely,

A. Schwencer, Chief
 Licensing Branch No. 2
 Division of Licensing

Enclosures:

1. Amendment No. 5 to NPF-11
2. Federal Register Notice

cc w/enclosures:

See next page

OFFICE	DL:LB#2/LA	DL:LB#2/PM	DL:LB#2/BC	DELD	
SURNAME	Edgerton:kw	ABournia	ASchwencer		
DATE	8/10/82	8/10/82	8/27/82	8/23/82	

COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-373

LA SALLE COUNTY STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

License No. NPF-11
Amendment No. 5

1. The Nuclear Regulatory Commission (the Commission or the NRC) having found that:
 - A. The application for amendment by the Commonwealth Edison Company, dated August 11, 1982, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of the Facility Operating License No. NPF-11 is hereby amended to read as follows:
 - (2) The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 5, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

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3. This amendment is effective as of August 13, 1982.

FOR THE NUCLEAR REGULATORY COMMISSION

A. Schwencer, Chief
Licensing Branch No. 2
Division of Licensing

Attachments:
Changes to the Technical
Specifications

Date of Issuance:

OFFICE	DL:LB#2/LA	DL:LB#2/PM	DL:LB#2/BC	QELD			
SURNAME	EGGston:kw	ABou:nia	ASchwencer	<i>[Signature]</i>			
DATE	8/20/82	8/20/82	8/27/82	8/13/82			

ATTACHMENT TO LICENSE AMENDMENT NO. 5

FACILITY OPERATING LICENSE NO. NPF-11

DOCKET NO. 50-373

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

REMOVE

3/4 7-8
B 3/4 7-1
3/4 6-12a
3/4 7-33
3/4 7-35
3/4 7-36
3/4 7-37
3/4 7-40
3/4 7-42
3/4 7-43
3/4 7-45

INSERT

3/4 7-8
B 3/4 7-1
3/4 6-12a
3/4 7-33
3/4 7-35
3/4 7-36
3/4 7-37
3/4 7-40
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3/4 7-43
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PLANT SYSTEMS

SURVEILLANCE REQUIREMENTS

- c. At least once per 18 months by:
 - 1. Performing a system functional test which includes simulated automatic actuation and verifying that each automatic valve in the flow path actuates to its correct position, but may exclude actual injection of coolant into the reactor vessel.
 - 2. Verifying that the system is capable of providing a flow of greater than or equal to 600 gpm to the reactor vessel when steam is supplied to the turbine at a pressure of 150 ± 15 psig using the test flow path.
 - 3. Performing a CHANNEL CALIBRATION of the discharge line "keep filled" pressure alarm instrumentation and verifying the low pressure setpoint to be ≥ 62 psig.
- d. By demonstrating MCC-121y and the 250-volt battery** and charger** OPERABLE:
 - 1. At least once per 7 days by verifying that:
 - a) MCC-121y is energized, and has correct breaker alignment, indicated power availability from the charger and battery, and voltage on the panel with an overall voltage of greater than or equal to 250 volts.
 - b) The electrolyte level of each pilot cell is above the plates,
 - c) The pilot cell specific gravity, corrected to 77°F, is greater than or equal to 1.200, and
 - d) The overall battery voltage is greater than or equal to 250 volts.
 - 2. At least once per 92 days by verifying that:
 - a) The voltage of each connected battery is greater than or equal to 250 volts under float charge and has not decreased more than 12 volts from the value observed during the original test,
 - b) The specific gravity, corrected to 77°F, of each connected cell is greater than or equal to 1.195 and has not decreased more than 0.05 from the value observed during the previous test, and
 - c) The electrolyte level of each connected cell is above the plates.
 - 3. At least once per 18 months by verifying that:
 - a) The battery shows no visual indication of physical damage or abnormal deterioration, and
 - b) Battery terminal connections are clean, tight, free of corrosion and coated with anti-corrosion material.

*The provisions of Specification 4.0.4 are not applicably provided the surveillance is performed within 12 hours after reactor steam pressure is adequate to perform the tests.

**The Unit 2 RCIC system battery and charger may be in service to supply Unit 1 RCIC system provided that they are demonstrated OPERABLE per Unit 1 Technical Specifications. This footnote shall be deleted upon issuance of an Operating License for Unit 2.

3/4.7 PLANT SYSTEMS

BASES

3/4.7.1 CORE STANDBY COOLING SYSTEM - EQUIPMENT COOLING WATER SYSTEMS

The OPERABILITY of the core standby cooling system - equipment cooling water systems and the ultimate heat sink ensure that sufficient cooling capacity is available for continued operation of safety-related equipment during normal and accident conditions. The redundant cooling capacity of these systems, assuming a single failure, is consistent with the assumptions used in the accident conditions within acceptable limits.

3/4.7.2 CONTROL ROOM AND AUXILIARY ELECTRIC EQUIPMENT ROOM EMERGENCY FILTRATION SYSTEM

The OPERABILITY of the control room and auxiliary electric equipment room emergency filtration system ensures that the rooms will remain habitable for operations personnel during and following all design basis accident conditions. The OPERABILITY of this system in conjunction with room design provisions is based on limiting the radiation exposure to personnel occupying the rooms to 5 rem or less whole body, or its equivalent. This limitation is consistent with the requirements of General Design Criteria 19 of Appendix "A", 10 CFR Part 50. Cumulative operation of the system with the heaters OPERABLE for 10 hours over a 31 day period is sufficient to reduce the buildup of moisture on the adsorbers and HEPA filters.

3/4.7.3 REACTOR CORE ISOLATION COOLING SYSTEM

The reactor core isolation cooling (RCIC) system is provided to assure adequate core cooling in the event of reactor isolation from its primary heat sink and the loss of feedwater flow to the reactor vessel without requiring actuation of any of the Emergency Core Cooling System equipment. The RCIC system is conservatively required to be OPERABLE whenever reactor pressure exceeds 150 psig even though the LPCI mode of the the residual heat removal (RHR) system provides adequate core cooling up to 350 psig.

The RCIC system specifications are applicable during OPERATIONAL CONDITIONS 1, 2 and 3 when reactor vessel pressure exceeds 150 psig because RCIC is the primary non-ECCS source of core cooling when the reactor is pressurized.

With the RCIC system inoperable, adequate core cooling is assured by the OPERABILITY of the HPCS system and justifies the specified 14 day out-of-service period.

The surveillance requirements provide adequate assurance that RCICS will be OPERABLE when required. Although all active components are testable and full flow can be demonstrated by recirculation during reactor operation, a complete functional test requires reactor shutdown. Initial startup test program data may be used to determine equivalent turbine/pump capabilities between test flow path and the vessel injection flow path. The pump discharge piping is maintained full to prevent water hammer damage and to start cooling at the earliest possible moment.

Table 4.6.1.5-2 (Continued)

TENDON LIFT-OFF FORCE

HOOP TENDONS

Tendon Number	Ends	First Year*		Y1	Y2
		Maximum (kips)	Minimum (kips)		
48AC	A	N/A	647.00	N/A	4.500
	B	N/A	647.00	N/A	4.500
3BA	A	N/A	656.46	N/A	4.226
	B	N/A	656.46	N/A	4.226
12BA	A	N/A	637.48	N/A	6.173
	B	N/A	637.48	N/A	6.173
21CB	A	N/A	637.48	N/A	6.173
	B	N/A	637.48	N/A	6.173
23BA	A	N/A	629.71	N/A	6.173
	B	N/A	629.71	N/A	6.173
38CB	A	N/A	631.76	N/A	5.437
	B	N/A	631.76	N/A	5.437
49AC	A	N/A	647.00	N/A	4.500
	B	N/A	647.00	N/A	4.500
68B	A	N/A	655.39	N/A	4.332
	B	N/A	655.39	N/A	4.332
4BA	A	N/A	651.16	N/A	4.226
	B	N/A	651.16	N/A	4.226
41CB	A	N/A	644.51	N/A	4.975
	B	N/A	644.51	N/A	4.975
50AC	A	N/A	650.35	N/A	4.500
	B	N/A	650.35	N/A	4.500
50CB	A	N/A	650.35	N/A	4.500
	B	N/A	650.35	N/A	4.500
53BA	A	N/A	649.82	N/A	4.538
	B	N/A	649.82	N/A	4.538
57AC	A	N/A	650.14	N/A	4.862
	B	N/A	650.14	N/A	4.862
39CB	A	N/A	644.69	N/A	5.437
	B	N/A	644.69	N/A	5.437
49BA	A	N/A	647.00	N/A	4.500
	B	N/A	647.00	N/A	4.500
71D	A	N/A	645.20	N/A	4.332
	B	N/A	645.20	N/A	4.332
1BA	A	N/A	655.82	N/A	3.914
	B	N/A	655.82	N/A	3.914
47AC	A	N/A	644.51	N/A	4.975
	B	N/A	644.51	N/A	4.975
57BA	A	N/A	650.18	N/A	4.862
	B	N/A	650.18	N/A	4.862
48CB	A	N/A	646.48	N/A	4.507
	B	N/A	646.48	N/A	4.507
51AC	A	N/A	653.75	N/A	4.507
	B	N/A	653.75	N/A	4.507

*First Inspection

Table 3.7.9-2 (Continued)

Snubber No.	System snubber installed on location and elev.			Snubber No.	System snubber installed on location and elev.		
HP02-1524S	HP	Containment	690	LC01-1057S	LC	Auxiliary	681
HP02-1800S	HP	Reactor	686	LC01-1058S	LC	Auxiliary	681
HP08-1009S	HP	Reactor	704	LC01-1059S	LC	Auxiliary	706
HP08-1024S	HP	Reactor	699	LC01-1065S	LC	Auxiliary	722
HP08-1026S	HP	Reactor	700	LC01-1066S	LC	Auxiliary	706
HP08-1028S	HP	Reactor	699	LC01-1068S	LC	Auxiliary	722
HP08-1029S	HP	Containment	703	LC01-1069S	LC	Auxiliary	722
HP09-1013S	HP	Reactor	688	LC01-1070S	LC	Auxiliary	721
HP09-1036S	HP	Reactor	693	LC01-1089S	LC	Auxiliary	732
HP14-1003S	HP	Reactor	681	LC01-1096S	LC	Auxiliary	733
HP22-1003S	HP	Reactor	768	LC01-1106S	LC	Auxiliary	731
HP75-1003S	HP	Diesel Generator	735	LC01-1120S	LC	Auxiliary	732
HP75-1005S	HP	Diesel Generator	746	LC01-1121S	LC	Auxiliary	732
LC01-1002S	LC	Auxiliary	707	LC01-1123S	LC	Auxiliary	732
LC01-1003S	LC	Auxiliary	707	LC01-1126S	LC	Auxiliary	732
LC01-1004S	LC	Auxiliary	706	LC01-1807S	LC	Reactor	680
LC01-1005S	LC	Auxiliary	706	LC01-1808S	LC	Reactor	680
LC01-1007S	LC	Auxiliary	707	LC01-1809S	LC	Reactor	680
LC01-1008S	LC	Auxiliary	707	LC01-1810S	LC	Reactor	680
LC01-1013S	LC	Auxiliary	706	LC03-1817S	LC	Reactor	680
LC01-1014S	LC	Auxiliary	706	LC03-1820S	LC	Reactor	680
LC01-1015S	LC	Auxiliary	690	LC03-1823S	LC	Reactor	680
LC01-1019S	LC	Auxiliary	706	LC03-1826S	LC	Reactor	680
LC01-1021S	LC	Auxiliary	681	LC09-1001S	LC	Reactor	681
LC01-1022S	LC	Auxiliary	706	LC09-1013S	LC	Auxiliary	679
LC09-1018S	LC	Auxiliary	681	MS00-1015S	MS	Containment	780
LC09-1031S	LC	Auxiliary	702	MS00-1016S	MS	Containment	780
LC09-1036S	LC	Auxiliary	702	MS00-1017S	MS	Containment	775
LC09-1049S	LC	Auxiliary	681	MS00-1019S	MS	Containment	763
LC09-1050S	LC	Auxiliary	681	MS00-1020S	MS	Containment	743
LC09-1052S	LC	Auxiliary	681	MS00-1021S	MS	Containment	743
LC09-1053S	LC	Auxiliary	737	MS00-1022S	MS	Containment	743
LP01-1014S	LP	Reactor	681	MS00-1023S	MS	Containment	769
LP02-1013S	LP	Containment	778	MS00-1024S	MS	Containment	786
LP02-1015S	LP	Containment	774	MS00-1025S	MS	Containment	787
LP02-1016S	LP	Containment	774	MS00-1026S	MS	Containment	780
LP02-1017S	LP	Containment	774	MS00-1029S	MS	Containment	743
LP02-1018S	LP	Containment	774	MS00-1030S	MS	Containment	772
LP02-1019S	LP	Containment	774	MS00-1031S	MS	Containment	785
LP02-1020S	LP	Reactor	777	MS00-1032S	MS	Containment	790
LP02-1025S	LP	Reactor	703	MS00-1033S	MS	Containment	790
LP02-1054S	LP	Reactor	777	MS00-1034S	MS	Containment	780
LP02-1055S	LP	Reactor	777	MS00-1038S	MS	Containment	743
LP02-1057S	LP	Reactor	777	MS00-1039S	MS	Containment	743
LP02-1059S	LP	Reactor	777	MS00-1040S	MS	Containment	772
LP02-1062S	LP	Reactor	689	MS00-1041S	MS	Containment	786
LP02-1067S	LP	Containment	788	MS00-1042S	MS	Containment	790
LP19-1011S	LP	Reactor	763	MS00-1043S	MS	Containment	790
LP20-1025S	LP	Reactor	733	MS00-1044S	MS	Containment	780

Table 3.7.9-2 (Continued)

Snubber No.	System snubber installed on location and elev.			Snubber No.	System snubber installed on location and elev.		
MS04-1245S	MS	Containment	780	MS04-1298S	MS	Containment	761
MS04-1246S	MS	Containment	779	MS04-1299S	MS	Containment	770
MS04-1248S	MS	Containment	752	MS04-1300S	MS	Containment	772
MS04-1249S	MS	Containment	752	MS04-1301S	MS	Containment	772
MS04-1250S	MS	Containment	771	MS04-1302S	MS	Containment	781
MS04-1251S	MS	Containment	771	MS04-1304S	MS	Containment	781
MS04-1252S	MS	Containment	771	MS04-1305S	MS	Containment	774
MS04-1253S	MS	Containment	778	MS04-1307S	MS	Containment	771
MS04-1256S	MS	Containment	766	MS04-1308S	MS	Containment	773
MS04-1258S	MS	Containment	761	MS04-1320S	MS	Containment	746
MS04-1259S	MS	Containment	780	MS04-1321S	MS	Containment	768
MS04-1261S	MS	Containment	774	MS04-1322S	MS	Containment	766
MS04-1262S	MS	Containment	748	MS04-1323S	MS	Containment	766
MS04-1265S	MS	Containment	745	MS04-1324S	MS	Containment	749
MS04-1266S	MS	Containment	764	MS04-1325S	MS	Containment	766
MS04-1267S	MS	Containment	767	MS04-1328S	MS	Containment	751
MS04-1269S	MS	Containment	780	MS04-1329S	MS	Containment	772
MS04-1270S	MS	Containment	781	MS04-1330S	MS	Containment	772
MS04-1271S	MS	Containment	747	MS04-1331S	MS	Containment	774
MS04-1272S	MS	Containment	747	MS04-1332S	MS	Containment	745
MS04-1274S	MS	Containment	769	MS04-1333S	MS	Containment	748
MS04-1275S	MS	Containment	769	MS04-1334S	MS	Containment	774
MS04-1276S	MS	Containment	772	MS04-1337S	MS	Containment	749
MS04-1277S	MS	Containment	773	MS04-1341S	MS	Containment	748
MS04-1278S	MS	Containment	773	MS04-1344S	MS	Containment	782
MS04-1279S	MS	Containment	780	MS04-1345S	MS	Containment	743
MS04-1281S	MS	Containment	773	MS04-1346S	MS	Containment	743
MS04-1282S	MS	Containment	753	MS04-1347S	MS	Containment	757
MS04-1283S	MS	Containment	751	MS04-1349S	MS	Containment	742
MS04-1288S	MS	Containment	770	MS04-1350S	MS	Containment	759
MS04-1289S	MS	Containment	773	MS04-1351S	MS	Containment	758
MS04-1290S	MS	Containment	772	MS04-1353S	MS	Containment	749
MS04-1291S	MS	Containment	780	MS04-1354S	MS	Containment	749
MS04-1293S	MS	Containment	781	MS04-1355S	MS	Containment	759
MS04-1294S	MS	Containment	744	MS04-1356S	MS	Containment	766
MS04-1358S	MS	Containment	775	MS04-1546S	MS	Containment	760
MS04-1359S	MS	Containment	780	MS04-1549S	MS	Containment	760
MS04-1362S	MS	Containment	751	MS04-1550S	MS	Containment	760
MS04-1363S	MS	Containment	759	MS04-1551S	MS	Containment	781
MS04-1364S	MS	Containment	759	MS04-1553S	MS	Containment	748
MS04-1366S	MS	Containment	757	MS04-1683S	MS	Containment	769
MS04-1367S	MS	Containment	775	MS04-1705S	MS	Containment	751
MS04-1368S	MS	Containment	743	MS05-1002S	MS	Containment	810
MS04-1369S	MS	Containment	779	MS05-1004S	MS	Containment	804
MS04-1371S	MS	Containment	783	MS05-1005S	MS	Containment	804
MS04-1372S	MS	Containment	749	MS05-1008S	MS	Containment	804
MS04-1373S	MS	Containment	766	MS05-1010S	MS	Containment	804
MS04-1376S	MS	Containment	745	MS06-1006S	MS	Containment	787
MS04-1378S	MS	Containment	761	MS06-1016S	MS	Containment	776
MS04-1379S	MS	Containment	760	MS06-1017S	MS	Containment	779

Table 3.7.9-2 (Continued)

Snubber No.	System snubber installed on location and elev.			Snubber No.	System snubber installed on location and elev.		
MS04-1384S	MS	Containment	749	MS10-1002S	MS	Containment	761
MS04-1390S	MS	Containment	783	MS10-1007S	MS	Containment	761
MS04-1391S	MS	Containment	783	MS10-1013S	MS	Containment	759
MS04-1392S	MS	Containment	783	MS14-1030S	MS	Containment	738
MS04-1393S	MS	Containment	783	MS14-1031S	MS	Containment	738
MS04-1394S	MS	Containment	751	MS14-1034S	MS	Containment	738
MS04-1395S	MS	Containment	751	MS14-1037S	MS	Containment	739
MS04-1396S	MS	Containment	747	MS14-1038S	MS	Containment	738
MS04-1397S	MS	Containment	744	MS14-1039S	MS	Containment	739
MS04-1398S	MS	Containment	758	MS14-1044S	MS	Containment	739
MS04-1399S	MS	Containment	760	MS14-1047S	MS	Containment	740
MS04-1502S	MS	Containment	760	MS14-1048S	MS	Containment	741
MS04-1503S	MS	Containment	763	MS14-1050S	MS	Containment	739
MS04-1504S	MS	Containment	771	MS14-1051S	MS	Containment	740
MS04-1505S	MS	Containment	770	MS14-1052S	MS	Containment	741
MS04-1506S	MS	Containment	758	MS14-1054S	MS	Containment	741
MS04-1508S	MS	Containment	748	MS14-1055S	MS	Containment	741
MS04-1509S	MS	Containment	756	MS14-1056S	MS	Containment	741
MS04-1510S	MS	Containment	749	MS14-1058S	MS	Containment	741
MS04-1511S	MS	Containment	760	MS14-1059S	MS	Containment	741
MS04-1513S	MS	Containment	761	MS14-1063S	MS	Containment	739
MS14-1066S	MS	Containment	741	NB15-1002S	NB	Containment	830
				NB15-1005S	NB	Containment	828
MS25-1023S	MS	Auxiliary	689	NB15-1008S	NB	Containment	827
MS25-1061S	MS	Auxiliary	689	NB16-1002S	NB	Containment	810
MS88-1005S	MS	Containment	787	NB16-1005S	NB	Containment	810
MS88-1006S	MS	Containment	787	NB16-1006S	NB	Containment	809
MSB8-1011S	MS	Reactor	740	NB23-1003S	NB	Containment	808
MSB8-1012S	MS	Reactor	740	NB25-1002S	NB	Containment	808
MSB8-1013S	MS	Reactor	740	PC01-1014S	PC	Reactor	741
MSB8-1015S	MS	Reactor	740	PC01-1015S	PC	Reactor	741
MSB8-1020S	MS	Reactor	740	PC01-1016S	PC	Reactor	741
MSC6-1003S	MS	Containment	761	PC01-1017S	PC	Reactor	741
MSC6-1004S	MS	Reactor	761	PC01-1019S	PC	Reactor	747
MSC6-1005S	MS	Containment	760	PC01-1020S	PC	Reactor	747
MSC6-1006S	MS	Reactor	761	PC01-1800S	PC	Reactor	751
MSC6-1009S	MS	Containment	761	RG10-0014S	RG	Auxiliary	795
MSC6-1013S	MS	Containment	760	RG21-0012S	RG	Auxiliary	797
MSC6-1015S	MS	Containment	760	RH01-1005S	RH	Reactor	675
MSC6-1016S	MS	Containment	761	RH01-1006S	RH	Reactor	675
MSC6-1018S	MS	Containment	760	RH01-1008S	RH	Reactor	681
MSC6-1021S	MS	Containment	760	RH01-1013S	RH	Reactor	681
MSC6-1024S	MS	Containment	761	RH01-1017S	RH	Reactor	681
MSD1-1001S	MS	Containment	787	RH01-1018S	RH	Reactor	681
MSF9-1002S	MS	Containment	785	RH01-1025S	RH	Reactor	678
MSF9-1004S	MS	Containment	784	RH02-1010S	RH	Reactor	705
MSF9-1006S	MS	Containment	787	RH02-1012S	RH	Reactor	686

Table 3.7.9-2 (Continued)

Snubber No.	System snubber installed on location and elev.			Snubber No.	System snubber installed on location and elev.		
MSF9-1007S	MS	Containment	788	RH02-1017S	RH	Reactor	732
NB11-1003S	NB	Containment	808	RH02-1018S	RH	Reactor	686
NB13-1001S	NB	Containment	832	RH02-1019S	RH	Reactor	686
NB13-1002S	NB	Containment	832	RH02-1024S	RH	Reactor	735
NB13-1004S	NB	Containment	829	RH02-1025S	RH	Reactor	695
NB13-1006S	NB	Containment	828	RH02-1026S	RH	Reactor	703
NB13-1025S	NB	Containment	813	RH02-1027S	RH	Reactor	695
NB13-1027S	NB	Containment	811	RH02-1043S	RH	Reactor	735
NB13-1028S	NB	Containment	814	RH02-1047S	RH	Reactor	725
NB13-1031S	NB	Containment	811	RH02-1048S	RH	Reactor	722
RH02-1051S	RH	Reactor	732	RH03-1517S	RH	Containment	741
RH02-1052S	RH	Reactor	712	RH03-1524S	RH	Reactor	733
RH02-1056S	RH	Reactor	710	RH03-1525S	RH	Reactor	736
RH02-1057S	RH	Reactor	696	RH03-1526S	RH	Reactor	736
RH02-1058S	RH	Reactor	686	RH03-1527S	RH	Reactor	700
RH02-1060S	RH	Reactor	710	RH03-1528S	RH	Reactor	730
RH02-1062S	RH	Reactor	715	RH03-1530S	RH	Reactor	736
RH02-1063S	RH	Reactor	727	RH03-1532S	RH	Reactor	736
RH02-1064S	RH	Reactor	696	RH03-1533S	RH	Reactor	715
RH02-1065S	RH	Reactor	685	RH03-1534S	RH	Reactor	736
RH02-1067S	RH	Reactor	683	RH03-1537S	RH	Reactor	734
RH03-1034S	RH	Reactor	729	RH03-1540S	RH	Reactor	718
RH03-1035S	RH	Reactor	733	RH03-1541S	RH	Reactor	700
RH03-1036S	RH	Reactor	719	RH03-1544S	RH	Containment	738
RH03-1037S	RH	Reactor	717	RH04-1020S	RH	Reactor	682
RH03-1038S	RH	Reactor	718	RH04-1021S	RH	Reactor	681
RH03-1044S	RH	Reactor	734	RH04-1022S	RH	Reactor	700
RH03-1046S	RH	Reactor	703	RH04-1023S	RH	Reactor	703
RH03-1047S	RH	Reactor	736	RH04-1024S	RH	Reactor	703
RH03-1049S	RH	Reactor	700	RH04-1025S	RH	Reactor	703
RH03-1051S	RH	Reactor	729	RH04-1027S	RH	Reactor	703
RH03-1500S	RH	Containment	738	RH04-1028S	RH	Reactor	703
RH03-1502S	RH	Containment	738	RH04-1029S	RH	Reactor	703
RH03-1503S	RH	Containment	738	RH04-1031S	RH	Reactor	703
RH03-1504S	RH	Containment	738	RH04-1032S	RH	Reactor	703
RH03-1505S	RH	Containment	740	RH04-1033S	RH	Reactor	703
RH03-1506S	RH	Containment	742	RH04-1035S	RH	Reactor	700
RH03-1507S	RH	Containment	738	RH04-1036S	RH	Reactor	692
RH03-1508S	RH	Containment	738	RH04-1038S	RH	Reactor	688
RH03-1509S	RH	Containment	738	RH04-1039S	RH	Reactor	688
RH03-1511S	RH	Containment	738	RH04-1040S	RH	Reactor	688
RH03-1512S	RH	Containment	738	RH04-1044S	RH	Reactor	682
RH03-1513S	RH	Containment	738	RH04-1045S	RH	Reactor	682
RH03-1514S	RH	Containment	738	RH04-1046S	RH	Reactor	688
RH03-1515S	RH	Containment	741	RH04-1051S	RH	Reactor	682
RH03-1516S	RH	Containment	741	RH04-1416S	RH	Containment	738
RH04-1418S	RH	Containment	738	RH12-1071S	RH	Reactor	688
RH04-1420S	RH	Containment	738	RH12-1072S	RH	Reactor	697
RH04-1500S	RH	Containment	738	RH12-1077S	RH	Reactor	682

Table 3.7.9-2 (Continued)

Snubber No.	System snubber installed on location and elev.		Snubber No.	System snubber installed on location and elev.	
RH40-1560S	RH	Containment	794	RH53-1574S	RH Reactor 771
RH40-1561S	RH	Containment	799	RH53-1575S	RH Reactor 771
RH40-1572S	RH	Reactor	734	RH56-1003S	RH Reactor 688
RH40-1573S	RH	Reactor	728	RH56-1007S	RH Reactor 688
RH41-1091S	RH	Reactor	708	RH59-1030S	RH Reactor 762
RH42-1032S	RH	Reactor	705	RH59-1031S	RH Reactor 762
RH42-1033S	RH	Reactor	706	RH59-1048S	RH Reactor 758
RH42-1037S	RH	Reactor	735	RH59-1049S	RH Reactor 738
RH59-1052S	RH	Reactor	758	RI01-1072S	RI Reactor 702
RH59-1056S	RH	Reactor	754	RI01-1073S	RI Reactor 701
RH82-1030S	RH	Reactor	708	RI01-1074S	RI Reactor 701
RH82-1037S	RH	Reactor	687	RI01-1076S	RI Reactor 692
RH82-1038S	RH	Reactor	686	RI01-1077S	RI Reactor 693
RH82-1040S	RH	Reactor	686	RI01-1080S	RI Reactor 685
RH82-1041S	RH	Reactor	687	RI01-1081S	RI Reactor 685
RH82-1046S	RH	Reactor	695	RI01-1083S	RI Reactor 676
RH82-1068S	RH	Auxiliary	698	RI01-1084S	RI Reactor 678
RH82-1074S	RH	Auxiliary	698	RI01-1085S	RI Reactor 678
RH83-1011S	RH	Reactor	689	RI01-1088S	RI Reactor 679
RH83-1014S	RH	Reactor	689	RI01-1089S	RI Reactor 683
RH83-1015S	RH	Reactor	689	RI01-1090S	RI Reactor 683
RH83-1016S	RH	Reactor	684	RI01-1091S	RI Reactor 683
RH83-1017S	RH	Reactor	684	RI01-1092S	RI Reactor 677
RH83-1018S	RH	Reactor	684	RI01-1093S	RI Reactor 678
RH83-1037S	RH	Reactor	689	RI01-1101S	RI Containment 718
RHA6-1003S	RH	Reactor	715	RI01-1102S	RI Containment 718
RHB4-1002S	RH	Containment	738	RI01-1103S	RI Containment 750
RHB4-1005S	RH	Containment	738	RI01-1106S	RI Containment 776
RHB4-1007S	RH	Containment	742	RI01-1108S	RI Containment 779
RHB4-1008S	RH	Containment	742	RI02-1007S	RI Reactor 680
RHB4-1011S	RH	Containment	745	RI02-1009S	RI Reactor 687
RI01-1006S	RI	Containment	753	RI09-1005S	RI Containment 742
RI01-1007S	RI	Containment	771	RI09-1007S	RI Containment 742
RI01-1008S	RI	Containment	769	RI09-1008S	RI Containment 747
RI01-1009S	RI	Containment	769	RI09-1009S	RI Containment 747
RI01-1010S	RI	Containment	775	RI09-1011S	RI Containment 743
RI01-1011S	RI	Containment	775	RI09-1016S	RI Containment 743
RI01-1012S	RI	Containment	774	RI09-1021S	RI Containment 743
RI01-1063S	RI	Reactor	743	RI09-1024S	RI Containment 747
RI01-1064S	RI	Reactor	743	RI09-1025S	RI Containment 746
RI01-1065S	RI	Reactor	744	RI09-1026S	RI Containment 747
RI01-1067S	RI	Reactor	740	RI09-1027S	RI Containment 743
RI01-1069S	RI	Reactor	736	RI16-1016S	RI Reactor 688
RI01-1070S	RI	Reactor	736	RI16-1022S	RI Reactor 683
RI16-1023S	RI	Reactor	683	RI41-1067S	RI Reactor 703
RI16-1025S	RI	Reactor	687	RI41-1068S	RI Reactor 703
RI24-1015S	RI	Containment	793	RI41-1089S	RI Reactor 703
RI24-1016S	RI	Containment	793	RI41-1092S	RI Reactor 711

Table 3.7.9-2 (Continued)

Snubber No.	System snubber installed on location and elev.		Snubber No.	System snubber installed on location and elev.	
RR00-1057S	RR	Containment	765	SC02-1027S	SC Reactor 781
RR00-1058S	RR	Containment	739	SC02-1036S	SC Containment 774
RR00-1059S	RR	Containment	743	SC02-1038S	SC Containment 770
RR00-1060S	RR	Containment	746	SC02-1047S	SC Containment 757
RR00-1061S	RR	Containment	746	SC02-1055S	SC Containment 750
RR00-1062S	RR	Containment	748	VG01-0001S	VG Reactor 831
RR01-1032S	RR	Containment	743	VG01-0005S	VG Reactor 831
RR07-1432S	RR	Containment	738	VG01-0006S	VG Reactor 831
RR17-1001S	RR	Containment	738	VG01-0008S	VG Reactor 831
RR17-1002S	RR	Containment	737	VG01-0010S	VG Reactor 831
RR17-1003S	RR	Containment	737	VG02-1004S	VG Reactor 794
RR17-1004S	RR	Containment	737	VG02-1005S	VG Reactor 794
RR17-1005S	RR	Containment	737	VG02-1800S	VG Reactor 802
RR17-1006S	RR	Containment	737	VG04-1003S	VG Reactor 821
RR17-1007S	RR	Containment	737	VG04-1005S	VG Reactor 809
RR17-1008S	RR	Containment	737	VG04-1011S	VG Reactor 794
RR28-1007S	RR	Containment	756	VG04-1014S	VG Reactor 794
RR28-1012S	RR	Containment	750	VG04-1015S	VG Reactor 798
RR28-1015S	RR	Containment	741	VG04-1019S	VG Reactor 798
VQ02-1035S	VQ	Reactor	805	LP-25-H04S	LP
VQ02-1040S	VQ	Reactor	810	N.P. 30A	RR-59
VQ05-1001S	VQ	Reactor	733	RR68-H-4	RR
VQ05-1009S	VQ	Reactor	738	RR68-H-6	RR
				N.P. 83	RR-59
				N.P. 85	RR-59
				RR69-H-4	RR
				N.P. 164	RR-59
				N.P. 170	RR-59
				N.P. 175	RR-59
				N.P. 180	RR-59
				N.P. 240	RR-59
				HG-08-H03S	HG
				HG-08-H08S	HG
				HG-08-H07S	HG
				N.P. 114	HG-61
				N.P. 115	HG-61
				N.P. 116	HG-61
				HG21-H04	HG
				FRH1213-H15	RH
				FRH1213-H14	RH
				FRH1213-H12	RH
				FRH1213-H11	RH
				FRH1213-H09	RH
				FRH1213-H07	RH
				FRH1213-H08	RH
				FRH1213-H06	RH
				FRH1213-H03	RH
				FRH1213-H02	RH
				FRH1213-H01	RH
FHP1204-H02	HP				
FHP1204-H03	HP				
FHP1204-H02	HP				
FHP1204-H03	HP				
N.P. 28B	MS-51A				
N.P. 38	MS-51A				
N.P. 41	MS-51A				
N.P. 48A	MS-51A				
N.P. 296B	MS-51A				
N.P. 336B	MS-51A				
N.P. 376B	MS-51A				
N.P. B55	MS-51A				
N.P. C43	MS-51A				
NB-125-H07S	RH				
N.P. 465	VG-03				
N.P. 475-X	VG-03				
N.P. 475-Z	VG-03				
N.P. 30B	RH-75A				
N.P. 30A	RH-76				
N.P. 30B	RH-76				
N.P. 135B	RH-76				
N.P. 112	HG-61				

Table 3.7.9-2 (Continued)

Snubber No.	System snubber installed on location and elev.	Snubber No.	System snubber installed on location and elev.
HP-12-H02S	HP	FRH1213-H18	RH
HP-12-H04S	HP	FRH1209-H03	RH
N.P. 125	RR-59	FRH1209-H04	RH
N.P. 135	RR-59	FRH1209-H05	RH
HG-06-H12S	HG	N.P. 116	RH-20
		FRH1209-H06	RH
N.P. 52	RH-68	FRH1231-H02	RI
N.P. 63	RH-68	FRH1207-H02	RH
N.P. 95	RH-68	FRH1207-H05	RH
RH52-H07	RH	FRI1210-H07	RI
RH52-H08	RH	FRI1209-H05	RI
RH52-H09	RH	FRH1232-H10	RI
RH52-H06	RH	FRH1232-H08	RI
RH52-H05	RH	FRH1232-H06	RI
RH52-H02	RH	N.P. 23	LC-71
RH52-H03	RH	N.P. A34	LC-71
N.P. 133	RH-68	FRH1206-H21	RH
FRH1211-H01	RH	FRH1206-H16	RH
FRH1211-H04	RH	N.P. 303	LC-62
FRH1211-H03	RH	N.P. 18-X	LC-63
FRH42-1037	RH	N.P. 18-Z	LC-63
H-4RH-87	RH	N.P. 23	LC-64
H-2RH-88	RH	N.P. 23-Z	LC-65
N.P. 120	RH-69	N.P. 23-X	LC-65
N.P. 135	RH-69	FRI1207-H01	RI
FRH1214-H-12S	CS	1RI07B-2-1-H08	RI
FRH1230-H07	RH	1RI07B-2-1-H06	RI
FRH1230-H06	RH	1RI07B-2-1-H05	RI
FRH1230-H05	RH	1RI07B-2-1-H16	RI
FRH1230-H03	RH	N.P. 53	RH-21
FRH1230-H02	RH	N.P. 90	RH-21
FRH1230-H01	RH	N.P. 100	RH-21
FRH1206-H03	RH	N.P. 125	RH-21
FRH1233-H05	RI	MS-52-H06S	MS
FRH1233-H04	RI	MS-53-H06S	MS
FRH42-1031	RH	MS-53-H05S	MS
FRH42-1032	RH	MS-50-H02S	MS
FRH42-1033	RH	MS-50-H04S	MS
N.P. 135	RH-16	MS-51-H05S	MS
FRH1231-H09	RI	RT-33-H08S	MS
FRH1231-H06	RI	RT-33-H09S	MS
FRH1231-H04	RI	RT-33-H10S	MS
RH25-H01	RH	N.P. 50	RH-C3
RH25-H02	RH	N.P. 52	RH-C3
RH25-H04	RH	M-1302-22-109	RH
RH25-H03	RH	M-1302-22-110	RH
RH25-H05	RH	M-1302-22-112	RH
RH25-H06	RH	M-1302-22-113	RH
HG-04-H04S	HG	M-1302-23-96	MS
N.P. 117	HG-61	N.P. 60A	RH-C3

Table 3.7.9-2 (Continued)

Snubber No.	System snubber installed on location and elev.			Snubber No.	System snubber installed on location and elev.	
M-1302-22-83	RR					
M-1302-22-33	RR					
M-1302-22-127	RR					
M-1302-24-110	RR					
M-1302-24-111	RR					
M-1302-24-106	RR					
M-1302-24-107	RR					
M-1302-24-103	RR					
M-1302-21-183	MS					
M-1302-21-181	MS					
N.P. 85B	MS-C1					
N.P. 65-Z	MS-C2					
N.P. 65-Y	MS-C2					
M-1302-21-162	MS					
M-1302-21-163	MS					
M-1302-21-74	MS					
M-1302-21-189	MS					
HP75-2009S	HP	Diesel Generator	749	2HG-24-H02	2HG-62	
HP75-2014S	HP	Diesel Generator	735	2HG-16-H01	2HG-62	
HP75-2017S	HP	Diesel Generator	747	RH-27H-H03S	2HG-61	
HP75-2800S	HP	Diesel Generator	743	RH-27H-H06S	2HG-61	
RH83-2006S	RH	Reactor	684	2HG-4H-H10S	2HG-61	
RH83-2007S	RH	Reactor	689	2HG-15H-H03S	2HG-61	
RH83-2009S	RH	Reactor	684	2HG-15-H05S	2HG-61	
RH83-2012S	RH	Reactor	689	2HG-15H-H01S	2HG-61	
RH83-2013S	RH	Reactor	684	2DG-66-H03	2CS-75	
RH83-2016S	RH	Reactor	689	2HG23-H06	2HG-70	
SC02-2967S	SC	Reactor	782	2HG23-H09	2HG-70	
VG01-0015S	VG	Reactor	831	2VQ-10-H02	2VG-03	
VG01-0016S	VG	Reactor	831	2VQ-10-H04	2VG-03	
VG01-2019S	VG	Reactor	830	2VQ-11-H01	2VG-03	
VG01-2021S	VG	Reactor	803	2VQ-11-H03	2VG-03	
VG02-0012S	VG	Reactor	831	2VQ-11-H06	2VG-03	
VG02-2019S	VG	Reactor	803	2VQ-11-H04	2VG-03	
VG02-2021S	VG	Reactor	831			
VG02-2022S	VG	Reactor	794			
VG02-2023S	VG	Reactor	826			
VG02-2024S	VG	Reactor	831			
VG03-2001S	VG	Reactor	794			
VQ04-2019S	VQ	Reactor	738			
VQ04-2020S	VQ	Reactor	733			
VQ05-2023S	VQ	Reactor	762			
VQ05-2024S	VQ	Reactor	762			
VQ05-2025S	VQ	Reactor	787			
VQ05-2026S	VQ	Reactor	732			
VQ05-2031S	VQ	Reactor	733			
VQ04-2800S	VQ	Reactor	733			
VQ06-2002S	VQ	Auxiliary	810			

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-373COMMONWEALTH EDISON COMPANYNOTICE OF ISSUANCE OF AMENDMENT OF FACILITY OPERATING LICENSE

The U.S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 5 to Facility Operating License No. NPF-11, issued to Commonwealth Edison Company, which revised the license for operation of the La Salle County Station, Unit No. 1 (the facility) located in Brookfield Township, La Salle County, Illinois. The Amendment is effective as of August 13, 1982.

The Amendment consists of changes to the Technical Specifications. The changes to the Technical Specifications were as follows: (1) demonstrate the RCIC system is operable at least every 18 months, by verifying, using the test flow path, that the system is capable of delivering a flow greater than or equal to 600 gpm to the reactor vessel when steam is supplied to the turbine at a pressure of 150 ± 15 psig, and (2) allow the use of initial startup test data for determining equivalent flows in the RCIC test line and vessel injection line.

The application for amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this Amendment was not required since the Amendment does not involve a significant hazards consideration.

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The Commission has determined that the issuance of this Amendment will not result in any significant environmental impact and that pursuant to 10 CFR Section 51.5(d)(4) an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this Amendment.

For further details with respect to this action, see (1) the application for amendment dated August 11, 1982, (2) NRC letter dated August 16, 1982, (3) Amendment No. 5 to License NPF-11 dated August 27, 1982, and (4) NRC letter dated August 27, 1982. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C. 20555, and the Public Library of Illinois Valley Community College, Rural Route No. 1, Ogelsby, Illinois 61348. A copy of items (1), (2) and (3) may be obtained upon request addressed to the U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 27th day of August 1982.

FOR THE NUCLEAR REGULATORY COMMISSION

A. Bournia, Acting Chief
Licensing Branch No. 2
Division of Licensing

*SEE ATTACHED PAGE FOR PREVIOUS CONCURRENCES

OFFICE	DD:LB#27/LA	DL:LB#2/PM	DL:LB#2/BC	DELD			
SURNAME	EHylton:pt*	ABournia *	ABournia	CWoodhead*			
DATE	8/20/82	8/20/82	8/27/82	8/23/82			

The Commission has determined that the issuance of this Amendment will not result in any significant environmental impact and that pursuant to 10 CFR Section 51.5(d)(4) an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this Amendment.

For further details with respect to this action, see (1) the application for amendment dated August 11, 1982, (2) NRC letter dated August 16, 1982, *and (4)* ~~and~~ (3) Amendment No. 5 to License NPF-11 dated August , 1982, ¹ All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C. 20555, and the Public Library of Illinois Valley Community College, Rural Route No. 1, Ogelsby, Illinois 61348. A copy of items (1), (2) and (3) may be obtained upon request addressed to the U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this day of August 1982.

FOR THE NUCLEAR REGULATORY COMMISSION

A. Schwencer, Chief
Licensing Branch No. 2
Division of Licensing

*NRC
distributed
August —
1982.*

OFFICE	DL:LB#2/LA	DL:LB#2/PM	DL:LB#2/BC	OELD		
SURNAME	EGton:kw	ABournia	ASchwencer			
DATE	8/17/82	8/18/82	8/ /82	8/23/82		