

August 27, 1982

Docket No. 50-249

Mr. L. DeGeorge
Director of Nuclear Licensing
Commonwealth Edison Company
P. O. Box 767
Chicago, Illinois 60690

Dear Mr. DeGeorge:

Subject: Dresden 3 Cycle 8 Reload

On April 29, 1982 we issued Amendment No. 63 to Facility Operating License No. DPR-25 for Dresden Station, Unit 3. The amendment authorized changes to the facility License and Technical Specifications (TS) to support Cycle 8 operation with reload fuel supplied by, and the associated analyses performed by the Exxon Nuclear Company (ENC).

Section 1.5 of our Safety Evaluation supporting that amendment identified certain conditions regarding our review of your fuel design analyses which could require you to submit additional information. Specifically, Section 1.5 stated: "... the licensee will be required to submit additional calculations for NRC approval of rod pressure and fuel centerline temperature with acceptable methods prior to reaching 10,000 MWD/MTU unless the use of RODEX2 is approved without modifications. The appropriate Technical Specifications have been modified to reflect this requirement." Operation for the first 10,000 MWD/MTU was approved based on your analyses using the approved GAPEXX code and the advanced status of our review of the ENC topical report on fuel design entitled "Generic Mechanical Design for Exxon Nuclear Jet Pump BWR Reload Fuel", XN-NF-81-21 (P), Revision 1.

We have since completed our review of the topical report and have issued our Safety Evaluation. As a result of that review, we have identified three areas, in addition to rod pressure and fuel centerline temperature, for which we would require confirmatory calculations for operation beyond 10,000 MWD/MTU. These areas are design strain, external corrosion, and transient strain for PCI. In the event a submittal is required, i.e. RODEX2 is not approved without modifications, these areas should also be addressed. As noted

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above, the Technical Specifications have already been appropriately conditioned to require the submittal, if necessary; therefore, no TS changes are required as a result of this review.

Please contact your NRC Project Manager if you have any questions pertaining to this matter.

Sincerely,

ORIGINAL SIGNED BY

Domenic B. Vassallo, Chief
Operating Reactors Branch #2
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cc: See next page

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