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Docket No. 50-316

Mr. John Dolan, Vice President
Indiana and Michigan Electric Company
Post Office Box 18
Bowling Green Station
New York, New York 10004

Dear Mr. Dolan:

The Commission has issued the enclosed Amendment No. 19 to Facility Operating License No. DPR-74 for the Donald C. Cook Nuclear Plant, Unit No. 2. The amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated November 2, 1979 as supplemented by letters dated December 11, 1979 and March 18, 1980. The amendments also delete license conditions which have been satisfied.

These amendments revise the maximum T_{avg} at Rated Thermal Power used for the overtemperature and overpower delta T trips and departing from nucleate boiling (DNB) limits. The deletion of the license conditions recognize the modifications for leak testing certain containment isolation valves and installation of qualified 600 volt containment power penetration circuits.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

Original signed by:
S. A. Varga

Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Enclosures:
1. Amendment No. 19 to DPR-74
2. Safety Evaluation
3. Notice of Issuance

cc: w/enclosures
See next page

Legal review of amendment & FR notice only

STSG JWetmore 05/15/80

OFFICE		DL:ORB1	DL:ORB1	DL:ORB1	DL:AD:OR	OELD
SURNAME		DWigginton:jb	CSParrish	SAVarga	TNovak	S. Treby
DATE		05/ /80	5/5/80	5/10/80	5/10/80	5/19/80

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

May 13, 1980

Docket No. 50-316

Mr. John Dolan, Vice President
Indiana and Michigan Electric Company
Post Office Box 18
Bowling Green Station
New York, New York 10004

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These amendments revise the maximum T_{avg} at Rated Thermal Power used for the overtemperature and overpower delta T trips and departing from nucleate boiling (DNB) limits. The deletion of the license conditions recognize the modifications for leak testing certain containment isolation valves and installation of qualified 600 volt containment power penetration circuits.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Enclosures:
1. Amendment No. 19 to DPR-74
2. Safety Evaluation
3. Notice of Issuance

cc: w/enclosures
See next page

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cc: Mr. Robert W. Jurgensen
Chief Nuclear Engineer
American Electric Power
Service Corporation
2 Broadway
New York, New York 10004

Gerald Charnoff, Esquire
Shaw, Pittman, Potts and Trowbridge
1800 M Street, N.W.
Washington, D. C. 20036

Citizens for a Better Environment
59 East Van Buren Street
Chicago, Illinois 60605

Maude Preston Palenske Memorial
Library
500 Market Street
St. Joseph, Michigan 49085

Mr. D. Shaller, Plant Manager
Donald C. Cook Nuclear Plant
P. O. Box 458
Bridgman, Michigan 49106

Mr. Robert Masse
Resident Inspector
Donald C. Cook Nuclear Plant
U. S. Nuclear Regulatory Commission
P. O. Box 458
Bridgman, Michigan 29160

Mr. Wade Schuler, Supervisor
Lake Township
Baroda, Michigan 49101

Mr. William R. Rustem (2)
Office of the Governor
Room 1 - Capitol Building
Lansing, Michigan 48913

Honorable James Bemenek, Mayor
City of Bridgman, Michigan 49106

Director, Technical Assessment Division
Office of Radiation Programs (AW-459)
U. S. Environmental Protection Agency
Crystal Mall #2
Arlington, Virginia 20460

U. S. Environmental Protection Agency
Federal Activities Branch
Region V Office
ATTN: EIS COORDINATOR
230 South Dearborn Street
Chicago, Illinois 60604

Maurice S. Reizen, M.D.
Director
Department of Public Health
P. O. Box 30035
Lansing, Michigan 48909

William J. Scanlon, Esquire
2034 Pauline Boulevard
Ann Arbor, Michigan 48103

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

INDIANA AND MICHIGAN ELECTRIC COMPANY

DOCKET NO. 50-316

DONALD C. COOK NUCLEAR PLANT UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 19
License No. DPR-74

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Indiana and Michigan Electric Company (the licensee) dated November 2, 1979 as supplemented on December 11, 1979 and March 18, 1980 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-74 is hereby amended to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 19, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. In addition, license condition 2.C.(3)(i) "Leak Rate Testing of Containment Isolation Valves" and 2.C.(3)(m) "600 Volt Containment Power Penetration" are deleted.

4. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: May 13, 1980

ATTACHMENT TO LICENSE AMENDMENTS

AMENDMENT NO. 19 TO FACILITY OPERATING LICENSE NO. DPR-74

DOCKET NO. 50-316

Revise Appendix A as follows:

Remove Pages	Insert Pages
2-7	2-7
2-8	2-8
2-9	2-9
3/4 2-16	3/4 2-16

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TABLE 2.2-1 (Continued)

REACTOR TRIP SYSTEM INSTRUMENTATION TRIP SETPOINTS

NOTATION

NOTE 1: Overtemperature $\Delta T \leq \Delta T_o \left[K_1 - K_2\left(\frac{1+t_1 S}{1+\tau_2 S}\right)(T-T') + K_3(P-P') - f_1(\Delta I)\right]$

where: ΔT_o = Indicated ΔT at RATED THERMAL POWER

T = Average temperature, °F

T' = Indicated T_{avg} at RATED THERMAL POWER $\leq$ 573.8°F

P = Pressurizer pressure, psig

P' = 2235 psig (indicated RCS nominal operating pressure)

$\frac{1+\tau_1 S}{1+\tau_2 S}$ = The function generated by the lead-lag controller for T_{avg} dynamic compensation

τ_1 & τ_2 = Time constants utilized in the lead-lag controller for T_{avg} τ_1 = 33 secs, τ_2 = 4 secs.

S = Laplace transform operator

TABLE 2.2-1 (Continued)

REACTOR TRIP SYSTEM INSTRUMENTATION TRIP SETPOINTS

NOTATION (Continued)

Operation with 4 Loops	Operation with 3 Loops
$K_1 = 1.334$	$K_1 = 1.116$
$K_2 = 0.01607$	$K_2 = 0.01607$
$K_3 = 0.000744$	$K_3 = 0.000744$

and f_1 (ΔI) is a function of the indicated difference between top and bottom detectors of the power-range nuclear ion chambers; with gains to be selected based on measured instrument response during plant startup tests such that:

(i) for $q_t - q_b$ between - 40 percent and + 3 percent, f_1 (ΔI) = 0 (where q_t and q_b are percent RATED THERMAL POWER in the top and bottom halves of the core respectively, and $q_t + q_b$ is total THERMAL POWER in percent of RATED THERMAL POWER).

(ii) for each percent that the magnitude of ($q_t - q_b$) exceeds - 40 percent, the ΔT trip setpoint shall be automatically reduced by 1.8 percent of its value at RATED THERMAL POWER.

(iii) for each percent that the magnitude of ($q_t - q_b$) exceeds + 3 percent, the ΔT trip setpoint shall be automatically reduced by 2.2 percent of its value at RATED THERMAL POWER.

TABLE 2.2-1 (Continued)

REACTOR TRIP SYSTEM INSTRUMENTATION TRIP SETPOINTS

NOTATION (Continued)

Note 2: Overpower $\Delta T \leq \Delta T_o[K_4 - K_5 \left(\frac{\tau_3 S}{1+\tau_3 S}\right) T - K_6 (T - T") - f_2(\Delta I)]$

where:

ΔT_o	=	Indicated ΔT at rated power
T	=	Average temperature, °F
$T"$	=	Indicated T_{avg} at RATED THERMAL POWER $\leq$ 573.8°F
K_4	=	1.078
K_5	=	0.02/°F for increasing average temperature and 0 for decreasing average temperature
K_6	=	0.00197 for $T > T"$; $K_6 = 0$ for $T \leq T"$
$\frac{\tau_3 S}{1+\tau_3 S}$	=	The function generated by the rate lag controller for T_{avg} dynamic compensation
τ_3	=	Time constant utilized in the rate lag controller for T_{avg} τ_3 = 10 secs.
S	=	Laplace transform operator
$f_2(\Delta I)$	=	0 for all ΔI

Note 3: The channel's maximum trip point shall not exceed its computed trip point by more than 4 percent.

DNB PARAMETERS

LIMITING CONDITION FOR OPERATION

3.2.5 The following DNB related parameters shall be maintained within the limits shown on Table 3.2-1:

a. Reactor Coolant System T_{avg}.

b. Pressurizer Pressure.

APPLICABILITY: MODE 1

ACTION:

With any of the above parameters exceeding its limit, restore the parameter to within its limit within 2 hours or reduce THERMAL POWER to less than 5% of RATED THERMAL POWER within the next 4 hours.

SURVEILLANCE REQUIREMENTS

4.2.5 Each of the parameters of Table 3.2-1 shall be verified to be within their limits at least once per 12 hours.

TABLE 3.2-1

DNB PARAMETERS

PARAMETER	LIMITS	
	4 Loops In Operation	3 Loops In Operation
Reactor Coolant System T_{avg}	$\leq$ 578 °F	$\leq$ 570 °F
Pressurizer Pressure	$\geq$ 2220 psia*	$\geq$ 2220 psia*

*Limit not applicbale during either a THERMAL POWER ramp increase in excess of 5% RATED THERMAL POWER per minute or a THERMAL POWER step increase in excess of 10% RATED THERMAL POWER.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 19 TO FACILITY OPERATING LICENSE NO. DPR-74

INDIANA AND MICHIGAN ELECTRIC COMPANY

DONALD C. COOK NUCLEAR PLANT UNIT NO. 2

DOCKET NO. 50-316

A. Maximum T_{avg} at Rated Thermal Power

1.0 Introduction

In Reference 1, Indiana and Michigan Electric Company requested a Technical Specification change for D. C. Cook Unit 2 which involves changing the maximum T_{avg} at Rated Thermal Power used for the overtemperature and overpower ΔT trips and DNB limits. Supporting analysis for this change was submitted as Attachment A to Reference 2.

Revised pages and additional information were submitted in Reference 3.

2.0 Discussion

The thermal overpower ΔT (OPΔT) and overtemperature ΔT (OT ΔT) reactor trips provide protection against departure from nucleate boiling (DNB) and fuel centerline melting (excessive kw/ft) respectively during postulated transients. The methodology for derivation of the limiting safety system settings is presented in Reference 4.

2.1 Overtemperature Trip

OTΔT trip is designed to ensure operation within the DNB design basis and the hot leg boiling limit. The setpoint is a function of a vessel ΔT, vessel average temperature primary system pressure and axial flux difference. The static equation is:

$$\Delta T_{sp} = k_1 - k_2 (T_{avg} - T') + k_3 (P - P') - F(\Delta I)$$

where:

ΔT_{sp} = OTΔT setpoint (% of full power ΔT)

k_1 = a preset, manually adjustable bias

k_2 = a constant based on the effect of temperature on design limits

k_3 = a constant based on the effects of pressure (% full power/psi)

T_{avg} = Average reactor coolant temperature (^oF)

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T' = Nominal T_{avg} at full power (^oF)

P = Pressurizer Pressure (psig)

P' = Nominal reactor coolant system pressure (psig)

$f(\Delta I)$ = A function of the neutron flux difference between upper and lower long ion chambers

A dynamic term is also included to compensate for inherent instrument delays and piping lags between the core and the loop temperature sensors. For the OTΔT setpoint a lead/lag function $\left(\frac{1 + T_1S}{1 + T_2S}\right)$ is applied to the $k_2(T_{avg} - T')$ term.

2.2 Overpower Trip

The OPΔT trip is designed to prevent fuel centerline melting with an overpower setpoint of 118% of the nominal full power ΔT. A compensating term is included since thermal power is not precisely proportional to ΔT, as well as a flux difference term.

The static equation is:

$$\Delta T_{sp} = K_4 - K_6 (T_{avg} - T') - F(\Delta I)$$

where:

ΔT_{sp} = overpower ΔT setpoint (% of full power ΔT)

K_4 = a present, manually adjustable bias (% of full-power ΔT)

K_6 = a constant that accounts for the effects of coolant density and heat capacity on the relationship between ΔT and thermal power (% of full power ΔT/^oF)

T' = indicated average reactor coolant temperature at full power (^oF)

T_{avg} = average reactor-coolant temperature (^oF)

$F(\Delta I)$ = a function of the neutron flux difference between upper and lower long ion chamber section (% full power ΔT). Increases in ΔT beyond a predefined deadband result in a decrease in the trip setpoint.

The dynamic term for the OPΔT setpoint is $-K_5 \frac{T_3S}{1+T_3S} T_{avg}$ where:

K_5 = a constance that compensates for piping and thermal time delay

$T_3S/1+T_3S$ = rate/lag function

T_{avg} = average reactor coolant temperature

2.3 DNB Limits

The DNB parameters limits in the T.S. assure that each parameter is maintained within the normal steady-state envelope of operation assumed in the safety analysis.

3.0 Proposed Modifications

The licensee has proposed the following changes:

a. For OTΔT setpoint (pps. 2-7 and 2-8, Table 2.2-1 of T.S.)

1. T' increased from 572.2°F to 573.8°F
2. k_1 (4 loop operation) decreased from 1.36 to 1.334
3. k_1 (3 loop operation) decreased from 1.142 to 1.116

b. For OPΔT setpoint (p. 2-9, Table 2.2-1 of T.S.)

1. T' increased from 572.2°F to 573.8°F

c. For DNB Parameters (Table 3.2-1 of T.S.)

1. 4 loop operation T_{avg} limit increased from 576.2°F to 578°F
2. 3 loop operation T_{avg} limit increased from 569.8°F to 570°F

4.0 Evaluation

4.1 Analysis

Revised analyses were provided for transients potentially affected by the proposed changes. These events include:

Rod Withdrawal
Loss of Flow - 4 pump coastdown
Loss of Flow - 1 pump coastdown, 4 loops in operation
Startup of an Inactive Loop
Loss of Load
Loss of Feedwater
Excessive Heat Removal due to Feedwater Systems Malfunctions
Excessive Load Increase
Steam Line Rupture

The input parameters, codes and evaluation models used were the same as those used in the FSAR (Reference 5) except as discussed below.

The OTΔT and OPΔT setpoints are as given in Section 3.0.

The maximum Tavg was increased from 576.2°F (572.2°F (nom) + 4°F) to 578°F (573.8°F (nom) + 4°F, rounded). The three loop maximum Tavg was 570°F.

Nominal Tavg (used in analysis of most events with the Westinghouse Improved Thermal Design Basis (Reference 7))was 573.8°F.

The loss of load event was analyzed with the Improved Basis for DNB evaluations and also with the maximum temperature and pressure to ensure that the peak pressure remains below the limit.

The results show that only small changes occur for most events. The minimum DNBR occurs for the rod withdrawal event, with a MDNBR of 2.0. The design limit for Cook 2 with the WRB-1 correlation (Reference 6) is 1.8.

The steam line break analysis was also reviewed to ensure that the return to power following the scram does not violate any core thermal limits.

The effect of a temperature increase of 1.8°F (578-576.2°F) on steady-state DNBR is a reduction of .04, so that the steady-state DNBR at the maximum allowable temperature remains well above the limit.

4.2 Technical Specification Changes

For the static OTΔT equation, the proposed changes to T' and K_1 balance each other so that the ΔT_{sp} is unchanged. The dynamic effect is a higher ΔT_{sp} for any particular value of Tavg. However, since the event starts at a higher average temperature, the ΔT rise before reaching the trip is less than previously and, in fact, the analysis shows that a reactor trip occurs slightly earlier into the transient than it did with the existing setpoints.

The OPΔT setpoint change allows operation at the 118% overpower limit for Tavg up to 573.8°F, with the setpoint decreasing after that point. This is consistent with the safety analysis for full power operation at a nominal core average temperature of 573.8°F.

The DNB limits are simply rounded off values based on nominal temperatures plus 4°F for indication error. Periodically, plant conditions as determined by instrument readout are compared to these limits, and adjusted as required so that steady-state operation is maintained within the bounds of the analysis. The effect of the roundoff (.2°F) is negligible.

5.0 References

1. Letter from J. E. Dolan (AEP) to H. R. Denton (NRC), Serial Number AEP: NRC: 00297, November 2, 1979.

2. Letter from J. E. Dolan (AEP) to H. R. Denton (NRC), Serial Number AEP: NRC: 00297A, December 11, 1979.

3. Letter from J. E. Dolan (AEP) to H. R. Denton (NRC), Serial Number AEP: NRC: 00297D, March 18, 1980.

4. WCAP-8745 "Design Bases for the Thermal Overpower ΔT and Thermal Overtemperature ΔT Trip Functions," March 1977.

5. Indiana and Michigan Power Company "Donald C. Cook Nuclear Plant Final Safety Analysis Report," with Amendments 1-79.

6. WCAP-8762 "WRB-1 Critical Heat Flux Correlation "

7. WCAP-8567 "Improved Thermal Design Procedure"

B. Deletion of License Conditions

In Supplement No. 7 to the D. C. Cook Nuclear Plant, Unit 2 Safety Evaluation Report dated December 1977, the Staff's evaluation concluded that two license conditions were required to support the Staff's findings of adequate design and operation. The license conditions were as follows:

2.C.(3)(i) Leak Rate Testing of Containment Isolation Valves

Indiana and Michigan Power Company shall install prior to startup, following the first regularly scheduled refueling outage, test connections to allow Type C leak testing of containment isolation valves. Indiana and Michigan Power Company shall modify the containment isolation valves in the component cooling water system which are identified by and associated with the following containment penetration numbers: CPN 25(1), 25(2), 25(3), 25(4), 72(1), 72(2), 72(3) and 72(4). Indiana and Michigan Power Company shall modify to allow pneumatic leak rate testing the isolation valves identified by and associated with the following containment penetration numbers: CPN 38, 39 and 56.

2.C.(3)(m) 600 Volt Containment Power Penetrations

Indiana and Michigan Power Company shall modify the 600 volt containment electrical power penetration circuits to meet the requirements of Regulatory Guide 1.63 prior to startup following the first regularly scheduled refueling outage. This modification consists of the installation of redundant circuit breakers in the 600 volt containment electrical power penetration circuits to protect the penetration seals by a trip of the 600 volt switchgear breakers in the event of a failure of the molded case circuit breakers.

Further discussion and evaluation of these requirements is not required beyond that which is included in the Safety Evaluation Report, Supplement No. 7.

In the inspection report number 50-315/80-02; 50-316/80-02 dated February 11, 1980, the Office of Inspection and Enforcement Resident Inspector reported that the license conditions had been met as evidenced by his review and inspection of the required modification and installations. Therefore it is recommended that the license conditions 2.C.(3)(i) and (m) be deleted.

C. Environmental Consideration

We have determined that the amendments do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendments involve an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of these amendments.

D. Conclusion

We have concluded that the proposed changes to T_{avg} and to the Technical Specifications are acceptable and the license conditions 2.C.(3)(i) and (m) should be deleted. We have also concluded, based on the considerations discussed above, that: (1) because the amendments do not involve a significant increase in the probability or consequences of accidents previously considered and do not involve a significant decrease in a safety margin, the amendments do not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Date: May 13, 1980

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-316

INDIANA AND MICHIGAN ELECTRIC COMPANY

NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 19 to Facility Operating License No. DPR-74 issued to Indian and Michigan Electric Company (the licensee), which revised Technical Specifications for operation of Donald C. Cook Nuclear Plant, Unit No. 2 (the facility) located in Berrien County, Michigan. The amendment is effective as of the date of issuance.

The amendment revises the maximum T_{avg} at Rated Thermal Power used for the overtemperature and overpower delta T trips and departure from nucleate boiling (DNB) limits. The amendment also deletes license conditions which required modifications for leak testing certain containment isolation valves and installation of qualified 600 volt containment power penetration circuits. These modifications and installations have been completed.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments. Prior public notice of the amendment was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of the amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of the amendment.

For further details with respect to this action, see (1) the application for amendment dated November 2, 1979 as supplemented December 11, 1979 and March 18, 1980, (2) Amendment No. 19 to License No. DPR-74, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D. C. and at the Maude Reston Palenske Memorial Library, 500 Market Street, St. Joseph, Michigan 49085. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 13th day of May, 1980.

FOR THE NUCLEAR REGULATORY COMMISSION

Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing