February 10, 1988

Docket No. 50-316

Mr. Milton P. Alexich, Vice President
Indiana Michigan Power Company
c/o American Electric Power Service Corporation
1 Riverside Plaza
Columbus, Ohio 43216

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Dear Mr. Alexich:

SUBJECT: NOTICE OF CONSIDERATION OF ISSUANCE OF AMENDMENT (TAC NO. 67039)

Re: D. C. Cook Nuclear Power Plant, Unit 2

The Commission has forwarded the enclosed "Notice of Consideration of Issuance of Amendment to Facility Operating License and Proposed No Significant Hazards Consideration Determination" to the Office of the Federal Register for publication. This notice relates to three of the changes in your application dated January 11, 1988, which requested certain tests normally designated as 18-month surveillances to be delayed until the end of the next refueling outage currently scheduled to begin during the second quarter of 1988. The other changes requested by your application of January 11, 1988, will be the subject of a separate notice.

Sincerely,

Original signed by

John F. Stang, Project Manager
Project Directorate III-1
Division of Reactor Projects - III, IV, V
& Special Projects

Enclosure:
As stated

cc w/enclosure:
See next page

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Indiana Michigan Power Company

Donald C. Cook Nuclear Plant

cc:
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UNITED STATES NUCLEAR REGULATORY COMMISSION

INDIANA MICHIGAN POWER COMPANY

DOCKET NO. 50-316

NOTICE OF CONSIDERATION OF ISSUANCE OF AMENDMENT TO FACILITY OPERATING LICENSE AND PROPOSED NO SIGNIFICANT HAZARDS CONSIDERATION DETERMINATION

The U. S. Nuclear Regulatory Commission (the Commission) is considering issuance of an amendment to Facility Operating License No. DPR-74 issued to the Indiana Michigan Power Company (the licensee) for operation of the D. C. Cook Nuclear Plant, Unit 2, located in Berrien County, Michigan.

The proposed amendment is the second of two submittals that request surveillance interval extensions for Unit 2, Cycle 6. The changes requested in this proposed amendment supplement the extension requests submitted in the licensee's amendment request dated October 28, 1987. Those changes were granted by the Commission's staff in Amendment No. 97 to Facility Operating License No. DPR-74. The proposed amendment would again change the Technical Specifications to allow certain tests normally designated as 18-month surveillances to be delayed until the end of the next refueling outage currently scheduled to begin during the second quarter of 1988. These tests include residual heat removal (RHR) auto-closure interlock test, steam generator snubber tests, control rod indication system functional testing, calibration of resistance temperature detectors (RTDs), testing of engineered safety feature (ESF) manual actuation switches and channel calibrations and interlock testing involving the pressurizer pressure instrumentation. There are also two proposed editorial changes. These revisions to the Technical

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Specifications would be made in response to the licensee's application for amendment dated January 11, 1988.

The Commission has determined that failure to act in a timely manner to grant extensions for the RHR auto-closure interlock testing, steam generator snubber testing and Rod Position Indications Systems Functional Testing would result in shutdown of Unit 2. Therefore, the Commission has insufficient time to issue its usual 30-day notice of the proposed action for public comment for these three requested extensions. The remaining three extension requests will be the subject of a separate notice.

Before issuance of the proposed license amendment, the Commission will have made findings required by the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations.

The Commission's standards for determining whether a significant hazards consideration exists are stated in 10 CFR 50.92. A proposed amendment to an operating license for a facility involves no significant hazards consideration if operation of the facility in accordance with the proposed amendment would not (1) involve a significant increase in the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in a margin of safety. Therefore, in accordance with 10 CFR 50.91 and 50.92, the licensee has provided the following analysis:

(1) RHR Auto-closure Interlock Test

Criterion 1

The surveillance test history of the auto-closure interlock has shown that the system is highly reliable, and gives us no reason to believe the equipment would be inoperable during an extension period. The wide-range pressure transmitters, which provide input into the auto-closure interlock, will have a current calibration. Additionally, we note that when the RHR

system is not in service, power is removed from the suction valve operators, thus preventing inadvertent valve opening and eliminating the need for the auto-closure interlock. For these reasons, we believe the extension we are requesting will not result in a significant increase in the probability or consequences of a previously evaluated accident, nor will it result in a significant reduction in a margin of safety.

Criterion 2

This extension will not result in a change in plant configuration or operation. Therefore, the extension should not create the possibility of a new or different kind of accident from any previously evaluated or analyzed.

Criterion 3

See Criterion 1, above.

(2) Steam Generator Snubbers

Criterion 1

Thirteen steam generator snubbers have been functionally tested at the Cook Nuclear Plant since 1983 with only one failure, the cause of which was not generic. Visual inspections have been performed on snubbers since 1975, revealing no problems or potential problems. Based on this surveillance history, we have no reason to believe the steam generator snubbers will be inoperable during the extension period. Thus, it is believed that this change will not result in a significant increase in the probability or consequences of a previously evaluated accident, nor will it significantly reduce a margin of safety.

Criterion 2

Delaying the snubber functional test will not result in a change in plant design or operation. Therefore, we believe that the change will not create the possibility of a new or different kind of accident from any accident previously analyzed or evaluated.

Criterion 3

See Criterion 1, above.

(3) Rod Position Indication System Functional Testing

Criterion 1

T/S-required comparison of the RPI channels to the demand position indication system would be expected to indicate significant degradation in the RPI channels. In addition, other surveillance, such as the determination of the quadrant power tilt ratio and incore flux mapping, provide a comparison of core performance to design and would be expected

to indicate significant deviations of the control rods from their indicated position. Also, the RPI channel surveillance history is good and provides no reason to believe the channels would be inoperable during the extension period. For all these reasons, we believe the change will not involve a significant increase in the probability or consequences of a previously analyzed accident and that it will not involve a significant reduction in a margin of safety.

Criterion 2

The proposed change will not result in a change in plant configuration or operation. Thus, the change should not create the possibility of a new or different kind of accident from any accident previously analyzed or evaluated.

Criterion 3

See Criterion 1, above.

The Commission's staff has reviewed the licensee's no significant hazards consideration determinations and concurs with the licensee's analysis. Therefore, the staff proposes to determine that the portion of the application for amendment requesting time extensions for testing the RHR auto-closure interlock, the steam generator snubbers and the Rod Position Indication System involves no significant hazards consideration.

If the proposed determination becomes final, an opportunity for a hearing will be published in the FEDERAL REGISTER at a later date and any hearing request will not delay the effective date of the amendment.

If the Commission decides in its final determination that the amendment does involve a significant hazards consideration, a notice of opportunity for a prior hearing will be published in the FEDERAL REGISTER and, if a hearing is granted, it will be held before any amendment is issued.

The Commission is seeking public comments on this proposed determination of no significant hazards consideration. Comments on the proposed determination may be telephoned to Martin Virgilio, Director of Project Directorate III-1, by collect call to 1-301-492-1315 or submitted in writing to

the Rules and Procedures Branch, Division of Rules and Records, Office of Administration, U. S. Nuclear Regulatory Commission, Washington, D.C. 20555, and should cite the publication date and page number of this FEDERAL REGISTER notice. All comments received by February 25, 1988, will be considered in reaching a final determination. A copy of the application may be examined at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C., and at the Maude Preston Palenski Memorial Library, 500 Market Street, St. Joseph, Michigan 49085.

Dated at Rockville, Maryland, this 10th day of February, 1988.

FOR THE NUCLEAR REGULATORY COMMISSION

John J. Stefano, Acting Director
Project Directorate III-1
Division of Reactor Projects - III, IV, V
& Special Projects