

December 23, 1981

Docket No. 50-249



Mr. L. DelGeorge
Director of Nuclear Licensing
Commonwealth Edison Company
P.O. Box 767
Chicago, Illinois 60690

Dear Mr. DelGeorge:

The Commission has issued the enclosed Amendment No. 57 to Facility Operating License No. DPR-25 for Dresden Station Unit No. 3. This amendment consists of changes to the Technical Specifications in response to your application dated November 23, 1981. This amendment was authorized by telephone on November 23, 1981 and was confirmed by our letter dated November 24, 1981.

The amendment authorizes deletion of the requirement that CECO demonstrate the operability of the automatic pressure relief valves daily during the period beginning November 23, 1981 through November 29, 1981 while the High Pressure Coolant Injection System is inoperable.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

Thomas A. Ippolito, Chief
Operating Reactors Branch #2
Division of Licensing

Enclosures:

- 1. Amendment No. 57 to DPR-25
- 2. Safety Evaluation
- 3. Notice of Issuance

cc: w/enclosures
See next page

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Mr. L. DelGeorge

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-249

DRESDEN STATION UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 57
License No. DPR-25

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Commonwealth Edison Company (the licensee) dated November 23, 1981 complies with standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-25 is hereby amended to read as follows:

3.B Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 57, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of November 23, 1981.

FOR THE NUCLEAR REGULATORY COMMISSION



Thomas A. Ippolito, Chief
Operating Reactors Branch #2
Division of Licensing

Attachment:
Changes to the
Technical Specifications

Date of Issuance: December 23, 1981

ATTACHMENT TO LICENSE AMENDMENT NO. 57

FACILITY OPERATING LICENSE NO. DPR-25

DOCKET NO. 50-249

Replace page 77 with the enclosed page 77.

3.5 LIMITING CONDITION FOR OPERATION

containment cooling subsystem, both core spray subsystems and both diesel generators required for operation of such components if no external source of power were available, shall be operable.

4. If the requirements of 3.5.B cannot be met an orderly shutdown shall be initiated and the reactor shall be in a Cold Shutdown condition within 24 hours.

C. HPCI Subsystem

1. Except as specified in 3.5.C.2 below, the HPCI subsystem shall be operable whenever the reactor pressure is greater than 90 psig and irradiated fuel is in the reactor vessel.
2. From and after the date that the HPCI subsystem is made or found to be inoperable for any reason, reactor operation is permissible only during the succeeding seven days unless such subsystem is sooner made operable, provided that during such seven days all active components of the Automatic Pressure Relief Subsystem, the core spray subsystems, LPCI subsystem, and isolation cooling system are operable.
3. If the requirements of 3.5.C cannot be met an orderly shutdown shall be initiated and the reactor pressure shall be reduced to 90 psig within 24 hours.

4.5 SURVEILLANCE REQUIREMENT

C. Surveillance of HPCI Subsystem shall be performed as follows:

1. HPCI Subsystem Testing shall be as specified in 4.5.A.1.a, b, c, d, and f, except that the HPCI pump shall deliver at least 5000 gpm against a system head corresponding to a reactor vessel pressure of 1150 psig to 150 psig.
2. When it is determined that HPCI subsystem is inoperable, the LPCI subsystem, both core spray subsystems, the automatic pressure relief subsystem, and the motor operated isolation valves and shell side make-up system for the isolation condenser system shall be demonstrated to be operable immediately. The automatic pressure relief * and motor operated isolation valves and shell side make-up system of the isolation condenser shall be demonstrated to be operable daily thereafter.

*Note: From November 23, 1981 through November 29, 1981, with the HPCI inoperable, CECO is not required to demonstrate the operability of the automatic pressure relief subsystem daily. During this period, two feedwater pumps shall be operating at power levels above 300 MWe and one feedwater pump operating as normally required and one additional feedwater pump shall be operable at power levels less than 300 MWe.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 57 TO FACILITY OPERATING LICENSE NO. DPR-25

COMMONWEALTH EDISON COMPANY

DRESDEN STATION, UNIT NO. 3

DOCKET NO. 50-249

1.0 Introduction

By letter dated November 23, 1981 Commonwealth Edison Company (CECo) requested a technical specification change deleting that portion of Specification 4.5.C.2 that requires that the automatic pressure relief subsystem be demonstrated operable daily upon declaring that the High Pressure Coolant Injection System inoperable.

2.0 Background

Technical Specification 4.5.C.2 requires that:

"When it is determined that HPCI subsystem is inoperable, the LPCI subsystem, both core spray subsystems, the automatic pressure relief subsystem, and the motor operated isolation valves and shell side make-up system for the isolation condenser system shall be demonstrated to be operable immediately. The automatic pressure relief and motor operated isolation valves and shell side make-up system of the isolation condenser shall be demonstrated to be operable daily thereafter."

On November 22, 1981 as a result of an apparent faulty valve position indication, CECo personnel determined that the inboard High Pressure Coolant Injection (HPCI) steam supply isolation valve would not properly function. Upon discovery of the nonfunctioning valve, CECo personnel closed the outboard isolation valve, declared the HPCI system inoperable, and complied with demonstrating the operability of other systems required by Technical Specification 4.5.C.2. This specification requires, in part, testing the automatic pressure relief subsystem immediately, and daily thereafter. The testing was completed successfully on November 22, 1981 and the operability of the automatic pressure relief subsystem was demonstrated by opening each automatic pressure relief valve, discharging steam to the torus, and reseating the valve.

Because there have been instances in the past in which relief valves have failed to reseat after opening and because the testing of these valves causes a significant thermal and mechanical shock to the reactor vessel and the suppression chamber (torus), Commonwealth Edison has proposed to delete that portion of the surveillance test that requires the daily testing of these automatic pressure relief valves on subsequent days that the HPCI system continues to be inoperable.

3.0 Evaluation

We have reviewed CECO's request and have determined that Commonwealth Edison has adequately demonstrated the operability of the automatic pressure relief subsystem on November 22, 1981 and that there is a low probability that the system would become inoperable during the 7 remaining days that the plant is permitted to remain in operation with the HPCI declared inoperable. We have concluded that the increased probability of having one of the pressure relief valves stick open during the daily testing outweighs the benefits gained by the daily testing and that the safety of the facility will not be adversely affected by deleting the requirement to test these valves daily after the November 22, 1981 demonstration of their operability.

During the same period of permitted operation (November 23 through November 29, 1981) with the HPCI out of service, CECO will be required to have two feedwater pumps operating at power levels in excess of 300 MWe and one feedwater pump operating as required below 300 MWe and one additional pump operable. The operation of these feed pumps provided added assurance that a high pressure make up capability is available for transients with offsite power available which do not cause isolation.

Similar relief was provided to CECO under similar conditions for continued Unit 2 operation in Amendment No. 65 to Provisional Operating License No. DPR-19 dated September 15, 1981.

4.0 Environmental Considerations

We have determined that this amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that this amendment involves an action which is insignificant from the standpoint of environmental impact and pursuant to 10 CFR §51.5(d)(4) that an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

5.0 Conclusion

We have concluded, based on the considerations discussed above, that (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: December 23, 1981

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-249COMMONWEALTH EDISON COMPANYNOTICE OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSE

The U.S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 57 to Facility Operating License No. DPR-25, issued to Commonwealth Edison Company, which revised the Technical Specifications for operation of the Dresden Nuclear Power Station, Unit No. 3 located in Grundy County, Illinois. The amendment became effective November 23, 1981.

The amendment authorizes deletion of the requirement to demonstrate the operability of the automatic pressure relief valves daily during the period beginning November 23, 1981 through November 29, 1981 while the High Pressure Coolant Injection System is inoperable.

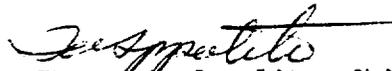
The application for amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of the amendment was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR Section 51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of the amendment.

For further details with respect to this action, see (1) the application for amendment dated November 23, 1981, (2) the Commission's letter to the licensee dated November 24, 1981, (3) Amendment No. 57 to Facility Operating License No. DPR-25, and (4) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C., and at the Morris Public Library, 604 Liberty Street, Morris, Illinois. A copy of items (2), (3) and (4) may be obtained upon request addressed to the U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 23rd day of December, 1981.

FOR THE NUCLEAR REGULATORY COMMISSION


Thomas A. Ippolito, Chief
Operating Reactors Branch #2
Division of Licensing