

DISTRIBUTION

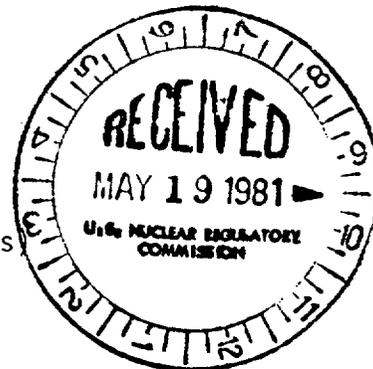
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NRC PDR
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C. Parrish
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OELD
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G. Deegan (4)
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MAY 13 1981

Docket No. 50-316

Mr. John Dolan, Vice President
Indiana and Michigan Electric Company
Post Office Box 18
Bowling Green Station
New York, New York 10008



Dear Mr. Dolan:

The Commission has issued the enclosed Amendment No. 31 to Facility Operating License No. DPR-74 for the Donald C. Cook Nuclear Plant, Unit No. 2. The amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated May 1, 1981 as supplemented by letter dated May 7, 1981.

This amendment revises the Axial Power Distribution Monitoring System (APDMS) technical specifications and the setpoint for the P-7 permissive (interlock).

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

Original Signed By:

Steven A. Varga, Chief
Operating Reactors Branch No. 1
Division of Licensing

Enclosures:

- 1. Amendment No. 31 to DPR-74
- 2. Safety Evaluation
- 3. Notice of Issuance

cc: w/enclosures
See next page

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W. legal objection

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| OFFICE ▶ | ORB 1 | ORB 1 | ORB 1 | AD:OR | OELD | | |
| SURNAME ▶ | C.Parrish | S.Minor/rs | S.Varga | T.Novak | R. Long | | |
| DATE ▶ | 5/1/81 | 5/13/81 | 5/13/81 | 5/13/81 | 5/13/81 | | |

MAY 13 1981

Mr. John Dolan
Indiana and Michigan Electric Company

cc: Mr. Robert W. Jurgensen
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

INDIANA AND MICHIGAN ELECTRIC COMPANY

DOCKET NO. 50-316

DONALD C. COOK NUCLEAR PLANT UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 31
License No. DPR-74

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Indiana and Michigan Electric Company (the licensee) dated May 1, 1981 as supplemented by letter dated May 7, 1981, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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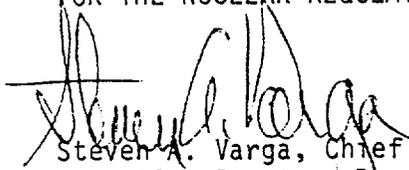
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-74 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 31, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Steven A. Varga, Chief
Operating Reactors Branch No. 1
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: MAY 13 1981

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 31 TO FACILITY OPERATING LICENSE NO. DPR-74

DOCKET NO. 50-316

Revise Appendix A as follows:

Remove Pages

3/4 2-1
3/4 2-2
3/4 2-3
3/4 2-4
3/4 2-17
3/4 2-18
3/4 2-19
3/4 3-7
3/4 3-8
B 3/4 2-1
B 3/4 2-2

Insert Pages

3/4 2-1
3/4 2-2
3/4 2-3*
3/4 2-4
3/4 2-17
3/4 2-18
3/4 2-19
3/4 3-7*
3/4 3-8
B 3/4 2-1*
B 3/4 2-2

*No change, provided for convenience

3/4.2 POWER DISTRIBUTION LIMITS

AXIAL FLUX DIFFERENCE (AFD)

LIMITING CONDITION FOR OPERATION

3.2.1 The indicated AXIAL FLUX DIFFERENCE (AFD) shall be maintained within a +5% target band (flux difference units) about the target flux difference.

APPLICABILITY: MODE 1 ABOVE 50% RATED THERMAL POWER*

ACTION:

- a. With the indicated AXIAL FLUX DIFFERENCE outside of the +5% target band about the target flux difference and with THERMAL POWER:
 1. Above 81% of RATED THERMAL POWER, within 15 minutes:
 - a) Either restore the indicated AFD to within the target band limits, or
 - b) Reduce THERMAL POWER to less than 81% of RATED THERMAL POWER.
 2. Between 50% and 81% of RATED THERMAL POWER:
 - a) POWER OPERATION may continue provided:
 - 1) The indicated AFD has not been outside of the +5% target band for more than 1 hour penalty deviation cumulative during the previous 24 hours, and
 - 2) The indicated AFD is within the limits shown on Figure 3.2-1. Otherwise, reduce THERMAL POWER to less than 50% of RATED THERMAL POWER within 30 minutes and reduce the Power Range Neutron Flux-High Trip Setpoints to ≤ 55% of RATED THERMAL POWER within the next 4 hours.
 - b) Surveillance testing of the Power Range Neutron Flux Channels may be performed pursuant to Specification 4.3.1.1.1 provided the indicated AFD is maintained within the limits of Figure 3.2-1. A total of 16 hours operation may be accumulated with the AFD outside of the target band during this testing without penalty deviation.

*See Special Test Exception 3.10.2

POWER DISTRIBUTION LIMITS

ACTION: (Continued)

- c) Surveillance testing of the APDMS may be performed pursuant to Specification 4.3.3.7.1 provided the indicated AFD is maintained within the limits of Figure 3.2-1. A total of 6 hours of operation may be accumulated with the AFD outside of the target band during this testing without penalty deviation.
- b. THERMAL POWER shall not be increased above 81% of RATED THERMAL POWER unless the indicated AFD is within the $\pm 5\%$ target band and ACTION 2.a) 1), above has been satisfied.
- c. THERMAL POWER shall not be increased above 50% of RATED THERMAL POWER unless the indicated AFD has not been outside of the $\pm 5\%$ target band for more than 1 hour penalty deviation cumulative during the previous 24 hours.

SURVEILLANCE REQUIREMENTS

4.2.1.1 The indicated AXIAL FLUX DIFFERENCE shall be determined to be within its limits during POWER OPERATION above 15% of RATED THERMAL POWER by:

- a. Monitoring the indicated AFD for each OPERABLE excore channel:
 - 1. At least once per 7 days when the AFD Monitor Alarm is OPERABLE, and
 - 2. At least once per hour for the first 24 hours after restoring the AFD Monitor Alarm to OPERABLE status.
- b. Monitoring and logging the indicated AXIAL FLUX DIFFERENCE for each OPERABLE excore channel at least once per hour for the first 24 hours and at least once per 30 minutes thereafter, when the AXIAL FLUX DIFFERENCE Monitor Alarm is inoperable. The logged values of the indicated AXIAL FLUX DIFFERENCE shall be assumed to exist during the interval preceding each logging.

POWER DISTRIBUTION LIMITS

SURVEILLANCE REQUIREMENTS (Continued)

4.2.1.2 The indicated AFD shall be considered outside of its +5% target band when at least 2 of 4 or 2 of 3 OPERABLE excore channels are indicating the AFD to be outside the target band. Penalty deviation outside of the +5% target band shall be accumulated on a time basis of:

- a. One minute penalty deviation for each one minute of POWER OPERATION outside of the target band at THERMAL POWER levels equal to or above 50% of RATED THERMAL POWER, and
- b. One-half minute penalty deviation for each one minute of POWER OPERATION outside of the target band at THERMAL POWER levels between 15% and 50% of RATED THERMAL POWER.

4.2.1.3 The target flux difference of each OPERABLE excore channel shall be determined by measurement at least once per 92 Effective Full Power Days. The provisions of Specification 4.0.4 are not applicable.

4.2.1.4 The target flux difference shall be updated at least once per 31 Effective Full Power Days by either determining the target flux difference pursuant to 4.2.1.3 above or by linear interpolation between the most recently measured value and 0 percent at the end of the cycle life. The provisions of Specification 4.0.4 are not applicable.

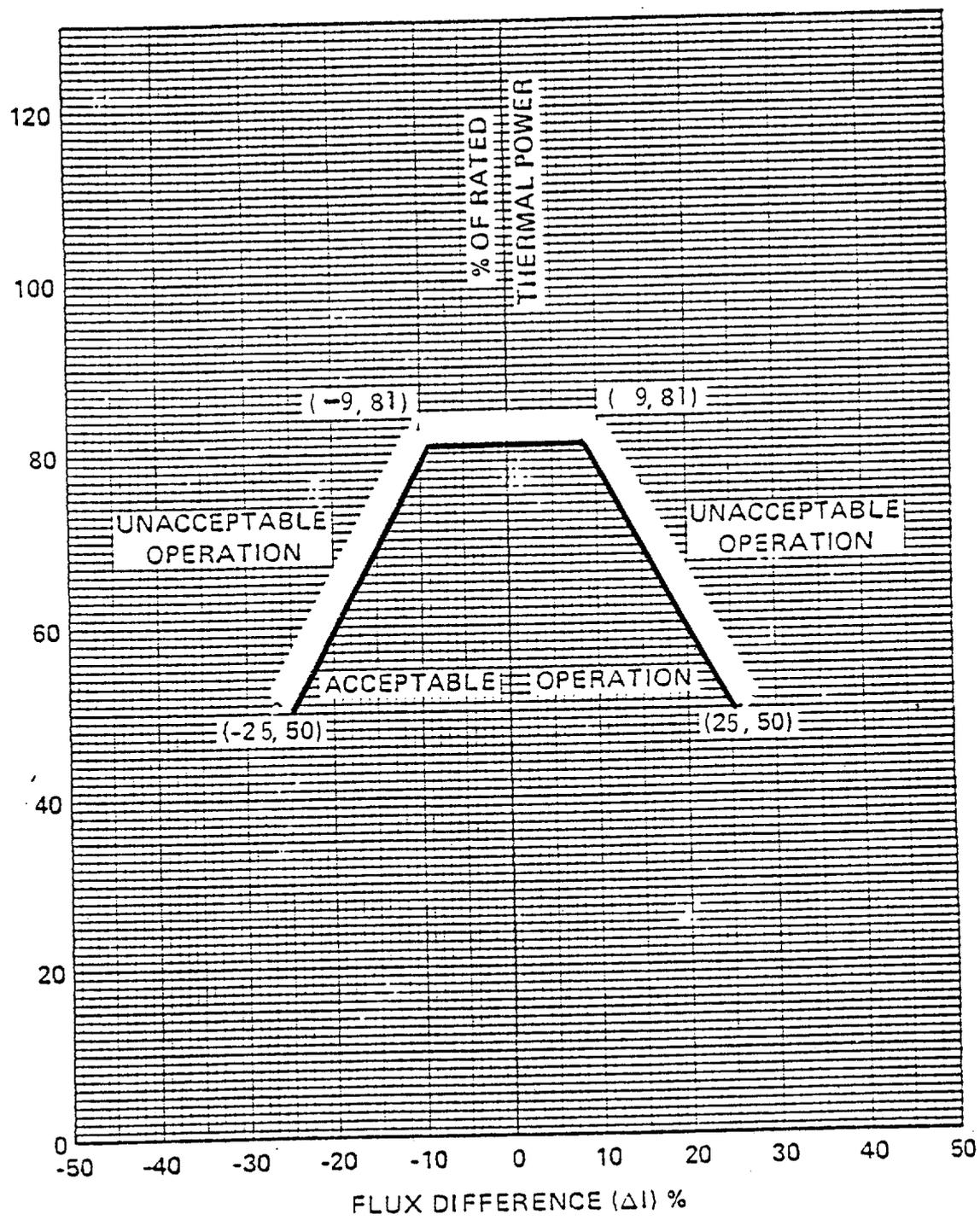


FIGURE 3.2-1 AXIAL FLUX DIFFERENCE LIMITS AS A FUNCTION OF RATED THERMAL POWER

POWER DISTRIBUTION LIMITS

AXIAL POWER DISTRIBUTION

LIMITING CONDITION FOR OPERATION

3.2.6 The axial power distribution shall be limited by the following relationship:

$$[F_j(Z)]_s = \frac{[1.99] [K(Z)]}{(\bar{R}_j)(P_L) (.103) (1 + \sigma_j)(1.07)}$$

Where:

- a. $F_j(Z)$ is the normalized axial power distribution from thimble j at core elevation Z .
- b. P_L is the fraction of RATED THERMAL POWER.
- c. $K(Z)$ is the function obtained from Figure 3.2-2 for a given core height location.
- d. \bar{R}_j , for thimble j , is determined from at least $n=6$ in-core flux maps covering the full configuration of permissible rod patterns above 91% of RATED THERMAL POWER in accordance with:

$$\bar{R}_j = \frac{1}{n} \sum_{i=1}^n R_{ij}$$

Where:

$$R_{ij} = \frac{F_{Q_i}^{Meas}}{[F_{ij}(Z)]_{max}}$$

and $[F_{ij}(Z)]_{max}$ is the maximum value of the normalized axial distribution at elevation Z from thimble j in map i which has a measured peaking factor without uncertainties or densification allowance of F_Q^{Meas} .

POWER DISTRIBUTION LIMITS

LIMITING CONDITION FOR OPERATION (Continued)

σ_j is the standard deviation associated with thimble j, expressed as a fraction or percentage of \bar{R}_j , and is derived from n flux maps from the relationship below, or 0.02, (2%) whichever is greater.

$$\sigma_j = \frac{\left[\frac{1}{n-1} \sum_{i=1}^n (\bar{R}_j - R_{ij})^2 \right]^{1/2}}{\bar{R}_j}$$

The factor 1.07 is comprised of 1.02 and 1.05 to account for the axial power distribution instrumentation accuracy and the measurement uncertainty associated with F_Q using the movable detector system respectively.

The factor 1.03 is the engineering uncertainty factor.

APPLICABILITY: MODE 1 above 91% OF RATED THERMAL POWER[#].

ACTION:

- a. With a $F_j(Z)$ factor exceeding $[F_j(Z)]_S$ by ≤ 4 percent, reduce THERMAL POWER one percent for every percent by which the $F_j(Z)$ factor exceeds its limit within 15 minutes and within the next two hours either reduce the $F_j(Z)$ factor to within its limit or reduce THERMAL POWER to 91% or less of RATED THERMAL POWER.
- b. With a $F_j(Z)$ factor exceeding $[F_j(Z)]_S$ by > 4 percent, reduce THERMAL POWER to 91% or less of RATED THERMAL POWER within 15 minutes.

[#]The APDMS may be out of service when surveillance for determining power distribution maps is being performed.

POWER DISTRIBUTION LIMITS

SURVEILLANCE REQUIREMENTS

4.2.6.1 $F_j(Z)$ shall be determined to be within its limit by:

- a. Either using the APDMS to monitor the thimbles required per Specification 3.3.3.6 at the following frequencies.
 1. At least once per 8 hours, and
 2. Immediately and at intervals of 10, 30, 60, 90, 120, 240 and 480 minutes following:
 - a) Increasing the THERMAL POWER above 91% of RATED THERMAL POWER, or
 - b) Movement of control bank "D" more than an accumulated total of 5 steps in any one direction.
- b. Or using the movable incore detectors at the following frequencies when the APDMS is inoperable:
 1. At least once per 8 hours, and
 2. At intervals of 30, 60, 90, 120, 240 and 480 minutes following:
 - a) Increasing the THERMAL POWER above 91% of RATED THERMAL POWER, or
 - b) Movement of control bank "D" more than an accumulated total of 5 steps in any one direction.

4.2.6.2 When the movable incore detectors are used to monitor $F_j(Z)$, at least 2 thimbles shall be monitored and an $F_j(Z)$ accuracy equivalent to that obtained from the APDMS shall be maintained.

TABLE 3.3-1 (Continued)

- ACTION 8 - With the number of OPERABLE channels one less than the Total Numbers of Channels and with the THERMAL POWER level above P-7, place the inoperable channel in the tripped condition within 1 hour; operation may continue until performance of the next required CHANNEL FUNCTIONAL TEST.
- ACTION 9 - With a channel associated with an operating loop inoperable, restore the inoperable channel to OPERABLE status within 2 hours or be in HOT STANDBY within the next 6 hours; however, one channel associated with an operating loop may be bypassed for up to 2 hours for surveillance testing per Specification 4.3.1.1.1.
- ACTION 10 - With one channel inoperable, restore the inoperable channel to OPERABLE status within 2 hours or reduce THERMAL POWER to below P-8 within the next 2 hours. Operation below P-8 may continue pursuant to ACTION 11.
- ACTION 11 - With less than the Minimum Number of Channels OPERABLE, operation may continue provided the inoperable channel is placed in the tripped condition within 1 hour.
- ACTION 12 - With the number of channels OPERABLE one less than required by the Minimum Channels OPERABLE requirement, restore the inoperable channel to OPERABLE status within 48 hours or be in HOT STANDBY within the next 6 hours and/or open the reactor trip breakers.

REACTOR TRIP SYSTEM INTERLOCKS

| <u>DESIGNATION</u> | <u>CONDITION AND SETPOINT</u> | <u>FUNCTION</u> |
|--------------------|--|--|
| P-6 | With 2 of 2 Intermediate Range Neutron Flux Channels $< 6 \times 10^{-11}$ amps. | P-6 prevents or defeats the manual block of source range reactor trip. |

TABLE 3.3-1 (Continued)

| <u>DESIGNATION</u> | <u>CONDITION AND SETPOINT</u> | <u>FUNCTION</u> |
|--------------------|---|---|
| P-7 | With 2 of 4 Power Range Neutron Flux Channels \geq 11% of RATED THERMAL POWER or 1 of 2 Turbine impulse chamber pressure channels \geq 66 psia. | P-7 prevents or defeats the automatic block of reactor trip on: Low flow in more than one primary coolant loop, reactor coolant pump under-voltage and under-frequency, turbine trip, pressurizer low pressure, and pressurizer high level. |
| P-8 | With 2 of 4 Power Range Neutron Flux channels \geq 51% of RATED THERMAL POWER. | P-8 prevents or defeats the automatic block of reactor trip on low coolant flow in a single loop. |
| P-10 | With 3 of 4 Power range neutron flux channels $<$ 9% of RATED THERMAL POWER. | P-10 prevents or defeats the manual block of: Power range low setpoint reactor trip, Intermediate range reactor trip, and intermediate range rod stops. Provides input to P-7. |

3/4.2 POWER DISTRIBUTION LIMITS

BASES

The specifications of this section provide assurance of fuel integrity during Condition I (Normal Operation) and II (Incidents of Moderate Frequency) events by: (a) maintaining the calculated DNBR in the core at or above design during normal operation and in short term transients, and (b) limiting the fission gas release, fuel pellet temperature and cladding mechanical properties to within assumed design criteria. In addition, limiting the peak linear power density during Condition I events provides assurance that the initial conditions assumed for the LOCA analyses are met and the ECCS acceptance criteria limit of 2200°F is not exceeded.

The definitions of certain hot channel and peaking factors as used in these specifications are as follows:

- $F_Q(Z)$ Heat Flux Hot Channel Factor, is defined as the maximum local heat flux on the surface of a fuel rod at core elevation Z divided by the average fuel rod heat flux, allowing for manufacturing tolerances on fuel pellets and rods.
- $F_{\Delta H}^N$ Nuclear Enthalpy Rise Hot Channel Factor, is defined as the ratio of the integral of linear power along the rod with the highest integrated power to the average rod power.
- $F_{xy}(Z)$ Radial Peaking Factor, is defined as the ratio of peak power density to average power density in the horizontal plane at core elevation Z .

3/4.2.1 AXIAL FLUX DIFFERENCE (AFD)

The limits on AXIAL FLUX DIFFERENCE assure that the $F_Q(Z)$ upper bound envelope of 1.99 times the normalized axial peaking factor is not exceeded during either normal operation or in the event of xenon redistribution following power changes.

Target flux difference is determined at equilibrium xenon conditions. The full length rods may be positioned within the core in accordance with their respective insertion limits and should be inserted near their normal position for steady state operation at high power levels. The value of the target flux difference obtained under these conditions divided by the fraction of RATED THERMAL POWER is the target flux difference at RATED THERMAL POWER for the associated core burnup conditions. Target flux differences for other THERMAL POWER levels are obtained by multiplying the RATED THERMAL POWER value by the appropriate fractional THERMAL POWER level. The periodic updating of the target flux difference value is necessary to reflect core burnup considerations.

3/4.2 POWER DISTRIBUTION LIMITS

BASES

Although it is intended that the plant will be operated with the AXIAL FLUX DIFFERENCE within the +5% target band about the target flux difference, during rapid plant THERMAL POWER reductions, control rod motion will cause the AFD to deviate outside of the target band at reduced THERMAL POWER levels. This deviation will not affect the xenon redistribution sufficiently to change the envelope of peaking factors which may be reached on a subsequent return to RATED THERMAL POWER (with the AFD within the target band) provided the time duration of the deviation is limited. Accordingly, a 1 hour penalty deviation limit cumulative during the previous 24 hours is provided for operation outside of the target band but within the limits of Figure 3.2-1 while at THERMAL POWER levels between 50% and 81% of RATED THERMAL POWER. For THERMAL POWER levels between 15% and 50% of RATED THERMAL POWER, deviations of the AFD outside of the target band are less significant. The penalty of 2 hours actual time reflects this reduced significance.

Provisions for monitoring the AFD on an automatic basis are derived from the plant process computer through the AFD Monitor Alarm. The computer determines the one minute average of each of the OPERABLE excore detector outputs and provides an alarm message immediately if the AFD for at least 2 of 4 or 2 of 3 OPERABLE excore channels are outside the target band and the THERMAL POWER is greater than 81% of RATED THERMAL POWER. During operation at THERMAL POWER levels between 50% and 81% and between 15% and 50% RATED THERMAL POWER, the computer outputs an alarm message when the penalty deviation accumulates beyond the limits of 1 hour and 2 hours, respectively.

Figure B 3/4 2-1 shows a typical monthly target band.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

AMENDMENT NO. 31 TO FACILITY OPERATING LICENSE NO. DPR-74

INDIANA AND MICHIGAN ELECTRIC COMPANY

DONALD C. COOK NUCLEAR PLANT UNIT NO. 2

DOCKET NO. 50-316

Introduction

In a letter dated May 1, 1981 to H. R. Denton, Indiana and Michigan Electric Company (I&MEC) requested changes to the D. C. Cook Unit 2 Technical Specifications for the Axial Power Distribution Monitoring System (APDMS) to accommodate a slightly different core power profile predicted for the Cycle 3 design of the reactor. In addition in the May 1, 1981 letter I&MEC requested changes to the P-7 permissive (interlock) technical specification. By letter dated May 7, 1981 I&MEC provided additional information with regard to the APDMS technical specification change requests.

Evaluation

1. Axial Power Distribution Monitoring System (APDMS).

Westinghouse Electric Corporation has performed calculations for the D. C. Cook Unit 2 Cycle 3 reload which predict a slightly higher axial core power profile than the previous cycle. The calculations used approved techniques for predicting the maximum height-dependent peaking factors expected during a reload Cycle 1 and 2.

There is no change in the peaking factor envelope (1.99 x the axial shaping curve) which must be maintained in normal operation of the power plant because these peaking factors are used as initial conditions in the LOCA analysis. However, above the power level which is defined as the ratio of the highest predicted axially dependent peaking factor to the LOCA envelope, additional surveillance of the peaking factor is required in the form of the Axial Power Distribution Monitoring System (APDMS). This level, called the APDMS turn on power is reduced to 81 percent of rated thermal power for the predicted Cycle 3 peaking factor envelope.

Because of the reduced APDMS turn on point, the axial flux difference limits as a function of thermal power are also changed. We have reviewed the change as well as the APDMS turn on point and find them satisfactory.

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Since the proposed Technical Specification changes use approved techniques and changes do not reduce the safety margins required to meet 10 CFR 50.46 and Appendix K, we find them acceptable for Cycle 3 operation of D. C. Cook Unit 2.

2. P-7 Permissive (Interlock).

The P-7 permissive (interlock) is applied to remove the automatic block to reactor trip (when reactor power increases above approximately 10 percent) from low reactor coolant flow in more than one loop, reactor coolant pump under voltage, reactor coolant pump under frequency, turbine trip, pressurizer low pressure and pressurizer high level. A two out of four Power Range Neutron Flux channel trip or, a one out of the Turbine Impulse Chamber Pressure channel trip will remove the automatic block.

I&MEC in the recent refueling outage has modified the unit's high pressure turbine and has raised the Impulse Chambers Pressure channel trip setpoint to 66 psia (previously set at 55 psia). The new trip setpoint will not violate the approximately 10 percent reactor power limit for removal of the automatic block to reactor trip. We have reviewed the I&MEC proposed interlock P-7 trip setpoint change, in Table 3.3-1 of the technical specifications for D. C. Cook Unit 2 and find it acceptable.

Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of the amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: MAY 13 1981

REFERENCES

1. "F₀ Envelop Calculations," C. E. Eicheldinger letter NS-CE-687 to D. B. Vassallo (NRC) dated June 27, 1975.
2. "Justification of Peaking Factor Subcase Analysis," C. E. Eicheldinger letter NS-CE-1749 to J. F. Stolz (NRC) dated April 6, 1978.

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-316INDIANA AND MICHIGAN ELECTRIC COMPANYNOTICE OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 31 to Facility Operating License No. DPR-74 issued to Indiana and Michigan Electric Company (the licensee), which revised Technical Specifications for operation of Donald C. Cook Nuclear Plant, Unit No. 2 (the facility) located in Berrien County, Michigan. The amendment is effective as of the date of issuance.

The amendment revises the Axial Power Distribution Monitoring System (APDMS) technical specifications and the setpoint for the P-7 permissive (interlock).

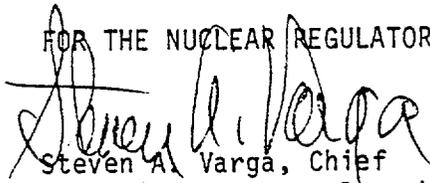
The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of the amendment was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of the amendment.

For further details with respect to this action, see (1) the application for amendment dated May 1, 1981 as supplemented May 7, 1981, (2) Amendment No. 31 to License No. DPR-74, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C., and at the Maude Reston Palenske Memorial Library, 500 Market Street, St. Joseph, Michigan 49085. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 13th day of May 1981.

FOR THE NUCLEAR REGULATORY COMMISSION


Steven A. Varga, Chief
Operating Reactors Branch No. 1
Division of Licensing