UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

April 6, 1995

MEMORANDUM TO: Sholly Coordinator

FROM: John B. Hickman, Project Manager
Project Directorate III-1
Division of Reactor Projects - III/IV

SUBJECT: REQUEST FOR PUBLICATION IN BIWEEKLY FR NOTICE - NOTICE OF CONSIDERATION OF ISSUANCE OF AMENDMENT TO FACILITY OPERATING LICENSE, PROPOSED NO SIGNIFICANT HAZARDS CONSIDERATION DETERMINATION, AND OPPORTUNITY FOR A HEARING (TAC NO. M91853)

Indiana Michigan Power Company, Docket No. 50-315, Donald C. Cook Nuclear Plant, Unit No. 1, Berrien County, Michigan

Date of amendment request: March 17, 1995

Description of amendment request: The proposed amendment would defer performance of the Type A containment integrated leakage rate test until the next refueling outage.

Basis for proposed no significant hazards consideration determination: As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

Per 10 CFR 50.92, a proposed change does not involve a significant hazards consideration if the change does not:

1. involve a significant increase in the probability or consequences of an accident previously evaluated,

2. create the possibility of a new or different kind of accident from any accident previously evaluated, or

3. involve a significant reduction in a margin of safety.

Criterion 1

The Cook Nuclear Plant Type A test history provides substantial justification for the proposed test schedule. Three Type A tests were performed over a seven year period with successful results. The tests indicate that the Cook Nuclear Plant has a low leakage containment. In addition, there are no adverse trends in the results from the previous Types A, B, and C tests or visual inspections that indicate

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a gradual degradation of the containment boundary. Further, there are no structural modifications planned which would adversely affect the structural capability of the containment and that would be a factor in deferring the Type A test one refueling outage. Containment leak rate testing is not an initiator of any accident. The proposed interval extension does not affect reactor operations or the accident analysis and has no radiological consequences, except for the dose savings associated with not performing the test. There will be no changes to 10 CFR 100 dose limits or the control room dose limits. Extending the test interval will not increase the probability of a malfunction of equipment important to safety. Based on these considerations, it is concluded that the change does not involve a significant increase in the probability or consequences of an accident previously evaluated.

Criterion 2

The proposed change does not involve physical changes to the plant or changes in plant operating configuration. The proposed change only relaxes the schedular requirements for conducting one Type A test from the T/Ss and defers performance of the test one cycle. The purpose of the test is to provide periodic verification of the leak-tight integrity of the primary reactor containment, and systems and components which penetrate containment. The tests assure that leakage through containment and systems and components penetrating containment will not exceed the allowable leak rate values established in 10 CFR 50, Appendix J. Thus, it is concluded that the proposed change does not create the possibility of a new or different kind of accident from any accident previously evaluated.

Criterion 3

The proposed change to the schedule for performing the Type A test does not reduce the margin of safety assumed in the accident analysis for any release of radioactive materials or reduce any margin of safety preserved by the technical specifications. The methodology, acceptance criteria, and the technical specification leak rate limits for the performance of the Type A test will not change. Type A tests will continue to be performed in accordance with 10 CFR 50, Appendix J and the applicable Cook Nuclear Plant Technical Specifications beginning in 1997. In addition, there are no adverse trends in the results from the previous Type A, B, and C tests or visual inspections that indicate a gradual degradation of the containment boundary. Based on these considerations, it is concluded that the change does not involve a significant reduction in a margin of safety.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

Local Public Document Room location: Maud Preston Palenske Memorial Library, 500 Market Street, St. Joseph, Michigan 49085

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