

August 15, 1996

Mr. E. E. Fitzpatrick, Vice President
Indiana Michigan Power Company
c/o American Electric Power Service Corporation
1 Riverside Plaza
Columbus, OH 43215

SUBJECT: DONALD C. COOK NUCLEAR PLANT, UNIT NOS. 1 AND 2 - ISSUANCE OF
AMENDMENTS RE: PORVS AND BLOCK VALVES EMERGENCY POWER SURVEILLANCE
(TAC NOS. M94563 AND M94564)

Dear Mr. Fitzpatrick:

The Commission has issued the enclosed Amendment No. 211 to Facility Operating License No. DPR-58 and Amendment No. 196 to Facility Operating License No. DPR-74 for the Donald C. Cook Nuclear Plant, Unit Nos. 1 and 2. The amendments consist of changes to the Technical Specifications in response to your application dated January 12, 1996.

The amendments modify the Technical Specifications to delete the surveillance requirement demonstrating operability of the emergency power supply for the pressurizer power operated relief valves and block valves.

A copy of our related Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Original signed by:

John B. Hickman, Project Manager
Project Directorate III-1
Division of Reactor Projects - III/IV
Office of Nuclear Reactor Regulation

Docket Nos. 50-315 and 50-316

- Enclosures: 1. Amendment No. 211 to DPR-58
- 2. Amendment No. 196 to DPR-74
- 3. Safety Evaluation

cc w/encl: See next page

DOCUMENT NAME: G:\WPDOCS\DCCOOK\C094563.AMD

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NAME	CJamerson	<input checked="" type="checkbox"/>	JHickman	<input checked="" type="checkbox"/>	RJones	<input checked="" type="checkbox"/>			FMReinhart	<input checked="" type="checkbox"/>
DATE	7/3/96	<input checked="" type="checkbox"/>	7/8/96	<input checked="" type="checkbox"/>	7/11/96	<input checked="" type="checkbox"/>	7/19/96	<input checked="" type="checkbox"/>	8/13/96	<input checked="" type="checkbox"/>

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Indiana Michigan Power Company

Donald C. Cook Nuclear Plant

cc:

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Lansing, Michigan 48909-8130

DATED: August 15, 1996

AMENDMENT NO. TO FACILITY OPERATING LICENSE NO. DPR-58-D. C. COOK-UNIT 1
AMENDMENT NO. TO FACILITY OPERATING LICENSE NO. DPR-74-D. C. COOK-UNIT 2

Docket File
PUBLIC
PDIII-1 Reading
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C. Jamerson
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G. Hill, IRM (4)
C. Grimes, O-11F23
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

INDIANA MICHIGAN POWER COMPANY

DOCKET NO. 50-315

DONALD C. COOK NUCLEAR PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 211
License No. DPR-58

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Indiana Michigan Power Company (the licensee) dated January 12, 1996, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

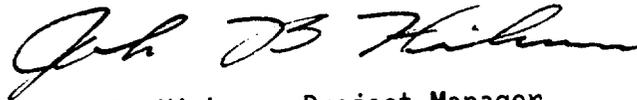
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-58 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 211, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance, with full implementation within 45 days.

FOR THE NUCLEAR REGULATORY COMMISSION



John B. Hickman, Project Manager
Project Directorate III-1
Division of Reactor Projects - III/IV
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance: August 15, 1996

ATTACHMENT TO LICENSE AMENDMENT NO. 211
TO FACILITY OPERATING LICENSE NO. DPR-58
DOCKET NO. 50-315

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the attached pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change.

REMOVE

3/4 4-36

INSERT

3/4 4-36

LIMITING CONDITION FOR OPERATION (Continued)

- g. With PORVs and block valves not in the same line inoperable due to causes other than excessive seat leakage, within 1 hour restore the valves to OPERABLE status or close and de-energize the associated block valve and place the associated PORV in manual control in each respective line. Apply the portions of ACTION c or d above, relating to the OPERATIONAL MODE, as appropriate for two or three lines unavailable.
- h. The provisions of Specification 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

- 4.4.11.1 In addition to the requirements of Specification 4.0.5, each PORV shall be demonstrated OPERABLE:
 - a. At least once per 31 days by performance of a CHANNEL FUNCTIONAL TEST, excluding valve operation, and
 - b. At least once per 18 months by operating the PORV through one complete cycle of full travel during MODES 3 or 4, and
 - c. At least once per 18 months by operating solenoid air control valves and check valves in PORV control systems through one complete cycle of full travel, and
 - d. At least once per 18 months by performing a CHANNEL CALIBRATION of the actuation instrumentation.
- 4.4.11.2 Each block valve shall be demonstrated OPERABLE at least once per 92 days by operating the valve through one complete cycle of full travel unless the block valve is closed in order to meet the requirements of ACTION b, c, or d in Specification 3.4.11.
- 4.4.11.3 Deleted.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

INDIANA MICHIGAN POWER COMPANY

DOCKET NO. 50-316

DONALD C. COOK NUCLEAR PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 196
License No. DPR-74

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Indiana Michigan Power Company (the licensee) dated January 12, 1996, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

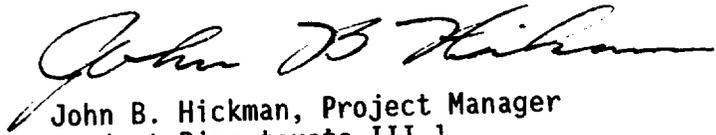
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-74 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 196, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance, with full implementation within 45 days.

FOR THE NUCLEAR REGULATORY COMMISSION



John B. Hickman, Project Manager
Project Directorate III-1
Division of Reactor Projects - III/IV
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of Issuance: August 15, 1996

ATTACHMENT TO LICENSE AMENDMENT NO. 196

FACILITY OPERATING LICENSE NO. DPR-74

DOCKET NO. 50-316

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the attached pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change.

REMOVE

3/4 4-33

INSERT

3/4 4-33

REACTOR COOLANT SYSTEM

LIMITING CONDITION FOR OPERATION (Continued)

- g. With PORVs and block valves not in the same line inoperable due to causes other than excessive seat leakage, within 1 hour restore the valves to OPERABLE status or close and de-energize the associated block valve and place the associated PORV in manual control in each respective line. Apply the portions of ACTION c or d above, relating to the OPERATIONAL MODE, as appropriate for two or three lines unavailable.
- h. The provisions of Specification 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

- 4.4.11.1 In addition to the requirements of Specification 4.0.5, each PORV shall be demonstrated OPERABLE:
 - a. At least once per 31 days by performance of a CHANNEL FUNCTIONAL TEST, excluding valve operation, and
 - b. At least once per 18 months by operating the PORV through one complete cycle of full travel during MODES 3 or 4, and
 - c. At least once per 18 months by operating solenoid air control valves and check valves in PORV control systems through one complete cycle of full travel, and
 - d. At least once per 18 months by performing a CHANNEL CALIBRATION of the actuation instrumentation.[†]
- 4.4.11.2 Each block valve shall be demonstrated OPERABLE at least once per 92 days by operating the valve through one complete cycle of full travel unless the block valve is closed in order to meet the requirements of ACTION b, c, or d in Specification 3.4.11.
- 4.4.11.3 Deleted.

[†]The provisions of Technical Specification 4.0.8 are applicable.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 211 TO FACILITY OPERATING LICENSE NO. DPR-58
AND AMENDMENT NO. 196 TO FACILITY OPERATING LICENSE NO. DPR-74

INDIANA MICHIGAN POWER COMPANY

DONALD C. COOK NUCLEAR PLANT, UNIT NOS. 1 AND 2

DOCKET NOS. 50-315 AND 50-316

1.0 INTRODUCTION

By letter dated January 12, 1996, the Indiana Michigan Power Company (the licensee) requested amendments to the Technical Specifications (TS) appended to Facility Operating License Nos. DPR-58 and DPR-74 for the Donald C. Cook Nuclear Plant, Unit Nos. 1 and 2. The proposed amendments would delete surveillance requirement 4.4.11.3, which requires demonstrating the operability of the emergency power supply for the pressurizer power operated relief valves (PORVs) and block valves.

2.0 EVALUATION

The Cook Nuclear Plant pressurizer has three PORVs which limit reactor coolant system pressure. The PORVs, which are spring closed and air-to-open upon actuation of their associated solenoid valve, operate automatically or by remote manual control. Remotely operated stop valves are provided to isolate the PORVs. The solenoid and block valves are electrically operated and powered from emergency buses. The PORV solenoids are powered by the 250V DC power supply and the block valves are powered by the 600V AC power supply.

TS 4.4.11.3 currently states: "The emergency power supply for the PORVs and block valves shall be demonstrated OPERABLE at least once per 18 months by operating the valves through a complete cycle of full travel while the emergency buses are energized by the onsite diesel generators and the onsite plant batteries. This testing can be performed in conjunction with the requirements of Specification 4.8.1.1.2.e and 4.8.2.3.2.d."

The licensee states that TS 4.4.11.3 was added in response to a NUREG-0578, "TMI-2 Lessons Learned Task Force Status Report and Short-Term Recommendations," post-TMI requirement. The requirement was based on the majority plant design which powered the PORVs and block valves from non-emergency power supplies. Although the Cook PORVs and block valves are permanently powered from Class 1E emergency buses, the licensee still implemented TS based on the NUREG requirement.

The current TS allows testing of the PORVs and block valves in conjunction with TS 4.8.1.1.2.e and 4.8.2.3.2.d. TS 4.8.1.1.2.e requires verifying the operability of the Emergency Diesel Generator (EDG) including its ability to start on the appropriate signals and to energize the emergency busses. During this test, the EDG powers the 600V AC power buses directly and powers the 250V DC power buses by means of battery chargers. TS 4.8.2.3.2.d requires performing a plant battery service test during shutdown to verify that the battery capacity is adequate to supply the actual or simulated emergency loads with the battery charger disconnected. The licensee states that the coincident testing was implemented to allow testing of the PORVs and block valves during testing that was already required to be performed on the EDGs and the plant batteries. These two surveillances, which demonstrate operability of the Class 1E emergency power supplies are not proposed to be changed by this amendment.

In addition, TS 4.4.11.1 and 4.4.11.2 are not proposed to be changed by this amendment. TS 4.4.11.1 requires demonstration of PORV operability by operating the PORV and solenoid air control and check valves through one complete cycle of full travel at least once per 18 months. During this surveillance the PORV's associated solenoid valves are powered by the 250V DC bus. TS 4.4.11.2 requires demonstration of block valve operability by operating the valve through one complete cycle of full travel at least once per 92 days. During this surveillance the block valve is powered by the 600V AC bus. Since these surveillances are conducted with the valves connected to their normal (emergency) buses, the surveillance demonstrated both the operability of the valves and the continuity of the valve's connection to the bus.

Section 50.36 of Title 10 of the Code of Federal Regulations established the regulatory requirements related to the content of TS. The rule requires that TS include items in specific categories, including safety limits, limiting conditions for operation, and surveillance requirements. The rule specifies that limiting conditions for operation (and associated surveillances) are to be included in a plant's TS if the item meets one or more of the following criteria: (1) an installed instrumentation that is used to detect, and indicate in the control room, a significant abnormal degradation of the reactor coolant pressure boundary, (2) a process variable, design feature, or operating restriction that is an initial condition of a design basis accident or transient analysis that either assumes the failure of or presents a challenge to the integrity of a fission product barrier, (3) a structure, system, or component that is part of the primary success path and which functions or actuates to mitigate a design basis accident or transient that either assumes the failure of or presents a challenge to the integrity of a fission product barrier, or (4) a structure, system, or component which operating experience or probabilistic risk assessment has shown to be significant to public health and safety.

The staff's review of the proposed change determined that the functionality of the emergency buses, the PORV solenoid valves, the block valves, and the connections between the valves and buses are all verified by other surveillances. Therefore, the performance of TS 4.4.11.3 constitutes an integrated test that is redundant to the other testing and unnecessary. Since

the operability of the PORV and block valves is fully demonstrated by other surveillances, TS 4.4.11.3 would not be specifically required by 10 CFR 50.36. Additionally, the staff has previously stated in NUREG 1431, "Standard Technical Specifications Westinghouse Plants," dated April 1995, that the surveillance which requires demonstrating emergency power operation of the PORV and block valve is not required for plants with permanent IE power supplies to the valve.

Based on the surveillance being unnecessary for the Cook plant design, that inclusion of the specific surveillance in TS is not required by 10 CFR 50.36 or other regulations, and the previously stated staff position, the proposed deletion of TS 4.4.11.3 is acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Michigan State official was notified of the proposed issuance of the amendments. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendments change surveillance requirements. The staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration and there has been no public comment on such finding (61 FR 7554). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

5.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: John Hickman, NRR

Date: August 15, 1996