

September 26, 1997

Mr. E. E. Fitzpatrick, Vice President
Indiana Michigan Power Company
Nuclear Generation Group
500 Circle Drive
Buchanan, MI 49107

SUBJECT: PUBLIC NOTICE; NRC STAFF PROPOSES TO AMEND OPERATING
LICENSES FOR DONALD C. COOK NUCLEAR PLANT, UNITS 1 AND 2

Dear Mr. Fitzpatrick:

Enclosed is a copy of the public notice that will be published in the *Herald-Palladium*, Benton Harbor, MI, as a result of exigent circumstances relating to your application for technical specification amendment dated September 19, 1997, modifying the Residual Heat Removal automatic interlock surveillance requirement.

A notice will be published in the Federal Register at a later date.

Sincerely,

ORIGINAL SIGNED BY:

John B. Hickman, Project Manger
Project Directorate III-3
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Docket Nos. 50-315 and 316

Enclosure: Public Notice

cc w/encl: See next page

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NAME	JHickman	<i>APH</i>	CBoyTe	<i>APH</i>	<i>APH</i>	Joe Gilliland		
DATE	09/26/97		09/26/97		09/26/97	09/26/97		

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NAME	JHickman	<i>CPH</i>	CBoyle	<i>CPB</i>	<i>APH</i>	Joe Gililand		
DATE	09/26/97		09/26/97		09/26/97	09/26/97		

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E. E. Fitzpatrick
Indiana Michigan Power Company

Donald C. Cook Nuclear Plant
Unit Nos. 1 and 2

cc:

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PUBLIC NOTICE
NRC STAFF PROPOSES TO AMEND OPERATING LICENSES FOR
DONALD C. COOK NUCLEAR PLANT, UNITS 1 AND 2

The U.S. Nuclear Regulatory Commission (NRC) has received an application dated September 19, 1997, from Indiana Michigan Power Company (the licensee) for an amendment to Facility Operating License Nos. DPR-58 and DPR-74 for the Donald C. Cook Nuclear Plant, Units 1 and 2, respectively, located in Berrien County, Michigan.

The proposed amendments would delete the interlock which would close the Residual Heat Removal (RHR) suction valves if the Reactor Coolant System (RCS) pressure were to increase to 600 psig while retaining the interlock which would prevent the suction valves from opening while the RCS pressure is above the RHR system design pressure. This change would maintain the interlock against opening to protect against an intersystem loss of coolant accident but would allow continued deactivation of the isolation valves when the RHR system is operating to assure RHR availability and provide Low Temperature Overpressure Protection (LTOP).

The September 19, 1997, application requested that these amendments be processed on an emergency basis because these license amendments are required to allow the Donald C. Cook Nuclear Plant, Units 1 and 2, to resume power operation. The licensee was unable to make a more timely application because it was not determined until recently that the licensee's actions to assure RHR availability and provide LTOP was inconsistent with one of the technical specifications. The NRC has determined that the licensee used its best efforts to make a timely application for the proposed changes and that exigent circumstances do exist and were not the result of any intentional delay on the part of the licensee. The Donald C. Cook Nuclear Plant, Units 1 and 2, cannot restart until the proposed amendments have been approved by the NRC.

Following an initial review of this application against the standards in 10 CFR 50.92, the NRC staff has made a proposed (preliminary) determination that the amendment requests involve no significant hazards consideration. According to 10 CFR 50.92(c), this means that the proposed amendments would not involve a significant increase in the probability or consequences of an accident previously evaluated, would not create the possibility of a new or different kind of accident from any accident previously evaluated, or involve a significant reduction in a margin of safety. The licensee's analysis of the no significant hazards consideration is presented below:

Criterion 1

This amendment request does not involve a significant increase in the probability or consequences of an accident previously evaluated. The change provides an alternative means of providing overpressurization protection for the RHR system, and thereby protection against potential intersystem LOCA. Operating procedure administrative requirements establish the necessary LTOP system configuration and ECCS equipment operability constraints for Mode 4 operation. The LTOP system has been analyzed to show that, if operated per the existing operating procedure constraints, it will protect the RHR system during postulated overpressure conditions.

Criterion 2

The proposed change does not create the possibility of a new or different kind of accident from any accident previously evaluated. The change involves a different response by the system to an overpressurization event, but we have shown by analysis that the alternative LTOP configuration is capable of

providing equivalent protection to the original suction valve auto-closure feature. The system remains protected from single failure of any of the available overpressure protection components. The change eliminates the potential for a single power supply or instrument failure isolating and damaging the RHR system while operating to remove decay heat in Mode 4.

Criterion 3

This proposed change does not involve a significant reduction in a margin of safety. The change maintains an equivalent margin of safety against intersystem LOCA concerns. Operating with the suction valves blocked open and the overpressure protection of the LTOP system, the change also helps to ensure the availability of the decay heat removal from the RCS during any postulated accident which would involve pressurization of the RCS. Operating with the original auto-closure isolation of the suction valves would automatically cut off decay heat removal via the RHR system in any such postulated event if the RCS reached the auto-closure setpoint and the suction valves closed.

The change eliminated the potential for a power supply or instrument failure isolating and damaging the RHR system while in Mode 4. The requested change maintains protection from inadvertently opening the RHR suction valves, thereby exposing the RHR system to high RCS system pressure by maintaining the requirement for the open interlock in all modes.

If the proposed determination that the requested license amendments involve no significant hazards consideration becomes final, the NRC will issue the amendments without first offering an opportunity for a public hearing. An opportunity for a hearing will be

published in the Federal Register at a later date and any hearing request will not delay the effective date of the amendments.

If the NRC decides in its final determination that the amendments do involve a significant hazards consideration, a notice of opportunity for a prior hearing will be published in the Federal Register and, if a hearing is granted, it will be held before the amendments are issued.

Comments on the proposed determination of no significant hazards consideration may be submitted to Dr. Gail H. Marcus, Director, Project Directorate III-3, by collect call to 1-301-415-1389 or by facsimile to 1-301-415-3061. Written comments may be submitted by mail to the Chief, Rules and Directives Branch, Division of Administrative Services, Office of Administration, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001. Written comments may also be delivered to Room 6D22, Two White Flint North, 11545 Rockville Pike, Rockville, MD, from 7:30 a.m. to 4:15 p.m. on Federal workdays. Copies of written comments received may be examined at the NRC Public Document Room, located in the Gelman Building, 2120 L Street, NW., Washington, DC. All comments received by 4:15 p.m., Eastern Standard Time, on October 1, 1997, will be considered in reaching a final determination.

A copy of the application may be examined at the NRC's Local Public Document Room located in the Maud Preston Palenske Memorial Library, 500 Market Street, St. Joseph, MI 49085, and at the NRC Public Document Room.