

March 16, 1993

Docket Nos. 50-315  
and 50-316

Mr. E. E. Fitzpatrick, Vice President  
Indiana Michigan Power Company  
c/o American Electric Power Service Corporation  
1 Riverside Plaza  
Columbus, Ohio 43216

Dear Mr. Fitzpatrick:

SUBJECT: DONALD C. COOK NUCLEAR PLANT, UNIT NOS. 1 AND 2 - AMENDMENT NOS. 170  
AND 153 TO FACILITY OPERATING LICENSE NOS. DPR-58 AND DPR-74  
(TAC NOS. M79857 AND M79858)

The Commission has issued the enclosed Amendment No. 170 to Facility Operating License No. DPR-58 and Amendment No. 153 to Facility Operating License No. DPR-74 for the Donald C. Cook Nuclear Plant, Unit Nos. 1 and 2. The amendments consist of changes to the Technical Specifications (TS) in response to your application dated February 27, 1991.

The amendments revise Technical Specification (TS) 3.6.3.1, "Containment Isolation Valves" for both units. Specifically, a statement is added to TS 3.6.3.1 for each unit to the effect that the provisions of TS 3.0.4 do not apply under certain circumstances.

A copy of our related Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Original Signed By:

William M. Dean, Sr. Project Manager  
Project Directorate III-1  
Division of Reactor Projects III/IV/V  
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 170 to DPR-58
2. Amendment No. 153 to DPR-74
3. Safety Evaluation

cc w/enclosures:

See next page

w/changes

OFFICE	LA:PD31*	PM:PD31*	OGC*	D:PD31 <i>me</i>	SCSB*
NAME	MShuttleworth	WDean:	EHoller	LMarsh	RBarrett
DATE	2/25/93	1/13/92	3/2/93	3/10/92	2/03/93

OFFICIAL RECORD COPY

FILENAME: G:\wpdocs\dccook\co79857.amd

\*SEE PREVIOUS CONCURRENCE

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PDR ADOCK 05000315  
P PDR

*JFol*  
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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

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A copy of our related Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

A handwritten signature in black ink, appearing to read "William M. Dean, Sr.", written in a cursive style.

William M. Dean, Sr. Project Manager  
Project Directorate III-1  
Division of Reactor Projects III/IV/V  
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 170 to DPR-58
2. Amendment No. 153 to DPR-74
3. Safety Evaluation

cc w/enclosures:  
See next page

Mr. E. E. Fitzpatrick  
Indiana Michigan Power Company

Donald C. Cook Nuclear Plant

cc:

Regional Administrator, Region III  
U.S. Nuclear Regulatory Commission  
799 Roosevelt Road  
Glen Ellyn, Illinois 60137

Mr. S. Brewer  
American Electric Power Service  
Corporation  
1 Riverside Plaza  
Columbus, Ohio 43216

Attorney General  
Department of Attorney General  
525 West Ottawa Street  
Lansing, Michigan 48913

Township Supervisor  
Lake Township Hall  
Post Office Box 818  
Bridgman, Michigan 49106

Al Blind, Plant Manager  
Donald C. Cook Nuclear Plant  
Post Office Box 458  
Bridgman, Michigan 49106

U.S. Nuclear Regulatory Commission  
Resident Inspector Office  
7700 Red Arrow Highway  
Stevensville, Michigan 49127

Gerald Charnoff, Esquire  
Shaw, Pittman, Potts and Trowbridge  
2300 N Street, N. W.  
Washington, DC 20037

Mayor, City of Bridgman  
Post Office Box 366  
Bridgman, Michigan 49106

Special Assistant to the Governor  
Room 1 - State Capitol  
Lansing, Michigan 48909

Nuclear Facilities and Environmental  
Monitoring Section Office  
Division of Radiological Health  
Department of Public Health  
3423 N. Logan Street  
P. O. Box 30195  
Lansing, Michigan 48909

DATED: March 16, 1993

AMENDMENT NO. 170 TO FACILITY OPERATING LICENSE NO. DPR-58-D. C. COOK  
AMENDMENT NO. 153 TO FACILITY OPERATING LICENSE NO. DRP-74-D. C. COOK

Docket File  
NRC & Local PDRs  
PDIII-1 Reading  
D.C. Cook Plant File  
J. Roe  
J. Zwolinski  
L. Marsh  
M. Shuttleworth  
W. Dean  
OGC-WF  
D. Hagan, 3302 MNBB  
G. Hill (8), P-137  
Wanda Jones, MNBB-7103  
C. Grimes, 11/F/23  
ACRS (10)  
GPA/PA  
OC/LFMB  
W. Shafer, R-III

cc: Plant Service list

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

INDIANA MICHIGAN POWER COMPANY

DOCKET NO. 50-315

DONALD C. COOK NUCLEAR PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 170  
License No. DPR-58

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Indiana Michigan Power Company (the licensee) dated February 27, 1991, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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P PDR

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-58 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No.170 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Ledyard B. Marsh, Director  
Project Directorate III-1  
Division of Reactor Projects III/IV/V  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: March 16, 1993

ATTACHMENT TO LICENSE AMENDMENT NO. 170

TO FACILITY OPERATING LICENSE NO. DPR-58

DOCKET NO. 50-315

Revise Appendix A Technical Specifications by removing the page identified below and inserting the attached page. The revised page is identified by amendment number and contains vertical lines indicating the area of change.

REMOVE

3/4 6-14

INSERT

3/4 6-14

## CONTAINMENT SYSTEMS

### 3/4.6.3 CONTAINMENT ISOLATION VALVES

#### LIMITING CONDITION FOR OPERATION

3.6.3.1 The containment isolation valves specified in Table 3.6-1 shall be OPERABLE with isolation times as shown in Table 3.6-1. The ACTION statement of T/S 3/4.6.3 is not applicable to the containment purge supply and exhaust isolation valves, VCR-101 through 106 and VCR-201 through 206, listed in Table 3.6-1. The Limiting Condition for Operation and its associated ACTION statement for these valves is given in Technical Specification 3/4.6.1.7.

APPLICABILITY: MODES 1, 2, 3 and 4.

#### ACTION:

With one or more of the isolation valve(s) specified in Table 3.6-1 inoperable, either:

- a. Restore the inoperable valve(s) to OPERABLE status within 4 hours, or
- b. Isolate each affected penetration within 4 hours by use of at least one deactivated automatic valve secured in the isolation position, or
- c. Isolate each affected penetration within 4 hours by use of at least one closed manual valve or blind flange, or
- d. Be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

The provisions of Specification 3.0.4 are not applicable.

#### SURVEILLANCE REQUIREMENTS

4.6.3.1.1 The isolation valves specified in Table 3.6-1 shall be demonstrated OPERABLE prior to returning the valve to service after maintenance, repair or replacement work is performed on the valve or its associated actuator, control or power circuit by performance of a cycling test and verification of isolation time.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

INDIANA MICHIGAN POWER COMPANY

DOCKET NO. 50-316

DONALD C. COOK NUCLEAR PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 153  
License No. DPR-74

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Indiana Michigan Power Company (the licensee) dated February 21, 1991, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-74 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 153, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Ledyard B. Marsh, Director  
Project Directorate III-1  
Division of Reactor Projects III/IV/V  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: March 16, 1993

ATTACHMENT TO LICENSE AMENDMENT NO. 153

FACILITY OPERATING LICENSE NO. DPR-74

DOCKET NO. 50-316

Revise Appendix A Technical Specifications by removing the page identified below and inserting the attached page. The revised page is identified by amendment number and contains vertical lines indicating the area of change.

REMOVE

3/4 6-13

INSERT

3/4 6-13

## CONTAINMENT SYSTEMS

### 3/4.6.3 CONTAINMENT ISOLATION VALVES

#### LIMITING CONDITION FOR OPERATION

3.6.3.1 The containment isolation valves specified in Table 3.6-1 shall be OPERABLE with isolation times as shown in Table 3.6-1. The ACTION statement of Technical Specification 3/4.6.3 is not applicable to the containment purge and exhaust isolation valves, VCR-101 through 106 and VCR-201 through 206, listed in Table 3.6-1. The Limiting Condition for Operation and its associated ACTION statement for these valves is given in Technical Specification 3/4.6.1.7.

APPLICABILITY: MODES 1, 2, 3 and 4.

#### ACTION:

With one or more of the isolation valve(s) specified in Table 3.6-1 inoperable, maintain at least one isolation valve OPERABLE in each affected penetration that is open and either:

- a. Restore the inoperable valve(s) to OPERABLE status within 4 hours, or
- b. Isolate each affected penetration within 4 hours by use of at least one deactivated automatic valve secured in the isolation position, or
- c. Isolate each affected penetration within 4 hours by use of at least one closed manual valve or blind flange, or
- d. Be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

The provisions of Specification 3.0.4 are not applicable.

#### SURVEILLANCE REQUIREMENTS

4.6.3.1.1 The isolation valves specified in Table 3.6-1 shall be demonstrated OPERABLE prior to returning the valve to service after maintenance, repair or replacement work is performed on the valve or its associated actuator, control or power circuit by performance of a cycling test and verification of isolation time.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 170 TO FACILITY OPERATING LICENSE NO. DPR-58  
AND AMENDMENT NO. 153 TO FACILITY OPERATING LICENSE NO. DPR-74

INDIANA MICHIGAN POWER COMPANY

DONALD C. COOK NUCLEAR PLANT, UNIT NOS. 1 AND 2

DOCKET NOS. 50-315 AND 50-316

1.0 INTRODUCTION

By letter dated February 21, 1991, the Indiana Michigan Power Company (the licensee) requested amendments to the Technical Specifications (TS) appended to Facility Operating License Nos. DPR-58 and DPR-74 for the Donald C. Cook Nuclear Plant, Unit Nos. 1 and 2. The proposed amendments would add a statement to TS 3.6.3.1 that would make TS 3.0.4 not applicable, thus allowing mode changes with a containment isolation valve inoperable. The requested change is in accordance with Generic Letter (GL) 91-08, "Removal of Component Lists from the Technical Specifications," by adding the following statement to the Limiting Condition for Operation (LCO) 3.6.3.1: "The provisions of specification 3.0.4 are not applicable." This exception would apply to all containment isolation valves listed in Table 3.6-1 associated with TS 3.6.3.1.

Enclosure 1 to GL 87-09, "Alternatives to the STS Requirements to Resolve Three Specific Problems with Limiting Conditions for Operation and Surveillance Requirements," states that although TS 3.0.4 may unduly restrict facility operation, nothing in the staff position should be interpreted as endorsing or encouraging a plant start-up with inoperable equipment. However, even with the staff guidance and recommendations in GL 87-09, the staff believes there is still room for misinterpretation of the intent of an exception to TS 3.0.4. Therefore, included in this Safety Evaluation is a discussion of the staff's intent and expectations in granting a TS 3.0.4 exception.

2.0 EVALUATION

The list of containment isolation valves generally found within TS typically include footnotes that modify the TS requirements for these valves. Generic Letter 91-08 provides guidance on removing the list of valves from the TS, if the licensee opts to do so. Part of this guidance is that these footnotes must be incorporated into the LCO so that they will remain in effect when the table listing the valves is removed from the TS. By incorporating this

guidance into the LCO, the licensee could delete the list of valves from TS 3.6.3.1 while still keeping the appropriate guidance in effect.

Generic Letter 87-09 noted that there has been an inconsistent application of TS 3.0.4 in that it would "...preclude entering a mode or specified condition if an LCO is not met, even if the Action Requirements would permit continued operation of the facility for an unlimited period of time. Generally, the individual specifications that have Action Requirements which allow continued operation note that Specification 3.0.4 does not apply. However, exceptions to Specification 3.0.4 have not been consistently applied and their bases are not well documented....Although the staff encourages the maintenance of all plant systems and components in an operable condition as a good practice, the T/Ss generally have not precluded entering a mode with inoperable equipment when the Action Requirements include remedial measures that provide an acceptable level of safety for continued operation." The action requirements for containment isolation valves (CIVs) permit continued operation with an inoperable valve when the associated penetration is isolated.

The licensee has administrative controls in place to limit the use of TS 3.0.4 exceptions. This includes PMI-4030, "Technical Specifications Review and Surveillance," which requires review and approval of a mode change under the TS 3.0.4 exception by the operations department superintendent and the plant manager. Additionally, the licensee's prioritization scheme for corrective maintenance classifies job orders concerning inoperable Technical Specification-related equipment as generally having the highest priority to be worked.

The staff has reviewed the licensee's submittal and has found the proposed change to be acceptable. The exception from requirements of TS 3.0.4 is applicable to all CIVs. This change is acceptable because it is consistent with the guidance provided in GL 91-08, as long as the intent and expectations regarding the exception from TS 3.0.4 (discussed below) are met.

The guidance provided in GL 87-09 and GL 91-08 should not be interpreted to allow CIVs to remain in a degraded condition indefinitely. Containment isolation valves serve a significant safety function in that they establish containment integrity. Therefore, it is urgent that these valves be returned to an operable status at the earliest possible opportunity.

In general, the staff believes that nuclear plants should start-up from an outage with all CIVs operable. But because of extenuating circumstances, such as unavailability of equipment needed for repair or replacement, or the discovery of inoperable CIVs just prior to start-up, there will be isolated instances where allowances will be made to support a start-up with inoperable CIVs. However, these isolated instances should coexist with the understanding that the inoperable CIV(s) be repaired or replaced at the earliest possible opportunity in accordance with 10 CFR Part 50, Appendix B. The licensee is also expected to keep the NRC staff abreast of such instances.

Moreover, in approving the above licensee-proposed change to TS 3.6.3.1, the staff holds the following position, which is contained in TS 4.0.4:

Entry into an operational mode or other specified condition shall not be made unless the Surveillance Requirement(s) associated with the Limiting Condition for Operation has been performed within the stated surveillance interval, or as otherwise specified.

### 3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Michigan State official was notified of the proposed issuance of the amendments. The State official had no comments.

### 4.0 ENVIRONMENTAL CONSIDERATION

The amendments change the requirements with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration and there has been no public comment on such finding (56 FR 22470). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

### 5.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors: T. Dunning  
A. Young  
W. Dean

Date: March 16, 1993