

June 27, 1990

Docket Nos. 50-315 and 50-316

Mr. Milton P. Alexich, Vice President Indiana Michigan Power Company c/o American Electric Power Service Corporation 1 Riverside Plaza Columbus, Ohio 43216

Dear Mr. Alexich:

SUBJECT: AMENDMENT NOS. 139 AND 126 TO FACILITY OPERATING LICENSE NOS. DPR-58 AND DPR-74: (TAC NOS. 73276 AND 73277)

The Commission has issued the enclosed Amendment No. 139 to Facility Operating License No. DPR-58 and Amendment No. 126 to Facility Operating License No. DPR-74 for the Donald C. Cook Nuclear Plant, Units Nos. 1 and 2. The amendments consist of changes to the Technical Specifications in response to your application dated March 29, 1989.

These amendments change Technical Specification (TS) Table 3.3-2, "Reactor Trip System Instrumentation Response Times," to explicitly require time response testing of the lower set point power range neutron flux (PRNF) reactor trip.

A copy of our related Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly <u>Federal Register</u> notice.

Sincerely,

Joseph Yutter

Voseph Giitter, Project Manager Project Directorate III-1 Division of Reactor Projects - III, IV, V & Special Projects Office of Nuclear Reactor Regulation

Enclosures:

- 1. Amendment No. 139 to DPR-58
- 2. Amendment No.126 to DPR-74
- 3. Safety Evaluation

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cc w/enclosures: See next page cc:

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Donald C. Cook Nuclear Plant

Mr. S. Brewer American Electric Power Service Corporation 1 Riverside Plaza Columbus, Ohio 43216

Docket No. 50-316

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Mr. Milton P. Alexich, Vice President Indiana Michigan Power Company c/o American Electric Power Service Corporation 1 Riverside Plaza Columbus, Ohio 43216

Dear Mr. Alexich:

SUBJECT: AMENDMENT NO. TO FACILITY OPERATING LICENSE NO. DPR-74: (TAC NOS. 73276 AND 73277)

The Commission has issued the enclosed Amendment No. to Facility Operating License No. DPR-74 for the Donald C. Cook Nuclear Plant, Unit No. 2. The amendment consist of changes to the Technical Specifications in response to your application dated March 29, 1989.

This amendment changes Technical Specification (TS) Table 3.3-2, "Reactor Trip System Instrumentation Response Times," to explicitly require time response testing of the lower set point power range neutron flux (PRNF) reactor trip.

A copy of our related Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Joseph Giitter, Project Manager Project Directorate III-1 Division of Reactor Projects - III, IV, V & Special Projects Office of Nuclear Reactor Regulation

Enclosures: 1. Amendment No. to DPR-74 2. Safety Evaluation

cc w/enclosures: See next page

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INDIANA MICHIGAN POWER COMPANY

DOCKET NO. 50-315

DONALD C. COOK NUCLEAR PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No.139 License No. DPR-58

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Indiana Michigan Power Company (the licensee) dated March 29, 1989, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

9007230228 900627 PDR ADOCK 05000315 PDC PDC 2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-74 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 139, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

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Robert C. Pierson, Acting Director Project Directorate III-1 Division of Reactor Projects - III, IV, V & Special Projects Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance: June 27, 1990

ATTACHMENT TO LICENSE AMENDMENT NO. 139

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FACILITY OPERATING LICENSE NO. DPR-58

DOCKET NO. 50-315

Revise Appendix A Technical Specifications by removing the page identified below and inserting the attached page. The revised page is identified by amendment number and contains marginal lines indicating the area of change.

REMOVE INSERT 3/4 3-10 3/4 3-10

TABLE 3.3-2

REACTOR TRIP SYSTEM INSTRUMENTATION RESPONSE TIMES

FUNCTIONAL UNIT

1. Manual Reactor Trip

2. Power Range, Neutron Flux

(High and Low Setpoint)

3. Power Range, Neutron Flux,

4. Power Range, Neutron Flux,

6. Source Range, Neutron Flux

9. Pressurizer Pressure--Low

10. Pressurizer Pressure--High

11. Pressurizer Water Level--High

7. Overtemperature delta T

8. Overpower delta T

5. Intermediate Range, Neutron Flux

High Positive Rate

High Negative Rate

RESPONSE TIME

NOT APPLICABLE

Less than or equal to 0.5 seconds

NOT APPLICABLE

Less than or equal to 0.5 seconds*

NOT APPLICABLE

NOT APPLICABLE

Less than or equal to 6.0 seconds

NOT APPLICABLE

Less than or equal to 1.0 seconds

Less than or equal to 1.0 seconds

NOT APPLICABLE

COOK NUCLEAR PLANT - UNIT 1

Neutron detectors are exempt from response time testing. Response time of the neutron flux signal portion of the channel shall be measured from detector output or input of first electronic component in channel.



INDIANA MICHIGAN POWER COMPANY

DOCKET NO. 50-316

DONALD C. COOK NUCLEAR PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 126 License No. DPR-74

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Indiana Michigan Power Company (the licensee) dated March 29, 1989, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

 Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-74 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 126, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

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Robert C. Pierson, Acting Director Project Directorate III-1 Division of Reactor Projects - III, IV, V & Special Projects Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance: June 27, 1990

ATTACHMENT TO LICENSE AMENDMENT NO. 126

FACILITY OPERATING LICENSE NO. DPR-74

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DOCKET NO. 50-316

Revise Appendix A Technical Specifications by removing the page identified below and inserting the attached page. The revised page is identified by amendment number and contains marginal lines indicating the area of change.

REMOVE INSERT

3/4 3-9 3/4 3-9

TABLE 3.3-2

REACTOR TRIP SYSTEM INSTRUMENTATION RESPONSE TIMES

FUNCTIONAL UNIT

1. Manual Reactor Trip

2. Power Range, Neutron Flux

(High and Low Setpoint)

3. Power Range, Neutron Flux,

4. Power Range, Neutron Flux,

High Positive Rate

High Negative Rate

RESPONSE TIME

NOT APPLICABLE

Less than or equal to 0.5 seconds

NOT APPLICABLE

Less than or equal to 0.5 seconds*

NOT APPLICABLE

NOT APPLICABLE

Less than or equal to 6.0 seconds

NOT APPLICABLE

Less than or equal to 1.0 seconds

Less than or equal to 1.0 seconds

NOT APPLICABLE

5. Intermediate Range, Neutron Flux 6. Source Range, Neutron Flux 7. Overtemperature delta T 8. Overpower delta T 9. Pressurizer Pressure--Low

10. Pressurizer Pressure -- High

11. Pressurizer Water Level--High

COOK NUCLEAR PLANT - UNIT 2

Neutron detectors are exempt from response time testing. Response time of the neutron flux signal portion of the channel shall be measured from detector output or input of first electronic component in channel.



SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION RELATED TO AMENDMENT NO. 139 AND AMENDMENT NO. 126 TO FACILITY OPERATING LICENSE NOS. DPR-58 AND DPR-74 INDIANA MICHIGAN POWER COMPANY

DONALD C. COOK NUCLEAR PLANT, UNIT NOS. 1 AND 2

DOCKET NOS. 50-315 AND 50-316

1.0 INTRODUCTION

By letter dated March 29, 1989, Indiana Michigan Power Company (the licensee) requested approval of an amendment to the D. C. Cook, Unit No. 2 plant Technical Specifications (TS). The proposed change includes clarification of testing requirements for the power range neutron flux (PRNF) reactor trip set points. In the request, the licensee indicated that lack of sufficient detail in the TS testing requirements for the PRNF reactor trip set points had prevented the response time testing of the low power set point trip function.

2.0 DISCUSSION

The Technical Specification amendment changes TS Table 3.3-2, "Reactor Trip System Instrumentation Response Times," to explicitly require that response time for the "Power Range, Neutron Flux (High and Low Set Point)" trips be less than or equal to 0.5 seconds. This wording is more explicit than that found in the Standard Technical Specifications (STS), which use the term "Power Range, Neutron Flux" when referring to required response times of the PRNF trip functions.

3.0 EVALUATION

The effect of the proposed TS change is to make the licensee's Technical Specification requirements concerning the high and low set point PRNF reactor trips more explicit than those found in the STS. Clarification of these response time requirements serves to make the Technical Specifications more consistent with the licensee's safety analyses. Credit is taken in the safety analyses of both units for both the high and low set point PRNF reactor trip functions in the mitigation of accident sequences. In the past, misunderstanding of the TS requirements for the PRNF function had prevented the time response testing of the low set point PRNF reactor trip function.

Addition of the TS clarification is primarily an administrative change, although it now explicitly requires testing of both the high and low set point PRNF trip functions. The addition of explicit wording regarding response time requirements of the PRNF trip ensures that both the high and low set point trip functions are response time tested. Therefore, this change is conservative.

Based on the above evaluation, the staff concludes that the licensee's proposed Technical Specification amendment, as delineated in the March 29, 1989 letter, is acceptable.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment involves a change in a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and a change in a surveillance requirement. We have determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that these amendments involve no significant hazards consideration and there has been no public comment on such finding. Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendment.

5.0 CONCLUSION

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We have concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Date: June 27, 1990

Principal Contributor: Robert J. Stransky, Jr., PD31