

May 8, 2002

Mr. Michael Kansler
Sr. Vice President and Chief
Operating Officer
Entergy Nuclear Operations, Inc.
440 Hamilton Avenue
White Plains, NY 10601

SUBJECT: JAMES A. FITZPATRICK NUCLEAR POWER PLANT - AMENDMENT
REGARDING ANTICIPATED TRANSIENT WITHOUT SCRAM (ATWS)
RECIRCULATION PUMP TRIP INSTRUMENTATION SETPOINT (TAC
NO. MB3153)

Dear Mr. Kansler:

The Commission has issued the enclosed Amendment No. 273 to Facility Operating License No. DPR-59 for the James A. FitzPatrick Nuclear Power Plant. The amendment consists of changes to the Technical Specifications (TSs) in response to your application transmitted by letter dated September 28, 2001.

The amendment revises the ATWS Recirculation Pump Trip Reactor Pressure High setpoint by replacing the current conditional setpoints, which are based upon the number of Safety Relief Valves out of service, with a single setpoint.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular biweekly *Federal Register* notice.

Sincerely,

/RA/

Guy S. Vissing, Sr. Project Manager, Section 1
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-333

Enclosures: 1. Amendment No. 273 to DPR-59
2. Safety Evaluation

cc w/encls: See next page

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cc w/encls: See next page

Package: ML021330077
Accession Number: ML021050296

TSs: ML021290213
*Safety Evaluation Provided on 2/20/02 - No major
changes

OFFICE	PDI-1\PM	PDI-1\LA	EEIB/SC*	SRXB/BC*	OGC	PDI-1\SC
NAME	GVissing	SLittle	EMarinos	JWeirmiel	RHoefling	PTam for RLauffer
DATE	5/1/02	5/1/02	2/20/02	2/20/02	4/24/02	5/8/02

OFFICIAL RECORD COPY

FitzPatrick Nuclear Power Plant

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DATED: May 8, 2002

AMENDMENT NO. 273 TO FACILITY OPERATING LICENSE NO. DPR-59 FITZPATRICK

PUBLIC
PDI-1 R/F
RLaifer
OGC
GHill (2)
WBeckner
ACRS
BPlatchek, RI
SRichards
GVissing
SLittle
SMazumdar

cc: Plant Service list

ENTERGY NUCLEAR OPERATIONS, INC.

DOCKET NO. 50-333

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 273
License No. DPR-59

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Entergy Nuclear Operations, Inc. (the licensee) dated September 28, 2001, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-59 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 273, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA by PTam for RLaufer/

Richard J. Laufer, Chief, Section 1
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: May 8, 2002

ATTACHMENT TO LICENSE AMENDMENT NO. 273

FACILITY OPERATING LICENSE NO. DPR-59

DOCKET NO. 50-333

Replace the following pages of the Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove Pages

76a
76c

Insert Pages

76a
76c

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 273 TO FACILITY OPERATING LICENSE NO. DPR-59

ENTERGY NUCLEAR OPERATIONS, INC.

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

DOCKET NO. 50-333

1.0 INTRODUCTION

By letter dated September 28, 2001, Entergy Nuclear Operations, Inc. (the licensee) submitted a request for changes to the James A. FitzPatrick Nuclear Power Plant Technical Specifications (TSs). The requested changes would revise the Anticipated Transient Without Scram (ATWS) Recirculation Pump Trip Reactor Pressure High setpoint by replacing the current conditional setpoints which are based upon the number of Safety Relief Valves (SRVs) out of service, with a single setpoint.

The licensee proposed the following changes to TS Table 3.2-7, ATWS Recirculation Pump Trip Instrumentation Requirements, Reactor Pressure - High, Trip Level Setting, associated with the number of SRVs in service:

- a. Change the setting to a single value of " ≤ 1155 psig" from the current setting of " ≤ 1120 or ≤ 1155 psig (Note 3)."
- b. Delete Note 3 which states "The ATWS Reactor Pressure High Recirculation Pump Trip setpoint shall be ≤ 1155 psig when either zero or one SRVs are out of service. The setpoint shall be ≤ 1120 psig when two or more SRVs are out of service."

2.0 EVALUATION

The current TS specifies two trip setpoints for the ATWS Recirculation Pump Trip, Reactor Pressure High, Instrumentation:

≤ 1120 psig for 1 or fewer SRVs out of service
and
 ≤ 1155 psig with 2 or more SRVs out of service.

The TS allows a maximum of two SRVs out of service during power operation and based on this criterion the licensee initiated an analysis of the ATWS Reactor Pressure High Recirculation Pump Trip setpoint, the results of which are reported in NEDC-32616P, "James A. FitzPatrick Nuclear Power Plant Anticipated Transient Without Scram (ATWS) Analysis for Recirculation Pump Trip Setpoint Changes," dated July 18, 1996 (Reference 2). The NRC approved this report by letter dated November 17, 1998.

The licensee stated that this analysis was performed using General Electric proprietary ODYN code described in GE Topical Report NEDO-24154. The Nuclear Regulatory Commission (NRC) staff reviewed ODYN and found it acceptable as documented in "Safety Evaluation for General Electric Topical Report Qualification of the One-Dimensional Core Transient Model for Boiling Water Reactors, NEDO-24154 and NEDO-24154-P Volumes I, II, III, June 1980." The licensee further stated that the GE Analysis of March 2000 (Reference 1) demonstrated that the high reactor pressure vessel (RPV) pressure setpoint should be \leq 1155 psig with two or fewer SRVs out of service which is well below the worst case RPV pressure of 1401 psig and the allowable RPV pressure of 1500 psig.

By letter dated October 12, 2001, the NRC approved the instrument setpoint methodology used by the licensee in Improved TSs Conversion Beyond Scope issues. Because the proposed changes are based on the NRC-approved calculation methodologies the NRC staff finds the proposed changes acceptable.

The proposed TS changes will remove inconsistencies in the current TSs. The current TSs prohibits plant operation at power with more than two SRVs out of service while Table 3.2-7 specifies ATWS Reactor Pressure High Recirculation Pump Trip setpoints with two or more SRVs out of service. The proposed changes have been established by the NRC-approved calculation methodologies and the NRC staff finds the proposed TS changes acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New York State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (66 FR 64294). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

6.0 REFERENCES

- (1) "ATWS Overpressure Analysis for FitzPatrick," GE-NE-A42-00137-2-01, March 2000.
- (2) "James A. FitzPatrick Nuclear Power Plant Anticipated Transient Without Scram (ATWS) Analysis for Recirculation Pump Trip Setpoint Changes," NEDC-32616P, July 18, 1996.
- (3) "FitzPatrick Power Uprate Impact Study, Engineering Report: Section 9.3.1, Anticipated Transient Without Scram Analyses for the James A. FitzPatrick Nuclear Power Plant," GE-NE-187-59-1191, November 1991.

Principal Contributor: Subinoy Mazumdar

Date: May 8, 2002