

December 6, 1988

Docket Nos. 50-315
and 50-316

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Mr. Milton P. Alexich, Vice President
Indiana Michigan Power Company
c/o American Electric Power Service Corporation
1 Riverside Plaza
Columbus, Ohio 43216

Dear Mr. Alexich:

SUBJECT: AMENDMENTS NOS. 119 AND 105 TO FACILITY OPERATING LICENSES NOS. DPR-58 AND DPR-74: CHANGES TO THE RADIOLOGICAL ENVIRONMENTAL TECHNICAL SPECIFICATIONS (TACS NOS. 67790/67791)

The Commission has issued the enclosed Amendment No. 119 to Facility Operating License No. DPR-58 and Amendment No. 105 to Facility Operating License No. DPR-74 for the Donald C. Cook Nuclear Plant, Units Nos. 1 and 2. The amendments consist of changes to the Technical Specifications (TSs) in response to your application dated February 1, 1988.

These amendments revise the TSs to make them more consistent with NRC guidelines concerning obtaining milk samples for analysis. In addition, the TS bases change to be more consistent with the Westinghouse Standard TSs with regard to the thyroid dose release pathway for a child, and an editorial error is corrected by removing redundant \leq signs.

Copies of our related Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

Original signed by

Wayne Scott, Project Manager
Project Directorate III-1
Division of Reactor Projects - III, IV, V
& Special Projects

Enclosures:

1. Amendment No. 119 to DPR-58
2. Amendment No. 105 to DPR-74
3. Safety Evaluation
4. Notice

cc w/enclosures:
See next page

* See attached concurrence.

LA/PD31:DRSP
RIngram
10/31/88

PM/PD31:DRSP
JStang
10/4/88

YRQ
(A)D/PD31:DRSP
TQUAY
11/29/88

OGC *
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PDC

Docket Nos. 50-315
and 50-316

Mr. Milton P. Alexich, Vice President
Indiana Michigan Power Company
c/o American Electric Power Service Corporation
1 Riverside Plaza
Columbus, Ohio 43216

Dear Mr. Alexich:

SUBJECT: AMENDMENTS NOS. AND TO FACILITY OPERATING LICENSES NOS.
DPR-58 AND DPR-74: CHANGES TO THE RADIOLOGICAL ENVIRONMENTAL
TECHNICAL SPECIFICATIONS (TACS NOS. 67790/67791)

The Commission has issued the enclosed Amendment No. to Facility
Operating License No. DPR-58 and Amendment No. to Facility Operating
License No. DPR-74 for the Donald C. Cook Nuclear Plant, Units Nos. 1 and 2.
The amendments consist of changes to the Technical Specifications (TSs) in
response to your application dated February 1, 1988.

These amendments revise the TSs to make them more consistent with NRC guide-
lines concerning obtaining milk samples for analysis. In addition, the TS
bases change to be more consistent with the Westinghouse Standard TSs with
regard to the thyroid dose release pathway for a child, and an editorial error
is corrected by removing redundant \leq signs.

Copies of our related Safety Evaluation and the Notice of Issuance are also
enclosed.

Sincerely,

John F. Stang, Project Manager
Project Directorate III-1
Division of Reactor Projects - III, IV, V
& Special Projects

Enclosures:

1. Amendment No. to DPR-58
2. Amendment No. to DPR-74
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cc w/enclosures:
See next page

LA/PD31:DRSP
RIngram
10/3/88

PM/PD31:DRSP
JStang
10/17/88

(A)D/PD31:DRSP
Wz Scott
10/17/88

OGC
Barnhart
10/12/88

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UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D. C. 20555
December 6, 1988

Dockets Nos. 50-315
and 50-316

Mr. Milton P. Alexich, Vice President
Indiana Michigan Power Company
c/o American Electric Power Service Corporation
1 Riverside Plaza
Columbus, Ohio 43216

Dear Mr. Alexich:

SUBJECT: AMENDMENTS NOS. 119 AND 105 TO FACILITY OPERATING LICENSES NOS.
DPR-58 AND DPR-74: CHANGES TO THE RADIOLOGICAL ENVIRONMENTAL
TECHNICAL SPECIFICATIONS (TACS NOS. 67790/67791)

The Commission has issued the enclosed Amendment No. 119 to Facility Operating License No. DPR-58 and Amendment No. 105 to Facility Operating License No. DPR-74 for the Donald C. Cook Nuclear Plant, Units Nos. 1 and 2. The amendments consist of changes to the Technical Specifications (TSs) in response to your application dated February 1, 1988.

These amendments revise the TSs to make them more consistent with NRC guidelines concerning obtaining milk samples for analysis. In addition, the TS bases change to be more consistent with the Westinghouse Standard TSs with regard to the thyroid dose release pathway for a child, and an editorial error is corrected by removing redundant \leq signs.

Copies of our related Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

Wayne Scott, Project Manager
Project Directorate III-1
Division of Reactor Projects - III, IV, V
& Special Projects

Enclosures:

1. Amendment No. 119 to DPR-58
2. Amendment No. 105 to DPR-74
3. Safety Evaluation
4. Notice

cc w/enclosures:
See next page

Mr. Milton Alexich
Indiana Michigan Power Company

Donald C. Cook Nuclear Plant

cc:
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U.S. Nuclear Regulatory Commission
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Township Supervisor
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Bridgeman, Michigan 49106

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Donald C. Cook Nuclear Plant
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Bridgman, Michigan 49106

U.S. Nuclear Regulatory Commission
Resident Inspectors Office
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Stevensville, Michigan 49127

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Shaw, Pittman, Potts and Trowbridge
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Washington, DC 20037

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Bridgeman, Michigan 49106

Special Assistant to the Governor
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Lansing, Michigan 48909

Nuclear Facilities and Environmental
Monitoring Section Office
Division of Radiological Health
Department of Public Health
3500 N. Logan Street
Post Office Box 30035
Lansing, Michigan 48909



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

INDIANA MICHIGAN POWER COMPANY

DOCKET NO. 50-315

DONALD C. COOK NUCLEAR PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 119
License No. DPR-58

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Indiana Michigan Power Company (the licensee) dated February 1, 1988, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;
and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-58 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 119, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Theodore R. Quay

Theodore Quay, Acting Director
Project Directorate III-1
Division of Reactor Projects - III, IV, V
& Special Projects

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 6, 1988



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

INDIANA MICHIGAN POWER COMPANY

DOCKET NO. 50-316

DONALD C. COOK NUCLEAR PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 105
License No. DPR-74

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Indiana Michigan Power Company (the licensee) dated February 1, 1988, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;
and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-74 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 105, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Theodore Quay, Acting Director
Project Directorate III-1
Division of Reactor Projects - III, IV, V
& Special Projects

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 6, 1988

ATTACHMENT TO LICENSE AMENDMENTS

AMENDMENT NO. 119 FACILITY OPERATING LICENSE NO. DPR-58

AMENDMENT NO. 105 FACILITY OPERATING LICENSE NO. DPR-74

DOCKETS NOS. 50-315 AND 50-316

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the attached pages. The revised pages are identified by amendment number and contain marginal lines indicating the area of change.

REMOVE

UNIT 1

3/4 12-4

B 3/4 11-12

UNIT 2

3/4 12-4

B 3/4 11-2

INSERT

3/4 12-4
3/4 12-4a

B 3/4 11-2

3/4 12-4
3/4 12-4a

B 3/4 11-2

TABLE 3.12-1 (Cont)

d. Sediment from Shoreline	L2, L3	2/year	Gamma Isotopic Analyses Semi-Annually.
4. Ingestion			
a. Milk	Each indicator farm and a background farm*	At least once per 15 days when animals are on Pasture. At Least Once Per 31 Days at Other Times.	Gamma Isotopic and I-131 Analysis of Each Sample.
b. Fish	Plant Site Off-Site	2/year	Gamma Isotopic Analysis on Edible Portion.
c. Food Products	Plant Site Off-Site (approx. 20 mi)	At time of Harvest One Sample of Each of the Following Classes of Food Products: 1. Grapes	Gamma Isotopic Analysis on Edible Portion.

^a Particulate sample filters should be analyzed for gross beta 24 hours or more after sampling to allow for radon and thoron daughter decay. If gross beta activity in air or water is greater than 10 times the yearly mean of control samples for any medium, gamma isotopic analysis should be performed on the individual samples.

* An indicator farm is defined as the nearest milk producer in each of the land sectors within 8 miles of the plant site who is willing to participate in the radiological environmental monitoring program. A background farm is defined as a milk producer in one of the less prevalent wind directions at a distance greater than 15 miles but less than 25 miles who is willing to participate in the radiological environmental monitoring program. If at least three indicator milk samples and one background milk sample cannot be obtained, vegetation sampling will be performed as a replacement for the milk sampling and no milk samples will be required.

TABLE 3.12-1 (Cont)

Plant Site	At time of Harvest One sample of Broad Leaf Vegetation	Gamma Isotopic Analysis.
3 indicator samples of broad leaf vegetation grown nearest to the offsite locations of highest calculated annual average ground level D/Q if at least three indicator milk samples and one background milk sample cannot be obtained.	Monthly when available	Gamma Isotopic and I-131 monthly when available
1 background sample of each of the similar vegetation grown 15-25 miles distant and in one of the less prevalent wind directions if at least three indicator milk samples and one background milk sample cannot be obtained.	Monthly when available	Gamma Isotopic and I-131 monthly when available

RADIOACTIVE EFFLUENTS

BASES

3/4.11.1.3 LIQUID WASTE TREATMENT. The OPERABILITY of the liquid radwaste treatment system ensures that this system will be available for use whenever liquid effluents require treatment prior to release to the environment. The requirements that the appropriate portions of this system be used when specified provide assurance that the releases of radioactive materials in liquid effluents will be kept "as low as is reasonably achievable." This specification implements the requirements of 10 CFR Part 50.36a, General Design Criteria Section 11.1 of the Final Safety Analysis Report for the Donald C. Cook Nuclear Plant, and design objective Section II.D of Appendix I to 10 CFR Part 50. The specified limits governing the use of appropriate portions of the liquid radwaste treatment system were specified as a suitable fraction of the dose design objectives set forth in Section II.A of Appendix I, 10 CFR Part 50, for liquid effluents.

3/4.11.1.4 LIQUID HOLDUP TANKS. Restricting the quantity of radioactive material contained in the specified tanks provides assurance that in the event of an uncontrolled release of the tanks' contents, the resulting concentrations would be less than the limits of 10 CFR Part 20, Appendix B, Table II, Column 2, at the nearest potable water supply and the nearest surface water supply in an UNRESTRICTED AREA.

This specification, being applicable to outside temporary tanks, does not apply to the refueling water storage tank, primary water storage tank, or the condensate storage tank, since they are a part of the permanent plant design.

3/4.11.2 GASEOUS EFFLUENTS

3/4.11.2.1 DOSE RATE. This specification is provided to ensure that the dose rate at any time at the SITE BOUNDARY from gaseous effluents from all units on the site will be within the annual dose limits of 10 CFR Part 20 for UNRESTRICTED AREAS. The annual dose limits are the doses associated with the concentrations of 10 CFR Part 20, Appendix B, Table II. These limits provide reasonable assurance that radioactive material discharged in gaseous effluents will not result in the exposure of an individual in an UNRESTRICTED AREA, to annual average concentrations exceeding the limits specified in Appendix B, Table II of 10 CFR Part 20 (10 CFR Part 20.106(b)). For individuals who may at times be within the SITE BOUNDARY, the occupancy of the individual will be sufficiently low to compensate for any increase in the atmospheric diffusion factor above that for the SITE BOUNDARY. The specified release rate limits restrict, at all times, the corresponding gamma and beta dose rates above background to an individual at or beyond the site boundary to less than or equal to 500 mrem/year to the total body or to less than or equal to 3000 mrem/year to the skin. These release rate limits also restrict, at all times, the corresponding thyroid dose rate above background to a child via inhalation pathway to less than or equal to 1500 mrems/year. Iodine adsorbing media refers to silver zeolite cartridges in Table 4.11-2 or the industry standard.

This specification applies to the release of gaseous effluents from all reactors at the site. The gaseous effluents from the shared system are proportioned among the units sharing that system.

TABLE 3.12-1 (Cont)

d. Sediment from Shoreline	L2, L3	2/year	Gamma Isotopic Analyses Semi-Annually.
4. Ingestion			
a. Milk	Each indicator farm and a background farm*	At least once per 15 days when animals are on Pasture. At Least Once Per 31 Days at Other Times.	Gamma Isotopic and I-131 Analysis of Each Sample.
b. Fish	Plant Site Off-Site	2/year	Gamma Isotopic Analysis on Edible Portion.
c. Food Products	Plant Site Off-Site (approx. 20 mi)	At time of Harvest One Sample of Each of the Following Classes of Food Products: 1. Grapes	Gamma Isotopic Analysis on Edible Portion.

^a Particulate sample filters should be analyzed for gross beta 24 hours or more after sampling to allow for radon and thoron daughter decay. If gross beta activity in air or water is greater than 10 times the yearly mean of control samples for any medium, gamma isotopic analysis should be performed on the individual samples.

* An indicator farm is defined as the nearest milk producer in each of the land sectors within 8 miles of the plant site who is willing to participate in the radiological environmental monitoring program. A background farm is defined as a milk producer in one of the less prevalent wind directions at a distance greater than 15 miles but less than 25 miles who is willing to participate in the radiological environmental monitoring program. If at least three indicator milk samples and one background milk sample cannot be obtained, vegetation sampling will be performed as a replacement for the milk sampling and no milk samples will be required.

TABLE 3.12-1 (Cont)

Plant Site	At time of Harvest One sample of Broad Leaf Vegetation	Gamma Isotopic Analysis.
3 indicator samples of broad leaf vegetation grown nearest to the offsite locations of highest calculated annual average ground level D/Q if at least three indicator milk samples and one background milk sample cannot be obtained.	Monthly when available	Gamma Isotopic and I-131 monthly when available
1 background sample of each of the similar vegetation grown 15-25 miles distant and in one of the less prevalent wind directions if at least three indicator milk samples and one background milk sample cannot be obtained.	Monthly when available	Gamma Isotopic and I-131 monthly when available

BASES

3/4.11.1.3 LIQUID WASTE TREATMENT. The OPERABILITY of the liquid radwaste treatment system ensures that this system will be available for use whenever liquid effluents require treatment prior to release to the environment. The requirements that the appropriate portions of this system be used when specified provide assurance that the releases of radioactive materials in liquid effluents will be kept "as low as is reasonably achievable." This specification implements the requirements of 10 CFR Part 50.36a, General Design Criteria Section 11.1 of the Final Safety Analysis Report for the Donald C. Cook Nuclear Plant, and design objective Section II.D of Appendix I to 10 CFR Part 50. The specified limits governing the use of appropriate portions of the liquid radwaste treatment system were specified as a suitable fraction of the dose design objectives set forth in Section II.A of Appendix I, 10 CFR Part 50, for liquid effluents.

3/4.11.1.4 LIQUID HOLDUP TANKS. Restricting the quantity of radioactive material contained in the specified tanks provides assurance that in the event of an uncontrolled release of the tanks' contents, the resulting concentrations would be less than the limits of 10 CFR Part 20, Appendix B, Table II, Column 2, at the nearest potable water supply and the nearest surface water supply in an UNRESTRICTED AREA.

This specification, being applicable to outside temporary tanks, does not apply to the refueling water storage tank, primary water storage tank, or the condensate storage tank, since they are a part of the permanent plant design.

3/4.11.2 GASEOUS EFFLUENTS

3/4.11.2.1 DOSE RATE. This specification is provided to ensure that the dose rate at any time at the SITE BOUNDARY from gaseous effluents from all units on the site will be within the annual dose limits of 10 CFR Part 20 for UNRESTRICTED AREAS. The annual dose limits are the doses associated with the concentrations of 10 CFR Part 20, Appendix B, Table II. These limits provide reasonable assurance that radioactive material discharged in gaseous effluents will not result in the exposure of an individual in an UNRESTRICTED AREA, to annual average concentrations exceeding the limits specified in Appendix B, Table II of 10 CFR Part 20 (10 CFR Part 20.106(b)). For individuals who may at times be within the SITE BOUNDARY, the occupancy of the individual will be sufficiently low to compensate for any increase in the atmospheric diffusion factor above that for the SITE BOUNDARY. The specified release rate limits restrict, at all times, the corresponding gamma and beta dose rates above background to an individual at or beyond the site boundary to less than or equal to 500 mrem/year to the total body or to less than or equal to 3000 mrem/year to the skin. These release rate limits also restrict, at all times, the corresponding thyroid dose rate above background to a child via inhalation pathway to less than or equal to 1500 mrem/year. Iodine adsorbing media refers to silver zeolite cartridges in Table 4.11-2 or the industry standard.

This specification applies to the release of gaseous effluents from all reactors at the site. The gaseous effluents from the shared system are proportioned among the units sharing that system.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO.119 TO FACILITY OPERATING LICENSE NO. DPR-58
AND AMENDMENT NO.105 TO FACILITY OPERATING LICENSE NO. DPR-74
INDIANA MICHIGAN POWER COMPANY
DONALD C. COOK NUCLEAR PLANT, UNITS NOS. 1 AND 2
DOCKETS NOS. 50-315 AND 50-316

1.0 INTRODUCTION

By letter dated February 1, 1988, the Indiana Michigan Power Company (the licensee) requested amendments to the Technical Specifications (TSs) appended to Facility Operating Licenses Nos. DPR-58 and DPR-74 for the Donald C. Cook Nuclear Plant, Units Nos. 1 and 2. The proposed amendments would correct the current TSs relating to milk sampling. The proposed changes would make the TSs more consistent with the NRC guidelines concerning milk sampling. In addition, the proposed changes would modify the Bases for TS 3/4.11.2.1, "Dose Rate". The proposed changes would make the TS bases more consistent with the guidance provided in the Bases Section of NUREG-0472, Revision 3, and NUREG-0452, Revision 5. The proposed TS changes would also correct an editorial error by deleting redundant \leq signs.

2.0 EVALUATION

The proposed changes to the milk sampling TSs are intended to correct problems with the current TSs. Item 4a of Table 3.12-1 of the TSs requires that milk samples be collected for radiological analysis from the following specific areas:

- A. Stevensville, Michigan
- B. Bridgman, Michigan
- C. Galien, Michigan
- D. Dowagiac, Michigan
- E. South Bend, Indiana

However, no milk samples are collected in Stevensville or South Bend or sectors in which these cities are located because there are no willing farmers who wish to participate in the milk sampling program. Literal TS noncompliance was caused by listing the specific towns where samples were to be taken.

The proposed changes to the TSs are more consistent with the NRC guidelines. Specifically, the proposed changes require sampling at each indicator farm and each background farm. Indicator farm and background farm are defined as follows:

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Indicator Farm Nearest milk producer in each of the land sectors within 8 miles of the plant site who is willing to participate in the radiological environmental monitoring program.

Background Farm A milk producer in one of the less prevalent wind directions at a distance greater than 15 miles but less than 25 miles who is willing to participate in the radiological environmental monitoring program.

The number of locations sampled may vary due to the number of sectors which contain farms willing to participate in the milk sampling program. The possibility exists that no willing participants may be found within 8 miles of the plant site. In order to address this possibility, the proposed TSs require broad leaf vegetation sampling. Specifically, if fewer than three willing indicator farms are found, broad leaf vegetation samples will be collected and analyzed when available. The proposed changes constitute an improvement over the current TSs. In addition, the proposed changes will make the TSs more restrictive than the current TSs and are more in line with NRC guidance.

The proposed TS changes also would modify the Bases for TS 3/4.11.2.1, "Dose Rate". This change would require the licensee to base the thyroid dose rate limits on the child inhalation pathway rather than the cow-milk-infant pathway. The child inhalation pathway is a more direct pathway which results in less decay time and correspondingly higher calculated doses and is therefore more conservative than the less direct (longer decay time) cow-milk-infant pathway. Further, the distance to the nearest residence in each of the land covered sections is significantly closer to the point of discharge than the nearest commercial dairy farm. Thus, the concentration which would be inhaled is greater than the quantity which would be deposited on the ground to be incorporated into the grass-cow-milk-infant pathway. Accordingly, the proposed change to the bases makes the requirements more stringent than the existing requirements and will make the TSs more consistent with NRC guidance.

The proposed TS changes will also remove redundant \leq signs from Bases Section 3/4.11.2.1. This is an editorial change and does not affect the safe operation of the plant.

Based on the above evaluation, we find that the proposed changes to the TSs are acceptable.

3.0 ENVIRONMENTAL CONSIDERATION

An Environmental Assessment and Finding of No Significant Impact has been issued for these amendments (53 FR 46132), November 16, 1988).

4.0 CONCLUSION

We have concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Date: December 6, 1988

Principal Contributors: Wayne Meinke, John Stang

UNITED STATES NUCLEAR REGULATORY COMMISSIONINDIANA MICHIGAN POWER COMPANYDOCKETS NOS. 50-315 AND 50-316NOTICE OF ISSUANCE OF AMENDMENTS TOFACILITY OPERATING LICENSES

The United States Nuclear Regulatory Commission (the Commission) has issued Amendments Nos. 119 and 105 to Facility Operating Licenses Nos. DPR-58 and DPR-74, issued to the Indiana Michigan Power Company (the licensee), which revised the Technical Specifications (TSs) for operation of the Donald C. Cook Nuclear Plant, Units Nos. 1 and 2, located in Berrien County, Michigan. The amendments are effective as of the date of issuance.

These amendments revise the TSs to make them more consistent with NRC guidelines concerning obtaining milk samples for analysis. In addition, the TS bases change to be more consistent with the Westinghouse Standard TSs with regard to the thyroid dose release pathway for a child, and an editorial error is corrected by removing redundant \leq signs.

The application for the amendments complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings, as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments.

Notice of Consideration of Issuance of Amendments to Facility Operating Licenses and Opportunity for Hearing in connection with this action was

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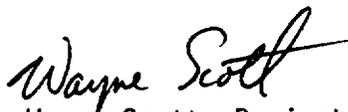
published in the FEDERAL REGISTER on July 14, 1988 (53 FR 26695). No request for hearing or petition to intervene was filed following this notice.

Also in connection with this action, the Commission prepared an Environmental Assessment and Finding of No Significant Impact which was published in the FEDERAL REGISTER on November 16, 1988, at 53 FR 46132 .

For further details with respect to this action, see (1) the application for amendments dated February 1, 1988, (2) Amendment Nos. 119 and 105 to Licenses Nos. DPR-58 and DPR-74, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 2120 L Street, NW., Washington, DC 20555, and at the Maude Preston Palenski Memorial Library, 500 Market Street, St. Joseph, Michigan 49085. A copy of items (2) and (3) may be obtained upon request addressed to the U.S. Nuclear Regulatory Commission, Washington, DC 20555, Attention: Director, Division of Reactor Projects - III, IV, V, and Special Projects.

Dated at Rockville, Maryland, this 6th day of December 1988.

FOR THE NUCLEAR REGULATORY COMMISSION



Wayne Scott, Project Manager
Project Directorate III-1
Division of Reactor Projects - III, IV, V
& Special Projects