December 10, 1987

Docket Nos. 50-315
and 50-316

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Mr. John Dolan, Vice President
Indiana and Michigan Power Company
c/o American Electric Power Service Corporation
1 Riverside Plaza
Columbus, Ohio 43216

Dear Mr. Dolan:

The Commission has issued the enclosed Amendment No. 112 to Facility Operating License No. DPR-58 and Amendment No. 95 to Facility Operating License No. DPR-74 for the Donald C. Cook Nuclear Plant, Unit Nos. 1 and 2. The amendments consist of changes to the Technical Specifications in response to your application dated May 19, 1986 and revised July 16, 1987.

These amendments revise the Technical Specifications by adding requirements for sump level instrumentation to satisfy the NUREG 0737 Item II.F.1.5.

A copy of the Safety Evaluation is also enclosed. The notice of issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

/s/

David L. Wigginton, Project Manager
Project Directorate III-3
Division of Reactor Projects

Enclosures:
1. Amendment No. 112 to DPR-58
2. Amendment No. 95 to DPR-74
3. Safety Evaluation

cc w/enclosures:
See next page

Office:	LA/PDIII-3	PM/PDIII-3	PD/PDIII-3	OGC
Surname:	PKreutzer	DWigginton/tg	KPerkins	R Bachmann
Date:	12/2/87	12/3/87	12/3/87 12/10/87	12/4/87

cc:
Mr. M. P. Alexich
Vice President
Nuclear Operations
American Electric Power Service
Corporation
1 Riverside Plaza
Columbus, Ohio 43215

Attorney General
Department of Attorney General
525 West Ottawa Street
Lansing, Michigan 48913

Township Supervisor
Lake Township Hall
Post Office Box 818
Bridgeman, Michigan 49106

W. G. Smith, Jr., Plant Manager
Donald C. Cook Nuclear Plant
Post Office Box 458
Bridgman, Michigan 49106

U.S. Nuclear Regulatory Commission
Resident Inspectors Office
7700 Red Arrow Highway
Stevensville, Michigan 49127

Gerald Charnoff, Esquire
Shaw, Pittman, Potts and Trowbridge
2300 N Street, N.W.
Washington, DC 20037

Mayor, City of Bridgeman
Post Office Box 366
Bridgeman, Michigan 49106

Special Assistant to the Governor
Room 1 - State Capitol
Lansing, Michigan 48909

Nuclear Facilities and Environmental
Monitoring Section Office
Division of Radiological Health
Department of Public Health
3500 N. Logan Street
Post Office Box 30035
Lansing, Michigan 48909

Regional Administrator, Region III
U.S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, Illinois 60137

Mr. J. Feinstein
American Electric Power
Service Corporation
1 Riverside Plaza
Columbus, Ohio 43216

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

INDIANA AND MICHIGAN POWER COMPANY

DOCKET NO. 50-315

DONALD C. COOK NUCLEAR PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 112
License No. DPR-58

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Indiana and Michigan Power Company (the licensee) dated May 19, 1986 and revised July 16, 1987 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-58 is hereby amended to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 112 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. The Technical Specifications are to become effective before startup following the refueling outage currently scheduled in February 1989.

4. This license amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Kenneth E. Perkins, Jr. Director
Project Directorate III-3
Division of Reactor Projects

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 10, 1987

ATTACHMENT TO LICENSE AMENDMENTS

AMENDMENT NO. 112 FACILITY OPERATING LICENSE NO. DPR-58

DOCKET NO.50-315

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by amendment number and contain marginal lines indicating the area of change.

REMOVE	INSERT
Unit 1	
3/4 3-55	3/4 3-55**
3/4 3-56	3/4 3-56**
3/4 4-15	3/4 4-15
3/4 4-16	3/4 4-16*
B 3/4 3-4	B 3/4 3-4**

*Included for convenience only.
**Effective before start up following the refueling outage currently scheduled in February 1989.

TABLE 3.3-11
POST-ACCIDENT MONITORING INSTRUMENTATION

INSTRUMENT	MINIMUM CHANNELS OPERABLE
1. Containment Pressure	2
2. Reactor Coolant Outlet Temperature - T_{HOT} (Wide Range)	2
3. Reactor Coolant Inlet Temperature - T_{COLD} (Wide Range)	2
4. Reactor Coolant Pressure - Wide Range	2
5. Pressurizer Water Level	2
6. Steam Line Pressure	2/Steam Generator
7. Steam Generator Water Level - Narrow Range	1/Steam Generator
8. Refueling Water Storage Tank Water Level	2
9. Boric Acid Tank Solution Level	1
10. Auxiliary Feedwater Flow Rate	1/Steam Generator*
11. Reactor Coolant System Subcooling Margin Monitor	1**
12. PORV Position Indicator - Limit Switches***	1/Valve
13. PORV Block Valve Position Indicator - Limit Switches	1/Valve
14. Safety Valve Position Indicator - Acoustic Monitor	1/Valve
15. Incore Thermocouples (Core Exit Thermocouples)	2/Core Quadrant
16. Reactor Coolant Inventory Tracking System (Reactor Vessel Level Indication)	One Train (3 channels/Train)
17. Containment Sump Level	1****
18. Containment Water Level	2****

112

* Steam Generator Water Level Channels can be used as a substitute for the corresponding auxiliary feedwater flow rate channel instrument

** PRODAC 250 subcooling margin readout can be used as a substitute for the subcooling monitor instrument.

*** Acoustic monitoring of PORV position (1 channel per three valves - headered discharge) can be used as a substitute for the PORV Indicator - Limit Switches instruments.

**** The requirements for these instruments will become effective after the level transmitters are modified or replaced and become operational. The schedule for modification or replacement of the transmitters is described in the Bases.

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TABLE 4.3-7

POST-ACCIDENT MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

INSTRUMENT	CHANNEL CHECK	CHANNEL CALIBRATION
1. Containment Pressure	M	R*
2. Reactor Coolant Outlet Temperature - T_{HOT} (Wide Range)	M	R
3. Reactor Coolant Inlet Temperature - T_{COLD} (Wide Range)	M	R
4. Reactor Coolant Pressure - Wide Range	M	R
5. Pressurizer Water Level	M	R*
6. Steam Line Pressure	M	R*
7. Steam Generator Water Level - Narrow Range	M	R*
8. RWST Water Level	M	R
9. Boric Acid Tank Solution Level	M	R
10. Auxiliary Feedwater Flow Rate	M	R
11. Reactor Coolant System Subcooling Margin Monitor	M	R
12. PORV Position Indicator - Limit Switches	M	R*
13. PORV Block Valve Position Indicator - Limit Switches	M	R*
14. Safety Valve Position Indicator - Acoustic Monitor	M	R*
15. Incore Thermocouples (Core Exit Thermocouples)	M	R(1)
16. Reactor Coolant Inventory Tracking System (Reactor Vessel Level Indication)	M(2)	R(3)
17. Containment Sump Level**	M	R
18. Containment Water Level**	M	R

(1) Partial range channel calibration for sensor to be performed below P-12 in MODE 3.

(2) With one train of Reactor Vessel Level Indication inoperable, Subcooling Margin Indication and Core Exit Thermocouples may be used to perform a CHANNEL CHECK to verify the remaining Reactor Vessel Indication train OPERABLE.

(3) Completion of channel calibration for sensors to be performed below P-12 in MODE 3.

* The provisions of Specification 4.0.6 are applicable.

** The requirements for these instruments will become effective after the level transmitters are modified or replaced and become operational. The schedule for modification or replacement of the transmitters is described in the Bases.

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REACTOR COOLANT SYSTEM

OPERATIONAL LEAKAGE

LIMITING CONDITION FOR OPERATION

3.4.6.2 Reactor Coolant System leakage shall be limited to:

a. No PRESSURE BOUNDARY LEAKAGE,

b. 1 GPM UNIDENTIFIED LEAKAGE,

c. 1 GPM total primary-to-secondary leakage through all steam generators and 500 gallons per day through any one steam generator,

d. 10 GPM IDENTIFIED LEAKAGE from the Reactor Coolant System, and

e. 52 GPM CONTROLLED LEAKAGE.

f. 1 GPM leakage from any reactor coolant system pressure isolation valve specified in Table 3.4-0.

APPLICABILITY: MODES 1, 2, 3 and 4

ACTION:

a. With any PRESSURE BOUNDARY LEAKAGE, be in at least HOT STANDBY within 6 hours and in COLD SHUTDOWN within the following 30 hours.

b. With any Reactor Coolant System leakage greater than any one of the above limits, excluding PRESSURE BOUNDARY LEAKAGE, reduce the leakage rate to within limits within 4 hours or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

c. With any reactor coolant system pressure isolation valve(s) leakage greater than the above limit, except when:

1. The leakage is less than or equal to 5.0 gpm, and

2. The most recent measured leakage does not exceed the previous measured leakage* by an amount that reduces the

*To satisfy ALARA requirements, measured leakage may be measured indirectly (as from the performance of pressure indicators) if accomplished in accordance with approved procedures and supported by computations showing that the method is capable of demonstrating valve compliance with the leakage criteria.

3/4.3.3.7 FIRE DETECTION INSTRUMENTATION

OPERABILITY of the fire detection instrumentation ensures that adequate warning capability is available for the prompt detection of fires. This capability is required in order to detect and locate fires in their early stages. Prompt detection of fires will reduce the potential for damage to safety related equipment and is an integral element in the overall facility fire protection program.

In the event that a portion of the fire detection instrumentation is inoperable, the establishment of frequent fire patrols in the affected areas is required to provide detection capability until the inoperable instrumentation is restored to OPERABILITY. Use of containment temperature monitoring is allowed once per hour if containment fire detection is inoperable.

3/4.3.3.8 POST-ACCIDENT INSTRUMENTATION

The OPERABILITY of the post-accident instrumentation ensures that sufficient information is available on selected plant parameters to monitor and assess these variables during and following an accident.

The containment water level and containment sump level transmitters will be modified or replaced and OPERABLE by the end of the refueling outage scheduled to begin in February 1989.

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

INDIANA AND MICHIGAN POWER COMPANY

DOCKET NO. 50-316

DONALD C. COOK NUCLEAR PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 95
License No. DPR-74

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Indiana and Michigan Power Company (the licensee) dated May 19, 1986 and revised July 17, 1987 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-74 is hereby amended to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 95 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. The Technical Specifications are to become effective before startup following the refueling outage currently scheduled to begin in early 1988.

4. This license amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Kenneth E. Perkins, Jr., Director
Project Directorate III-3
Division of Reactor Projects

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 10, 1987

ATTACHMENT TO LICENSE AMENDMENTS

AMENDMENT NO 95 FACILITY OPERATING LICENSE NO. DPR-74

DOCKET NO.50-316

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by amendment number and contain marginal lines indicating the area of change.

REMOVE	INSERT
Unit 2	
3/4 3-46	3/4 3-46
3/4 3-47	3/4 3-47
B 3/4 3-2	B 3/4 3-2

Effective before startup following refueling outage currently scheduled to begin in early 1988.

TABLE 3.3-10
POST-ACCIDENT MONITORING INSTRUMENTATION

INSTRUMENT	MINIMUM CHANNELS OPERABLE
1. Containment Pressure	2
2. Reactor Coolant Outlet Temperature - T_{HOT} (Wide Range)	2
3. Reactor Coolant Inlet Temperature - T_{COLD} (Wide Range)	2
4. Reactor Coolant Pressure - Wide Range	2
5. Pressurizer Water Level	2
6. Steam Line Pressure	2/Steam Generator
7. Steam Generator Water Level - Narrow Range	1/Steam Generator
8. Refueling Water Storage Tank Water Level	2
9. Boric Acid Tank Solution Level	1
10. Auxiliary Feedwater Flow Rate	1/Steam Generator*
11. Reactor Coolant System Subcooling Margin Monitor	1**
12. PORV Position Indicator - Limit Switches***	1/Valve
13. PORV Block Valve Position Indicator - Limit Switches	1/Valve
14. Safety Valve Position Indicator - Acoustic Monitor	1/Valve
15. Incore Thermocouples (Core Exit Thermocouples)	2/Core Quadrant
16. Reactor Coolant Inventory Tracking System (Reactor Vessel Level Indication)	One Train (3 channels/Train)
17. Containment Sump Level	1****
18. Containment Water Level	2****

95

* Steam Generator Water Level Channels can be used as a substitute for the corresponding auxiliary feedwater flow rate channel instrument

** PRODAC 250 subcooling margin readout can be used as a substitute for the subcooling monitor instrument.

*** Acoustic monitoring of PORV position (1 channel per three valves - headered discharge) can be used as a substitute for the PORV Indicator - Limit Switches instruments.

**** The requirements for these instruments will become effective after the level transmitters are modified or replaced and become operational. The schedule for modification or replacement of the transmitters is described in the Bases.

95

Amendment No. 92, 95 (Effective before start up following refueling outage currently scheduled in early 1988)

TABLE 4.3-10

POST-ACCIDENT MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

INSTRUMENT	CHANNEL CHECK	CHANNEL CALIBRATION
1. Containment Pressure	M	R
2. Reactor Coolant Outlet Temperature - T_{HOT} (Wide Range)	M	R
3. Reactor Coolant Inlet Temperature - T_{COLD} (Wide Range)	M	R
4. Reactor Coolant Pressure - Wide Range	M	R
5. Pressurizer Water Level	M	R
6. Steam Line Pressure	M	R
7. Steam Generator Water Level - Narrow Range	M	R
8. RWST Water Level	M	R
9. Boric Acid Tank Solution Level	M	R
10. Auxiliary Feedwater Flow Rate	M	R
11. Reactor Coolant System Subcooling Margin Monitor	M	R
12. PORV Position Indicator - Limit Switches	M	R
13. PORV Block Valve Position Indicator - Limit Switches	M	R
14. Safety Valve Position Indicator - Acoustic Monitor	M	R
15. Incore Thermocouples (Core Exit Thermocouples)	M	R(1)
16. Reactor Coolant Inventory Tracking System (Reactor Vessel Level Indication)	M(2)	R(3)
17. Containment Sump Level*	M	R
18. Containment Water Level*	M	R

(1) Partial range channel calibration for sensor to be performed below P-12 in MODE 3.

(2) With one train of Reactor Vessel Level Indication inoperable, Subcooling Margin Indication and Core Exit Thermocouples may be used to perform a CHANNEL CHECK to verify the remaining Reactor Vessel Indication train OPERABLE.

(3) Completion of channel calibration for sensors to be performed below P-12 in MODE 3.

* The requirements for these instruments will become effective after the level transmitters are modified or replaced and become operational. The schedule for modification or replacement of the transmitters is described in the Bases.

3/4.3.3 MONITORING INSTRUMENTATION

3/4.3.3.1 RADIATION MONITORING INSTRUMENTATION

The OPERABILITY of the radiation monitoring channels ensures that 1) the radiation levels are continually measured in the areas served by the individual channels and 2) the alarm or automatic action is initiated when the radiation level trip setpoint is exceeded.

3/4.3.3.2 MOVABLE INCORE DETECTORS

The OPERABILITY of the movable incore detectors with the specified minimum complement of equipment ensures that the measurements obtained from use of this system accurately represent the spatial neutron flux distribution of the reactor core. The OPERABILITY of this system is demonstrated by irradiating each detector used and normalizing its respective output.

3/4.3.3.3 SEISMIC INSTRUMENTATION

The OPERABILITY of the seismic instrumentation ensures that sufficient capability is available to promptly determine the magnitude of a seismic event and evaluate the response of those features important to safety. This capability is required to permit comparison of the measured response to that used in the design basis for the facility.

3/4.3.3.4 METEOROLOGICAL INSTRUMENTATION

The OPERABILITY of the meteorological instrumentation ensures that sufficient meteorological data is available for estimating potential radiation doses to the public as a result of routine or accidental release of radioactive materials to the atmosphere. This capability is required to evaluate the need for initiating protective measures to protect the health and safety of the public.

3/4.3.3.5 REMOTE SHUTDOWN INSTRUMENTATION

The OPERABILITY of the remote shutdown instrumentation ensures that sufficient capability is available to permit shutdown and maintenance of HOT STANDBY of the facility from locations outside of the control room. This capability if required in the event control room habitability is lost and is consistent with General Design Criteria 19 of 10 CFR 50.

3/4.3.3.6 POST-ACCIDENT INSTRUMENTATION

The OPERABILITY of the post-accident instrumentation ensures that sufficient information is available on selected plant parameters to monitor and assess these variables during and following an accident.

The containment water level and containment sump level transmitters will be modified or replaced and OPERABLE by the end of the outage currently scheduled to begin in May 1988. 95

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO.112 TO FACILITY OPERATING LICENSE NO. DPR-58
AND AMENDMENT NO. 95 TO FACILITY OPERATING LICENSE NO. DPR-74

INDIANA AND MICHIGAN POWER COMPANY
DONALD C. COOK NUCLEAR PLANT, UNIT NOS. 1 AND 2
DOCKET NOS. 50-315 AND 50-316

1.0 BACKGROUND

NUREG-0737, "Clarification of TMI Action Plan Requirements," sets forth all required post TMI-related modifications to operating plants including schedules, applicability, method of implementation review, and licensee submittal dates.

Section II.F.1.5 of NUREG-0737 (with additional guidance from Regulatory Guide 1.97) requires that a continuous indication of containment water level be provided in the control room for all plants, both PWR's and BWR's. For PWR's, a narrow range instrument must be provided to measure from the bottom to the top of the containment sump. Wide range instruments must be available to measure from the bottom of the containment to the elevation equivalent to 600,000 gallons capacity. NUREG-0737 also requires that technical specifications (TS) be developed which provide assurance that facility operation is maintained within limits determined acceptable following the implementation of specific TMI action items, including containment water level monitoring.

Generic Letter 83-37, dated November 1, 1983, set forth guidance on the scope and content of TS covering NUREG-0737 items. Among the guidance provided was a proposed specification which set forth minimum channel requirements, limiting conditions for operation (LCO) and surveillance requirements for containment water level instrumentation.

The generic letter called for one channel of narrow range monitoring of containment sump level, and two channels for wide range monitoring of containment water level. The generic letter also set forth an LCO action statement, for narrow range monitors, which requires that an inoperable channel be restored to operable status within 30 days or the plant be brought to hot shutdown as required for other accident monitoring instrumentation. The suggestion for wide range monitoring instrumentation placed no specific outage time constraint on inoperable channels. The guidance suggested that the LCO action statement for wide range should be similar to that for other accident monitoring instrumentation included in a plant's existing technical specifications.

On July 19, 1984, the licensee submitted proposed technical specifications covering changes made pursuant to NUREG-0737, Section II.F.1.5. These changes deviated from the guidance contained in the generic letter by proposing that only one channel be required for wide range monitoring. The proposed changes further neglected to impose requirements for periodic channel checks of either wide or narrow range instrument channels as part of surveillance requirements.

Subsequent to a January 13, 1986, meeting between the staff and the licensee, the licensee's proposals were modified and resubmitted by letter dated May 19, 1986. Unlike the original proposal, the revised proposal requested TS changes to require (1) routine monthly channel checks for both containment sump (narrow range) and containment (wide range) water level instrumentation, and (2) the addition of two instrumentation channels to monitor containment water (wide range) level rather than the single channel as originally proposed by the licensee. The revised proposal also requested that the existing 30-day action statement in the Units 1 and 2 TS for other accident monitoring instrumentation be applied to containment water level instrumentation (wide range). These revisions brought the TS into accord with the guidance of the generic letter as far as recommended instrumentation and required surveillances are concerned. However, with respect to limiting conditions for operation and associated action statements for containment sump level, the licensee's revised proposal continued to deviate from the guidance contained in the generic letter.

The generic letter calls for the LCO for sump level instrumentation to include the requirement that the inoperable channel be restored to operable status within 30 days or the plant be brought to hot shutdown condition as required for other accident monitoring instrumentation. The plants' current TS require reactor shutdown within 12 hours following a 30-day period of inoperability of other accident monitoring instrumentation. But for inoperability of the sump level instrumentation channel, the licensee proposed to deviate from this shutdown requirement by substituting a requirement to submit a special report to the Regional Administrator. This report would outline available backup equipment, the cause of the inoperability, and the plans and schedule for restoring the system to operable status. This report would be submitted pursuant to Technical Specification 6.9.2, Special Reports, which the licensee proposed to amend to reflect its applicability to sump level instrumentation. The licensee felt justified in exempting containment sump level instrumentation from adhering to the 12-hour shutdown provision applied to other instrumentation channels because of the number of backup instrumentation systems available (humidity monitors, sump pump run time, containment atmosphere particulate radioactivity monitoring channels, containment atmosphere gaseous radioactivity monitoring channels, the containment water level instrumentation, and refueling water storage tank water level).

The licensee's May 19, 1986, letter also contained a proposed change to the Bases for Post-Accident Instrumentation which stated that (1) an acceptable instrument drift for containment water level channels is 25 percent of full scale, and (2) for sump level channels, a difference between readings on the two parallel channels of up to 25 percent of full scale is acceptable.

In a letter dated May 6, 1987, the staff provided a preliminary safety evaluation (SE) which addressed the proposed technical specification changes and requested the licensee to review the staff positions on the deviation for the sump level instrumentation LCO and on the unacceptability of a 25 percent drift in containment water level instrumentation. In response to the staff's request, in a July 16, 1987, letter the licensee has now proposed revised changes to the plants' Technical specifications which are intended to supersede the previously proposed changes for containment sump level and containment water level in their entirety.

In the May 19, 1986 letter, the licensee also proposed an editorial change to remove a duplication on the operational leakage Technical Specifications. The exact same words are currently shown on two separate pages.

2.0 EVALUATION

The licensee proposed to add containment sump level and containment water level instrumentation to the current post-accident monitoring instrumentation tables in the plants' technical specifications with one channel and two channels required, respectively (with an LCO of 30 days followed by hot shutdown). The licensee proposed a monthly channel check and a refueling outage channel calibration be added for the above instruments in the existing tables of post-accident monitoring instrumentation surveillance requirements in the plants' technical specifications. The staff finds these proposed changes acceptable on the basis that they follow the guidance contained in Generic Letter 83-37.

The licensee proposed to add a footnote: "The requirements for these instruments will become effective after the level transmitters are modified or replaced and become operational. The schedule for modification or replacement of the transmitters is described in the Bases." to the new entries (discussed above) for containment sump level and containment water level instrumentation. The licensee proposed to add: "The containment water level and containment sump level transmitters will be modified or replaced and OPERABLE by the end of the refueling outage scheduled to begin in February 1989." to Bases Section 3/4.3.3.8 in the Unit 1 technical specifications and to add: "The containment water level and containment sump level transmitters will be modified or replaced and OPERABLE by the end of the outage currently scheduled to begin in May 1988." to Bases Section 3/4.3.3.6 in the Unit 2 technical specifications. The staff finds these proposed changes acceptable on the basis that they reflect the schedular requirements associated with Generic Letter 83-37 and Regulatory Guide 1.97.

The licensee's proposed changes to correct the duplicative requirements on the operational leakage Technical Specifications for Unit 1 are also acceptable.

3.0 ENVIRONMENTAL CONSIDERATION

These amendments involve a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 or changes an inspection or surveillance requirement. The staff has determined that the amendments involve no significant increase in the amounts, and no

significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously published a proposed finding that these amendments involve no significant hazards consideration and there has been no public comment on such finding. Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR §51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

4.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: F. Burrows

Dated: December 10, 1987