April 19, 2002

Dr. George E. Apostolakis, Chairman Advisory Committee on Reactor Safeguards U. S. Nuclear Regulatory Commission Washington, D. C. 20555-0001

SUBJECT: PHASE 2 PRE-APPLICATION REVIEW FOR AP1000 PASSIVE PLANT

DESIGN

Dear Dr. Apostolakis:

Thank you for your letter of March 14, 2002, regarding the presentation to the Advisory Committee on Reactor Safeguards (ACRS) on March 8, 2002, of the staff's assessment of the AP1000 standard plant design pre-application review. The staff also had the opportunity to present a more detailed assessment to the ACRS subcommittees on Thermal-Hydraulic Phenomena and Future Plant Designs on February 14 and 15, 2002.

As presented on March 8, 2002, the scope of the staff's pre-application review was limited to assessing: (1) the applicability of the AP600 test program and analysis codes to the AP1000; (2) the applicability to AP1000 of three exemptions that were granted for the AP600; and (3) the acceptability of using design acceptance criteria (DAC) in lieu of detailed design information in the instrumentation and controls (I&C), control room (human factors engineering), and piping design areas.

The staff will document its assessment of the DAC issue in a Commission paper that should be made publicly available in April 2002. The staff documented its assessment of the other issues in a letter to Westinghouse Electric Company, dated March 25, 2002.

Your letter of March 14, 2002, conveyed ACRS conclusions and recommendations regarding the staff's pre-application review (Phase 2) and the planned design certification review (Phase 3) of the AP1000. In general, you indicated that ACRS agrees with the staff's conclusions regarding the pre-application review.

You also offered the recommendation that the Office of Nuclear Regulatory Research (RES) should further investigate acceptable ranges of ratios of Pi-groups for use in scaling. You stated that "[a]Ithough we do not believe that this work is needed for AP1000 certification, this issue is likely to arise with certification of future reactor designs and such a study could tie down this loose end of the code, scaling, applicability, and uncertainty (CSAU) process." The Office of Nuclear Reactor Regulation conveyed this recommendation to RES which plans to present its response at a future Thermal-Hydraulic Phenomena subcommittee meeting.

The staff will continue to engage ACRS during the design certification review. If you should have any questions or comments regarding the AP1000 standard plant design, please contact Mr. Lawrence J. Burkhart, the AP1000 Project Manager, at 301-415-3053 or ljb@nrc.gov.

Thank you for your continued support of the staff's review of the AP1000 standard design.

Sincerely,

/RA by Carl J. Paperiello Acting For/

William D. Travers Executive Director for Operations

Project No. 711

cc: Chairman Meserve
Commissioner Dicus
Commissioner Diaz
Commissioner McGaffigan
Commissioner Merrifield
SECY

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Commissioner McGaffigan Commissioner Merrifield

SECY

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