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 OPA  
 RDiggs  
 MGrotenhuis  
 CParrish

Docket Nos. 50-315  
and 50-316

Mr. John Dolan, Vice President  
Indiana and Michigan Electric Company  
Post Office Box 18  
Bowling Green Station  
New York, New York 10004

Dear Mr. Dolan:

The Commission has issued the enclosed Amendment No. 52 to Facility Operating License No. DPR-58 and Amendment No. 37 to Facility Operating License No. DPR-74 for the Donald C. Cook Nuclear Plant, Unit Nos. 1 and 2, respectively. The amendments consist of changes to the Technical Specifications in response to your application transmitted by letter dated May 26, 1981.

These amendments change the temperature requirements for the refueling water storage tank and makes several editorial changes.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

ORIGINAL SIGNED

Marshall Grotenhuis, Project Manager  
Operating Reactors Branch #1  
Division of Licensing

CP  
1

Enclosures:

1. Amendment No. 52 to DPR-58
2. Amendment No. 37 to DPR-74
3. Safety Evaluation
4. Notice of Issuance

cc w/enclosures:  
See next page

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PDR ADDCK 05000315  
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OFFICE	ORB#1:DL	ORB#1:DL	LGB	AD/CP	ORB#1:DL	AD/CP	OELD
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DATE	01/27/82	01/27/82:ds	01/28/82	01/28/82	01/28/82	01/28/82	02/1/82

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Indiana and Michigan Electric Company

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

INDIANA AND MICHIGAN ELECTRIC COMPANY

DOCKET NO. 50-315

DONALD C. COOK NUCLEAR PLANT UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 52  
License No. DPR-58

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Indiana and Michigan Electric Company (the licensee) dated May 26, 1981, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;  
and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

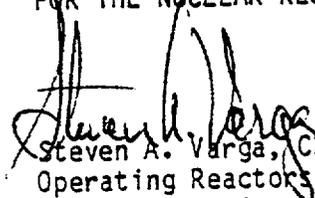
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-58 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 52, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

  
Steven A. Varga, Chief  
Operating Reactors Branch #1  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: February 10, 1982



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

INDIANA AND MICHIGAN ELECTRIC COMPANY

DOCKET NO. 50-316

DONALD C. COOK NUCLEAR PLANT UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 37  
License No. DPR-74

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Indiana and Michigan Electric Company (the licensee) dated May 26, 1981, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

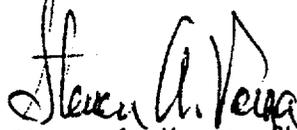
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-74 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 37, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Steven A. Varga, Chief  
Operating Reactors Branch #1  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: February 10, 1982

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 52 TO FACILITY OPERATING LICENSE NO. DPR-58

DOCKET NO. 50-315

Revise Appendix A as follows:

Remove Pages

1-7  
3/4 1-15  
3/4 2-2  
3/4 8-8

Insert Pages

1-7  
3/4 1-15  
3/4 2-2  
3/4 8-8

TABLE 1.2  
FREQUENCY NOTATION

<u>NOTATION</u>	<u>FREQUENCY</u>
S	At least once per 12 hours.
D	At least once per 24 hours.
W	At least once per 7 days.
M	At least once per 31 days
Q	At least once per 92 days.
SA	At least once per 184 days.
R	At least once per 549 days.
S/U	Prior to each reactor startup.
N.A.	Not applicable.

## REACTIVITY CONTROL SYSTEMS

### BORATED WATER SOURCES - SHUTDOWN

#### LIMITING CONDITION FOR OPERATION

---

3.1.2.7 As a minimum, one of the following borated water sources shall be OPERABLE:

- a. A boric acid storage system and associated heat tracing with:
  1. A minimum contained volume of 835 gallons,
  2. Between 20,000 and 22,500 ppm of boron, and
  3. A minimum solution temperature of 145°F.
- b. The refueling water storage tank with:
  1. A minimum contained volume of 9690 gallons,
  2. A minimum boron concentration of 1950 ppm, and
  3. A minimum solution temperature of 35°F.

APPLICABILITY: MODES 5 and 6.

#### ACTION:

With no borated water source OPERABLE, suspend all operations involving CORE ALTERATIONS or positive reactivity changes until at least one borated water source is restored to OPERABLE status.

#### SURVEILLANCE REQUIREMENTS

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4.1.2.7 The above required borated water source shall be demonstrated OPERABLE:

- a. At least once per 7 days by:
  1. Verifying the boron concentration of the water,
  2. Verifying the water level volume of the tank, and
  3. Verifying the boric acid storage tank solution temperature when it is the source of borated water.
- b. At least once per 24 hours by verifying the RWST temperature when it is the source of borated water and the outside air temperature is < 35°F.

## POWER DISTRIBUTION LIMITS

### LIMITING CONDITION FOR OPERATION (Continued)

- c) Surveillance testing of the APDMS may be performed pursuant to Specification 4.3.3.6.1 provided the indicated AFD is maintained within the limits of Figure 3.2-1. A total of 6 hours of operation may be accumulated with the AFD outside of the target band during this testing without penalty deviation.
- b. THERMAL POWER shall not be increased above 90% or  $0.9 \times \text{APL}$  (whichever is less) of RATED THERMAL POWER unless the indicated AFD is within the  $\pm 5\%$  target band and ACTION 2.a) 1), above has been satisfied.
- c. THERMAL POWER shall not be increased above 50% of RATED THERMAL POWER unless the indicated AFD has not been outside of the  $\pm 5\%$  target band for more than 1 hour penalty deviation cumulative during the previous 24 hours.
- d. During power reductions using control rods, the reporting requirements of Specification 6.9.1.9 shall not apply provided the action items above are satisfied.

### SURVEILLANCE REQUIREMENTS

4.2.1.1 The indicated AXIAL FLUX DIFFERENCE shall be determined to be within its limits during POWER OPERATION above 15% of RATED THERMAL POWER by:

- a. Monitoring the indicated AFD for each OPERABLE excore channel:
  - 1. At least once per 7 days when the AFD Monitor Alarm is OPERABLE, and
  - 2. At least once per hour for the first 24 hours after restoring the AFD Monitor Alarm to OPERABLE status.
- b. Monitoring and logging the indicated AXIAL FLUX DIFFERENCE for each OPERABLE excore channel at least once per hour for the first 24 hours and at least once per 30 minutes thereafter, when the AXIAL FLUX DIFFERENCE Monitor Alarm is inoperable. The logged values of the indicated AXIAL FLUX DIFFERENCE shall be assumed to exist during the interval preceding each logging.

## ELECTRICAL POWER SYSTEMS

### D.C. DISTRIBUTION - OPERATING

#### LIMITING CONDITION FOR OPERATION

---

3.8.2.3 The following D.C. bus trains shall be energized and OPERABLE with tie breakers between bus trains open:

TRAIN AB consisting of 250-volt D.C. bus AB, 250-volt D.C. battery bank AB, and a full capacity charger.

TRAIN CD consisting of 250-volt D.C. bus CD, 250-volt D.C. battery bank CD, and a full capacity charger.

APPLICABILITY: MODES 1, 2, 3 and 4.

#### ACTION:

- a. With 250-volt D.C. bus inoperable, restore the inoperable bus to OPERABLE status within 2 hours or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.
- b. With one 250-volt D.C. battery and/or its charger inoperable, restore the inoperable battery and/or charger to OPERABLE status within 2 hours or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

#### SURVEILLANCE REQUIREMENTS

---

4.8.2.3.1 Each D.C. bus train shall be determined OPERABLE and energized with tie breakers open at least once per 7 days by verifying correct breaker alignment and indicated power availability.

4.8.2.3.2 Each 250-volt battery bank and charger shall be demonstrated OPERABLE:

- a. At least once per 7 days by verifying that:
  1. The electrolyte level of each pilot cell is between the minimum and maximum level indication marks,

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 37 TO FACILITY OPERATING LICENSE NO. DPR-74

DOCKET NO. 50-316

Revise Appendix A as follows:

Remove Pages

1-8  
3/4 1-11  
3/4 1-15  
3/4 2-2  
3/4 8-9  
3/4 8-10

Insert Pages

1-8  
3/4 1-11  
3/4 1-15  
3/4 2-2  
3/4 8-9  
3/4 8-10

TABLE 1.2  
FREQUENCY NOTATION

<u>NOTATION</u>	<u>FREQUENCY</u>
S	At least once per 12 hours.
D	At least once per 24 hours.
W	At least once per 7 days.
M	At least once per 31 days
Q	At least once per 92 days.
SA	At least once per 184 days.
R	At least once per 549 days.
S/U	Prior to each reactor startup.
N.A.	Not applicable.

## REACTIVITY CONTROL SYSTEMS

### CHARGING PUMP - SHUTDOWN

#### LIMITING CONDITION FOR OPERATION

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3.1.2.3 At least one charging pump in the boron injection flow path required by Specification 3.1.2.1 shall be OPERABLE and capable of being powered from an OPERABLE emergency bus.

APPLICABILITY: MODES 5 and 6.

#### ACTION:

With no charging pump OPERABLE, suspend all operations involving CORE ALTERATIONS or positive reactivity changes until one charging pump is restored to OPERABLE status.

#### SURVEILLANCE REQUIREMENTS

---

4.1.2.3 At least the above required charging pump shall be demonstrated OPERABLE by verifying, that on recirculation flow, the pump develops a discharge pressure of  $\geq 2390$  psig when tested pursuant to Specification 4.0.5.

REACTIVITY CONTROL SYSTEMS

BORATED WATER SOURCES - SHUTDOWN

LIMITING CONDITION FOR OPERATION

---

3.1.2.7 As a minimum, one of the following borated water sources shall be OPERABLE:

- a. A boric acid storage system and associated heat tracing with:
  1. A minimum contained borated water volume 835 gallons,
  2. Between 20,000 and 22,500 ppm of boron, and
  3. A minimum solution temperature of 145°F.
- b. The refueling water storage tank with:
  1. A minimum contained borated water volume of 9690 gallons,
  2. A minimum boron concentration of 2000 ppm, and
  3. A minimum solution temperature of 35°F.

APPLICABILITY: MODES 5 and 6.

ACTION:

With no borated water source OPERABLE, suspend all operations involving CORE ALTERATIONS or positive reactivity changes until at least one borated water source is restored to OPERABLE status.

SURVEILLANCE REQUIREMENTS

---

4.1.2.7 The above required borated water source shall be demonstrated OPERABLE:

- a. At least once per 7 days by:
  1. Verifying the boron concentration of the water,
  2. Verifying the contained borated water volume, and
  3. Verifying the boric acid storage tank solution temperature when it is the source of borated water.
- b. At least once per 24 hours by verifying the RWST temperature when it is the source of borated water and the outside air temperature is < 35°F.

## POWER DISTRIBUTION LIMITS

### ACTION: (Continued)

- c) Surveillance testing of the APDMS may be performed pursuant to Specification 4.3.3.7.1 provided the indicated AFD is maintained within the limits of Figure 3.2-1. A total of 6 hours of operation may be accumulated with the AFD outside of the target band during this testing without penalty deviation.
- b. THERMAL POWER shall not be increased above 81% of RATED THERMAL POWER unless the indicated AFD is within the +5% target band and ACTION 2.a) 1), above has been satisfied.
- c. THERMAL POWER shall not be increased above 50% of RATED THERMAL POWER unless the indicated AFD has not been outside of the +5% target band for more than 1 hour penalty deviation cumulative during the previous 24 hours.
- d. During power reductions using control rods, the reporting requirements of Specification 6.9.1.9 shall not apply provided the action items above are satisfied.

## SURVEILLANCE REQUIREMENTS

---

4.2.1.1 The indicated AXIAL FLUX DIFFERENCE shall be determined to be within its limits during POWER OPERATION above 15% of RATED THERMAL POWER by:

- a. Monitoring the indicated AFD for each OPERABLE excore channel:
  1. At least once per 7 days when the AFD Monitor Alarm is OPERABLE, and
  2. At least once per hour for the first 24 hours after restoring the AFD Monitor Alarm to OPERABLE status.
- b. Monitoring and logging the indicated AXIAL FLUX DIFFERENCE for each OPERABLE excore channel at least once per hour for the first 24 hours and at least once per 30 minutes thereafter, when the AXIAL FLUX DIFFERENCE Monitor Alarm is inoperable. The logged values of the indicated AXIAL FLUX DIFFERENCE shall be assumed to exist during the interval preceding each logging.

## ELECTRICAL POWER SYSTEMS

### SURVEILLANCE REQUIREMENTS (Continued)

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2. The pilot cell specific gravity, corrected to 77°F and full electrolyte level, is  $\geq 1.200$ ,
  3. The pilot cell voltage is  $\geq 2.10$  volts, and
  4. The overall battery voltage is  $\geq 250$  volts.
- b. At least once per 92 days by verifying that:
1. The voltage of each connected cell is  $\geq 2.10$  volts under float charge and has not decreased more than 0.05 volts from the value observed during the original acceptance test,
  2. The specific gravity, corrected to 70°F and full electrolyte level, of each connected cell is  $\geq 1.200$  and has not decreased more than 0.03 from the value observed during the previous test, and
  3. The electrolyte level of each connected cell is between the minimum and maximum level indication marks.
- c. At least once per 18 months by verifying that:
1. The cells, cell plates and battery racks show no visual indication of physical damage or abnormal deterioration.
  2. The cell-to-cell and terminal connections are clean, tight, and coated with anti-corrosion material.
  3. The battery charger will supply at least 140 amperes at  $\geq 250$  volts for at least 4 hours.
- d. At least once per 18 months, during shutdown, by verifying that the battery capacity is adequate to supply and maintain in OPERABLE status the emergency loads for the specified times of Table 4.8-1A with the battery charger disconnected. The battery terminal voltage shall be maintained  $\geq 210$  volts throughout the entire test.
- e. At least once per 60 months, during shutdown, by verifying that the battery capacity is at least 80% of the manufacturer's rating when subjected to a performance discharge test. This performance discharge test shall be performed subsequent to the satisfactory completion of the required battery service test.

TABLE 4.8-1A  
BATTERY EMERGENCY LOADS

<u>AB Battery Loads</u>	<u>Minimum Time</u>
1. Channel III static inverter*	3 hrs
2. Channel IV static inverter*	3 hrs
3. Computer static inverter*	3 hrs
4. BOP static inverter*	3 hrs
5. Feed pump turbine 2E oil pump	1 hr
6. Control room emergency lighting	8 hrs
7. Main turbine oil pump "E"	3 hrs
8. Isolation valve control	8 hrs
9. All control circuits	8 hrs
<u>CD Battery Loads</u>	
1. Channel I static inverter*	3 hrs
2. Channel II static inverter*	3 hrs
3. Feed pump turbine 2W oil pump	1 hr
4. Generator seal oil pump	5 hrs
5. Main turbine oil pump "W"	3 hrs
6. Isolation valves	8 hrs
7. Annunciators	8 hrs
8. All control circuits	8 hrs

\* AC power sources to the inverters shall be turned off at the start of the test and may be turned on at the end of the specified time interval. Inverters may be left in this operating mode for the duration of the discharge test.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 52 TO FACILITY OPERATING LICENSE NO. DPR-58  
AND AMENDMENT NO. 37 TO FACILITY OPERATING LICENSE NO. DPR-74

INDIANA AND MICHIGAN ELECTRIC COMPANY  
DONALD C. COOK NUCLEAR PLANT UNIT NOS. 1 AND 2  
DOCKET NOS. 50-315 AND 50-316

I. Introduction

By letter dated May 25, 1981 (AEP:NRC:372) Indiana and Michigan Electric Company (the licensee) submitted a Technical Specification change request for the Donald C. Cook Nuclear Plant Unit Nos. 1 and 2. The request would change the temperature requirements for the refueling water storage tank make several editorial changes. Specifically, the letter requested seven changes as listed below:

Change No. 1 (Editorial)

Unit No. 1 - Page 3/4 8-8 - Section 3.8.2.3.

Under the definitions of the two D. C. bus trains, the second one should be changed from AB to CD.

Change No. 2 (Editorial)

Unit No. 2 - Pages 3/4 8-9 and 3/4 8-10; Section 4.8.2.3.2d

In item (d), the table number for battery emergency loads must be changed to 4.8-1A. This revision is due to the fact that currently there are two tables numbered "Table 4.8.1."

Change No. 3 (Editorial)

Unit 1; Page 1-7, Table 1.2

Unit 2; Page 1-8, Table 1.2

In Table 1.2 of both units, frequency notations of the surveillance scheduling will be specified as a specific number of days whenever the frequency is greater than 24 hours. The words "6 months" and "18 months" will be revised to read "184 days" and "549 days" respectively. This terminology conforms to ASME boiler and pressure vessel code, Section III.

Change No. 4

Unit 1; page 3/4 2-2; Section 3.2.1  
Unit 2; page 3/4 2-2; Section 3.2.1

As presently written, Specification 3.2.1 requires that a Licensee Event Report (LER) be submitted whenever the Axial Flux Difference (AFD) exceeds the target AFD by more than +5% regardless of the cumulative penalty deviation. In Section 3.2.1 of the Bases it is clearly stated that the use of control rods can be expected, in the majority of instances, to cause the actual AFD to differ from the target AFD by more than +5%.

It is our understanding that the intent of Specification 3.2.1 is to require LER submittal only in those instances when the cumulative penalty deviation exceeds the allowable values specified in the action statement of the specification. However, our present Specification 3.2.1 does not reflect this understanding. In order to clarify the requirements of the specification and to minimize submittal of unnecessary LERs, we are requesting that a Section 3.2.1.d be added to the existing AFD specification. The revised Technical Specification pages reflect this proposed change.

Change No. 5 (Editorial)

Unit 2; page 3/4 1-11; Section 4.1.2.3

The centrifugal charging pumps, when tested on recirculation flow during Modes 5 and 6, must develop a discharge pressure 2390 psig. The 15 psi pressure difference between discharge pressures of the pump during Modes 5 and 6 and during Modes 1-4 is due to a difference in the Volume Control Tank (VCT) pressure in those modes. This was reflected in the Unit 1 Technical Specifications earlier and is now being revised accordingly, in the Unit 2 Technical Specification.

Change No. 6 (The review is not completed in this item and it will be included in a later amendment).

Change No. 7

Unit 1; page 3.4 1-15; Section 3.1.2.7 and 4.1.2.7  
Unit 2; page 3/4 1-15; Section 3.1.2.7 and 4.1.2.7

This change involves the refueling water storage tank borated water solution minimum temperature for Modes 5 and 6. Modes 1, 2, 3, and 4 are covered by TS 3.1.2.8 and 3.5.5.

The licensee has indicated that Change Nos. 1, 2, 3, and 5 are editorial. We have reviewed these changes and agree that they are editorial. In our review we found that Change No. 4 is also editorial in nature. These changes will not be discussed further.

## II. Evaluation

### Change No. 7

The Technical Specifications require the refueling water storage tank borated water solution temperature be maintained at 70°F or above in Unit 1 and 80°F or above in Unit 2, when it is to be used as a borated water source in Modes 5 and 6. This temperature can be very difficult to maintain when adding cool water to the RWST by blending following draining of the tank for refueling purposes.

Since the present solution temperature limits are derived from emergency core cooling requirements, this high temperature limit should only apply in Modes 1, 2, 3 and 4 which is covered by Technical Specifications 3.1.2.8 and 3.5.5. The licensee therefore requested that the minimum temperature limit for Modes 5 and 6 be returned by its original value of 35°F.

We have reviewed the licensee's request and agree. During Modes 5 and 6, the proposed 35°-45° reduction in RWST temperature would only effect boron dilution events (cold shutdown). The staff has under review the matter of administrative measures to be taken against boron dilution events while the plant is shutdown, and their acceptability. We will require implementation of the findings of the staff evaluation of the potential for boron dilution events. Based on our review we find the proposed change acceptable.

## III. Environmental Consideration

We have determined that the amendments do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendments involve an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR 51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of these amendments.

## IV. Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendments do not involve a significant increase in the probability or consequences of accidents previously considered and do not involve a significant decrease in a safety margin, the amendments do not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Date: February 10, 1982

UNITED STATES NUCLEAR REGULATORY COMMISSION  
DOCKET NOS. 50-315 AND 50-316  
INDIANA AND MICHIGAN ELECTRIC COMPANY  
NOTICE OF ISSUANCE OF AMENDMENTS TO FACILITY  
OPERATING LICENSES

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 52 to Facility Operating License No. DPR-58, and Amendment No. 37 to Facility Operating License No. DPR-74 issued to Indiana and Michigan Electric Company (the licensee), which revised Technical Specifications for operation of Donald C. Cook Nuclear Plant, Unit Nos. 1 and 2 (the facilities) located in Berrien County, Michigan. The amendments are effective as of the date of issuance.

The amendments change the temperature requirements for the refueling water storage tank and make several editorial changes.

The application for the amendments complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments. Prior public notice of these amendments was not required since the amendments do not involve a significant hazards consideration.

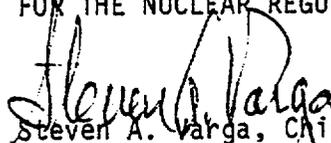
The Commission has determined that the issuance of these amendments will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of these amendments.

- 2 -

For further details with respect to this action, see (1) the application for amendments dated May 26, 1981, (2) Amendment Nos. 52 and 37 to License Nos. DPR-58 and DPR-74, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C. and at the Maude Reston Palenske Memorial Library, 500 Market Street, St. Joseph, Michigan 49085. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 10th day of February, 1982.

FOR THE NUCLEAR REGULATORY COMMISSION

  
Steven A. Varga, Chief  
Operating Reactors Branch #1  
Division of Licensing