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Docket Nos, 50-315 and 50-316

Mr. John Dolan, Vice President Indiana and Michigan Electric Company c/o American Electric Power Service Corporation 1 Riverside Plaza Columbus, Ohio 43216

Dear Mr. Dolan:

SUBJECT: DONALD C. COOK NUCLEAR PLANT, UNIT NOS. 1 and 2

DEC 16 1983

The Commission has issued the enclosed Order revising certain of the implementation dates for post-TMI related items set forth in NUREG-0737 for the Donald C. Cook Nuclear Plant. This revised Order is based on extension requests contained in your letters of May 27, 1983 (2), June 30, 1983, and October 14, 1983 and establishes the new completion dates for three post-TMI related items: II.D.1.2 Relief Valve and Safety Valve Test Program; II.F.1 Attachment 1 Noble Gas Effluent Monitor; and II.F.1 Attachment 2 Sampling and Analysis of Plant Effluents (Iodine Monitors).

The Commission's intention when it issued NUREG-0737 was that items would be completed in accordance with the staff's recommended schedule. However, our evaluation of your proposed schedule exceptions concludes that the proposed delays are acceptable.

A copy of the Order is being filed with the Office of the Federal Register for publication.

Sincerely,

Diginal signed by: S. A. Varga

Steven A. Varga, Chief Operating Reactors Branch No. 1 Division of Licensing

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Docket File ORB#1 NRC PDR Local PDR C Parrish D Wigginton D Eisenhut H Denton OELD (BURNS) SECY E JOrdan J Tavlor T Barnhart (8) ACRS (10) D BRinkman R Diggs OPA CMILES ORAB

Docket Nos. 50-315 and 50-316

Mr. John Dolan, Vice President Indiana and Michigan Electric Company c/o American Electric Power Service Corporation] Riverside Plaza Columbus, Ohio 43216

Dear Mr. Dolan:

SUBJECT: DONALD C. COOK NUCLEAR PLANT, UNIT NOS. 1 AND 2

The Commission has issued the enclosed Order revising certain of your commitments to implement certain post-TMI related items set forth in NUREG-0737 for the Donald C. Cook Nuclear Plant. This revised Order is based on commitments contained in your letters of May 27, 1983 (2), June 30, 1983, and October 14, 1983 and establishes the new completion dates for three post-TMI related items: II.D.1.2 Relief Valve and Safety Valve Test Program; II.F.1 Attachment 1 Noble Gas Effluent Monitor; and and II.F.1 Attachment 2 Sampling and Analysis of Plant Effluents (Iodine Monitors).

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Enclosure:

OFFICIAL RECORD COPY

Indiana and Michigan Electric Company

cc: Mr. M. P. Alexich Vice President Nuclear Engineering American Electric Power Service Corporation 1 Riverside Plaza Columbus, Ohio 43215

> Mr. William R. Rustem (2) Office of the Governor Room 1 - Capitol Building Lansing, Michigan 48913

Mr. Wade Schuler, Supervisor Lake Township Baroda, Michigan 49101

W. G. Smith, Jr., Plant Manager Donald C. Cook Nuclear Plant Post Office Box 458 Bridgman, Michigan 49106

U.S. Nuclear Regulatory Commission Resident Inspectors Office 7700 Red Arrow Highway Stevensville, Michigan 49127

Gerald Charnoff, Esquire Shaw, Pittman, Potts and Trowbridge 1800 M Street, N.W. Washington, DC 20036

Honorable Jim Catania, Mayor City of Bridgman, Michigan 49106

U.S. Environmental Protection Agency Region V Office ATTN: EIS COORDINATOR 230 South Dearborn Street Chicago, IL 60604

Maurice S. Reizen, M.D. Director Department of Public Health Post Office Box 30035 Lansing, Michigan 48109 Donald C. Cook Nuclear Plant, Units 1 and 2

The Honorable Tom Corcoran United States House of Representatives Washington, DC 20515

James G. Keppler Regional Administrator - Region III U.S. Nuclear Regulatory Commission 799 Roosevelt Road Glen Ellyn, IL 60137

UNITED STATES OF AMERICA

NUCLEAR REGULATORY COMMISSION

In the Matter of

Docket Nos. 50-315 and 50-316

INDIANA AND MICHIGAN ELECTRIC COMPANY

(Donald C. Cook Nuclear Plant Unit Nos. 1 and 2)

MODIFICATION OF ORDER CONFIRMING LICENSEE COMMITMENTS ON POST-TMI RELATED ISSUES

I.

Indiana and Michigan Electric Company (the licensee) is the holder of Facility Operating License Nos. DPR-58 and DPR-74 which authorizes the operation of the Donald C. Cook Nuclear Plant, Unit Nos. 1 and 2 (the facilities) at steady-state power levels not in excess of 3350 and 3391 megawatts thermal, respectively, for each unit. The facilities consist of pressurized water reactors (PWRs) located at the licensee's site in Barrien County, Michigan.

II.

Following the accident at Three Mile Island Unit No. 2 (TMI-2) on March 28, 1979, the Nuclear Regulatory Commission (NRC) staff developed a number of proposed requirements to be implemented on operating reactors and on plants under construction. These requirements include Operational Safety, Siting and Design, and Emergency Preparedness and are intended to provide substantial additional protection in the operation of nuclear facilities based on the experience from the accident at TMI-2 and the official studies and



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investigations of the accident. The staff's proposed requirements and schedule for implementation were set forth in NUREG-0737, "Clarification of TMI Action Plan Requirements." Among these requirements are a number of items, consisting of hardware modifications, administrative procedure implementation and specific information to be submitted by the licensee. The NRC issued an "Order Confirming Licensee Commitments on Post-TMI Related Issues," dated March 14, 1983, for Donald C. Cook, Unit Nos. 1 and 2 for a number of these items.

III.

By letters dated May 27, 1983 (2), June 30, 1983, and October 14, 1983, Indiana and Michigan Electric Company requested relief from the dates imposed by the said Order for three items for Donald C. Cook, Unit Nos. 1 and 2. The specific items are II.D.1.2, Relief Valve and Safety Valve Test Programs; II.F.1 Attachment 1, Noble Gas Effluent Monitors; and II.F.1 Attachment 2, Sampling and Analysis of Plant Effluents. Indiana and Michigan Electric Company requested that these completion dates be changed as follows:

II.D.1.2 from July 1, 1983 to December 15, 1983 II.F.1.1 from May 1983 to End of Unit 1 1983 Outage II.F.1.2 from April 1983 to One Week after Unit 2 Return to Power July 1983

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The staff's evaluation of the licensee's request for revised completion dates is provided herein:

II.D.1.2 Relief Valve and Safety Valve Program

In the May 27, 1983 letter (Ref. AEP:NRC:05 85F), the licensee provided notification that their analysis revealed that the discharge piping system for the pressurizers would be overloaded and the piping/supports will be stressed beyond the allowable stress limits. To eliminate the concern, the licensee proposed to drain the water from the safety valve loop seals. The preliminary analysis indicates that thermal hydraulic forces are substantially lower for the case where there is no water in the loop seals. The loop seals will remain drained by the addition of a drain pipe from the loop seals back to the pressurizer. The qualification of the as-built and modified piping system will be performed without the loop seals water and, if necessary, the supports for the down stream piping will be modified. This qualification and modification effort will require additional work which was not factored into the previous schedule. The staff believes the results of the analysis of the additional work should be presented in the Plant Specific Evaluation of the Relief and Safety Valves. The proposed submission date for the Evaluation of December 15, 1983, or to the end of the calendar year is acceptable.

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II.F.1, Attachment 1, Noble Gas Effluent Monitor

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II.F.1, Attachment 2, Sampling and Analysis of Plant Effluents

In the May 27, 1983 letter (Ref. AEP:NRC:0678F) and in the June 30, 1983 letter (Ref. AEP:NRC:687G), the licensee reported continuing environmental interface problems with the design and location of noble gas effluent monitors. The installation and associated efforts were to be accomplished during the planned outages for each Unit; Unit No. 1 was to be shutdown for refueling and Unit No. 2 was scheduled for a Technical Specification surveillance outage. Unit No. 2, however, was forced to shutdown ahead of schedule and the licensee was not prepared to complete the activities as planned. The licensee has now completed the noble gas effluent monitor installation, testing, and calibration on both Units. The iodine monitors were essentially completed but since the last efforts require the unit to be operating, the Unit No. 2 iodine monitors were rescheduled to be complete about one week after the unit returned to power; i.e., July 14, 1983. The licensee has subsequently indicated that the calibration and adjustment of iodine monitors on Unit No. 2 was completed on July 9, 1983. We have reviewed the delay in installation of these modifications and the augmented interim instrumentation and procedures that were in place for monitoring the effluents from the plant in the event of an accident.

- 5 -

We have concluded that the licensee made reasonable progress toward installation of the monitors and modifications and that the interim measures that had been established were adequate to permit continued operation of the Donald C. Cook Nuclear Plant, Unit Nos. 1 and 2.

We find, based on the above evaluation, that: (1) the licensee has taken corrective actions regarding the delays and has made a responsible effort to implement the NUREG-0737 requirements noted; (2) there was good cause for the several delays (interface problems and unscheduled unit outages); and (3) as noted above, interim compensatory measures were provided.

IV.

Accordingly, pursuant to Sections 103, 161i, and 161o of the Atomic Energy Act of 1954, as amended, and the Commission's regulations in 10 CFR Parts 2 and 50, IT IS HEREBY ORDERED THAT THE LICENSEE SHALL:

Implement and maintain post-TMI related Item II.D.1.2 no later than December 31, 1983, and maintain Items II.F.1.1 and II.F.1.2 (now complete) as described in the licensee's submittals noted in Section III herein.

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The licensee may request a hearing on this Order within 20 days of the date of publication of this Order in the <u>Federal Register</u>. Any request for a hearing shall be addressed to the Director, Office of Nuclear Reactor Regulation, U. S. Nuclear Regulatory Commission, Washington, D. C. 20555. A copy shall also be sent to the Executive Legal Director at the same address. If a hearing is to be held, the Commission will issue an Order designating the time and place of any such hearing.

If a hearing is held concerning this Order, the issue to be considered at the hearing shall be whether the licensee should comply with the requirements set forth in Section IV of this Order. This Order shall become effective upon the licensee's consent or upon expiration of the time within which the licensee may request a hearing or, if a hearing is requested by the licensee, on the date specified in an order following further proceedings on this Order.

FOR THE NUCLEAR REGULATORY COMMISSION

Division of Licensing

Office of Nuclear Reactor Regulation

Dated at Bethesda, Maryland this 16th day of December , 1983.