

Distribution
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 ACRS -10
 OPA C Miles
 R Diggs
 L J Harman
 E Jordan
 J Taylor
 W Jones
 D Brinkman
 OELD

November 23, 1983

Docket Nos. 50-315
 and 50-316

Mr. John Dolan, Vice President
 Indiana and Michigan Electric Company
 c/o American Electric Power Service Corporation
 1 Riverside Plaza
 Columbus, Ohio 43216

Dear Mr. Dolan:

The Commission has issued the enclosed Amendment No. 77 to Facility Operating License No. DPR-58 and Amendment No. 58 to Facility Operating License No. DPR-74 for the Donald C. Cook Nuclear Plant, Unit Nos. 1 and 2, respectively. The amendments consist of changes to the Technical Specifications in response to your application transmitted by letter dated April 8, 1983.

These amendments revise the Technical Specifications to limit overtime worked by plant staff members in accordance with Generic Letter No. 82-12 issued June 15, 1982 and would require an annual reporting of all challenges to the pressurizer power operated relief valves or safety valves.

A copy of the related Safety Evaluation is enclosed. The Notice of Issuance will be included in the Commission's next regular monthly Federal Register notice.

Sincerely,

David L. Wigginton, Project Manager
 Operating Reactors Branch No. 1
 Division of Licensing

Enclosures:

1. Amendment No. 77 to DPR-58
2. Amendment No. 58 to DPR-74
3. Safety Evaluation

cc w/enclosures:
 See next page

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 PDR ADOCK 05000315
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***SEE PREVIOUS CONCURRENCE

OFFICE	ORB#1****	ORB#1***	ORB#1****	AD/OR	OELD***		
SURNAME	CParrish	DWigginton;ef	SVarga	GLainas			
DATE	11/14/83	11/15/83	11/15/83	11/17/83	11/17/83		

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DATE	11/14/83	11/15/83	11/15/83	11/15/83	11/17/83	11/17/83	

Indiana and Michigan Electric Company

CC: Mr. M. P. Alexich
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American Electric Power
Service Corporation
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U. S. Nuclear Regulatory Commission
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Stevensville, Michigan 49127

Gerald Charnoff, Esquire
Shaw, Pittman, Potts and Trowbridge
1800 M Street, N.W.
Washington, D. C. 20036

Honorable Jim Catania, Mayor
City of Bridgman, Michigan 49106

U.S. Environmental Protection Agency
Region V Office
ATTN: EIS COORDINATOR
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Chicago, Illinois 60604

Maurice S. Reizen, M.D.
Director
Department of Public Health
P.O. Box 30035
Lansing, Michigan 48109

The Honorable Tom Corcoran
United States House of Representatives
Washington, D. C. 20515

James G. Keppler
Regional Administrator - Region III
U. S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, Illinois 60137

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

INDIANA AND MICHIGAN ELECTRIC COMPANY

DOCKET NO. 50-315

DONALD C. COOK NUCLEAR PLANT UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 77
License No. DPR-58

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Indiana and Michigan Electric Company (the licensee) dated April 8, 1983, complies with the standards and requirements of the Atomic Energy Act of 1954 as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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PDR

2. Accordingly, the licensee is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-58 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 77, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. The change in Technical Specification is to become effective within 30 days of issuance of the amendment. In the period between issuance of the amendment and the effective date of the new Technical Specifications, the licensee shall adhere to the Technical Specification for the systems, components, or operation existing at the time. The period of time between changeover of systems, components, or operation shall be minimized or compensated for by suitable temporary alternatives.
4. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Steven A. Varga, Chief
Operating Reactors Branch No. 1
Division of Licensing

Enclosure:
Changes to the Technical
Specifications

Date of issuance: November 23, 1983



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

INDIANA AND MICHIGAN ELECTRIC COMPANY

DOCKET NO. 50-316

DONALD C. COOK NUCLEAR PLANT UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 58
License No. OPR-74

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Indiana and Michigan Electric Company (the licensee) dated April 8, 1983, complies with the standards and requirements of the Atomic Energy Act of 1954 as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the licensee is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-74 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 58, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. The change in Technical Specification is to become effective within 30 days of issuance of the amendment. In the period between issuance of the amendment and the effective date of the new Technical Specifications, the licensee shall adhere to the Technical Specification for the systems, components, or operation existing at the time. The period of time between changeover of systems, components, or operation shall be minimized or compensated for by suitable temporary alternatives.
4. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Steven A. Varga, Chief
Operating Reactors Branch No. 1
Division of Licensing

Enclosure:
Changes to the Technical
Specifications

Date of issuance: November 23, 1983

ATTACHMENT TO LICENSE AMENDMENTS

AMENDMENT NO. 77 TO FACILITY OPERATING LICENSE NO. DPR-58

AMENDMENT NO. 58 TO FACILITY OPERATING LICENSE NO. DPR-74

DOCKET NOS. 50-315 AND 50-316

Revise Appendix A as follows:

Remove Pages

Insert Pages

Unit 1

6-1
6-2
6-15

6-1
6-2*
6-15

Unit 2

6-1
6-2
6-15

6-1
6-2*
6-15

*Included for convenience only

6.0 ADMINISTRATIVE CONTROLS

6.1 RESPONSIBILITY

6.1.1 The Plant Manager shall be responsible for overall facility operation and shall delegate in writing the succession to this responsibility during his absence.

6.2 ORGANIZATION

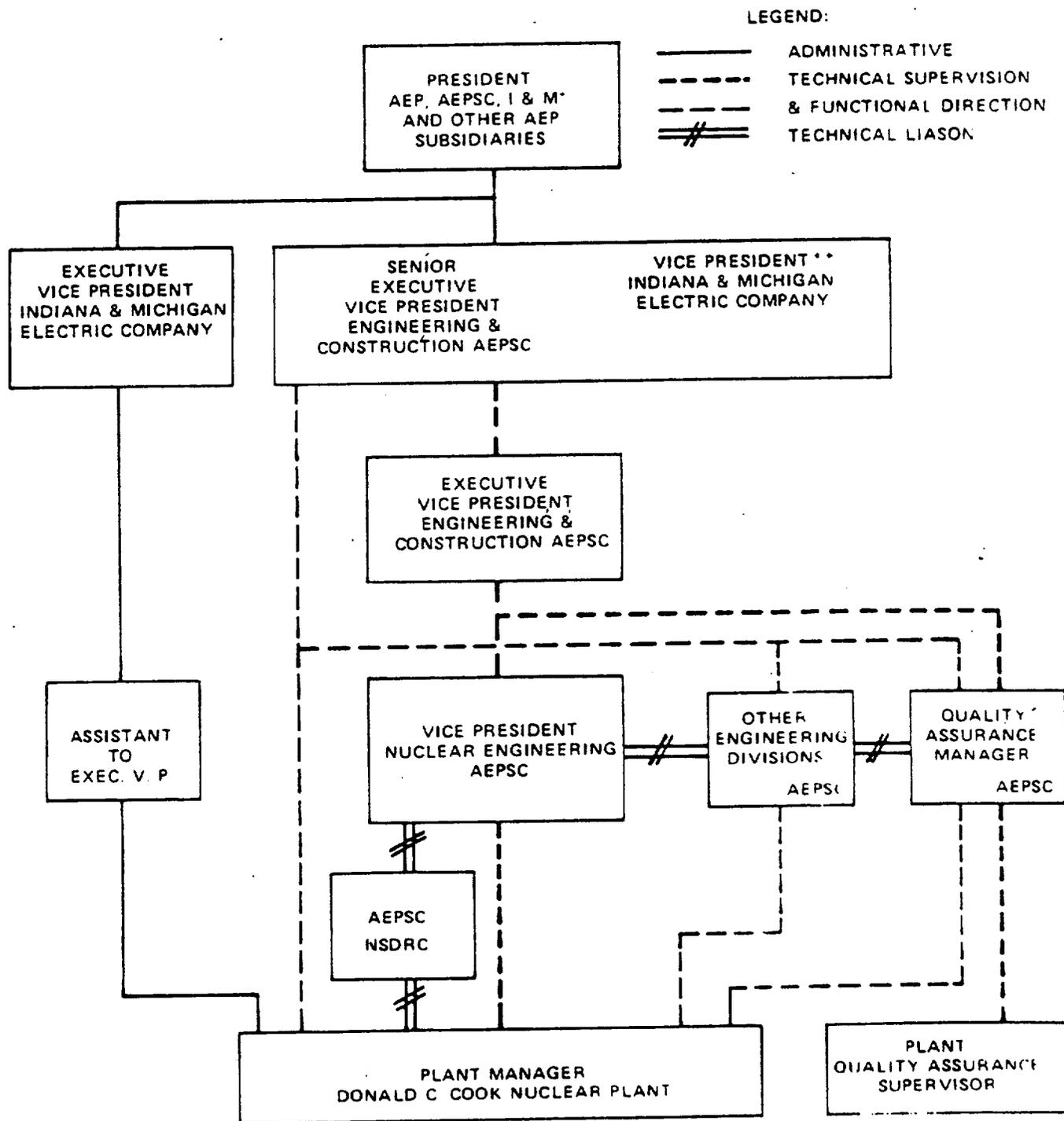
OFFSITE

6.2.1 The offsite organization for facility management and technical support shall be as shown on Figure 6.2-1.

FACILITY STAFF

6.2.2 The Facility organization shall be as shown on Figure 6.2-2 and:

- a. Each on duty shift shall be composed of at least the minimum shift crew composition shown in Table 6.2-1.
- b. At least one licensed Operator shall be in the control room when fuel is in the reactor.
- c. At least two licensed Operators shall be present in the control room during reactor start-up, scheduled reactor shutdown and during recovery from reactor trips.
- d. An individual qualified in radiation protection procedures shall be on site when fuel is in the reactor.
- e. ALL CORE ALTERATIONS after the initial fuel loading shall be directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation.
- f. A site Fire Brigade of at least 5 members shall be maintained onsite at all times. The fire Brigade shall not include 3 members of the minimum shift crew necessary for safe shutdown of the unit or any personnel required for other essential functions during a fire emergency.
- g. The amount of overtime worked by plant staff members performing safety-related functions must be limited in accordance with NRC Policy Statement on working hours (Generic Letter No. 82-12).



* INDIANA AND MICHIGAN ELECTRIC COMPANY

** Responsible for Fire Protection Program

FIGURE 6.2 1 Offsite Organization for Facility Management and Technical Support

ADMINISTRATIVE CONTROLS

power operation), supplementary reports shall be submitted at least every three months until all three events have been completed.

ANNUAL REPORTS^{1/}

6.9.1.4 Annual reports covering the activities of the unit as described below for the previous calendar year shall be submitted prior to March 1 of each year. The initial report shall be submitted prior to March 1 of the year following initial criticality.

6.9.1.5 Reports required on an annual basis shall include:

- a. A tabulation on an annual basis of the number of station, utility and other personnel (including contractors) receiving exposures greater than 100 mrem/yr and their associated man rem exposure according to work and job functions,^{2/} e.g., reactor operations and surveillance, inservice inspection, routine maintenance, special maintenance (describe maintenance), waste processing, and refueling. The dose assignment to various duty functions may be estimates based on pocket dosimeter, TLD, or film badge measurements. Small exposures totalling less than 20% of the individual total dose need not be accounted for. In the aggregate, at least 80% of the total whole body dose received from external sources shall be assigned to specific major work functions.
- b. The complete results of steam generator tube inservice inspections performed during the report period (reference Specification 4.4.5.5.b).
- c. Documentation of all challenges to the pressurizer power operated relief valves (PORVs) or safety valves.

¹ A single submittal may be made for a multiple unit station. The submittal should combine those sections that are common to all units at the station.

² This tabulation supplements the requirements of 20.407 of 10 CFR Part 20.

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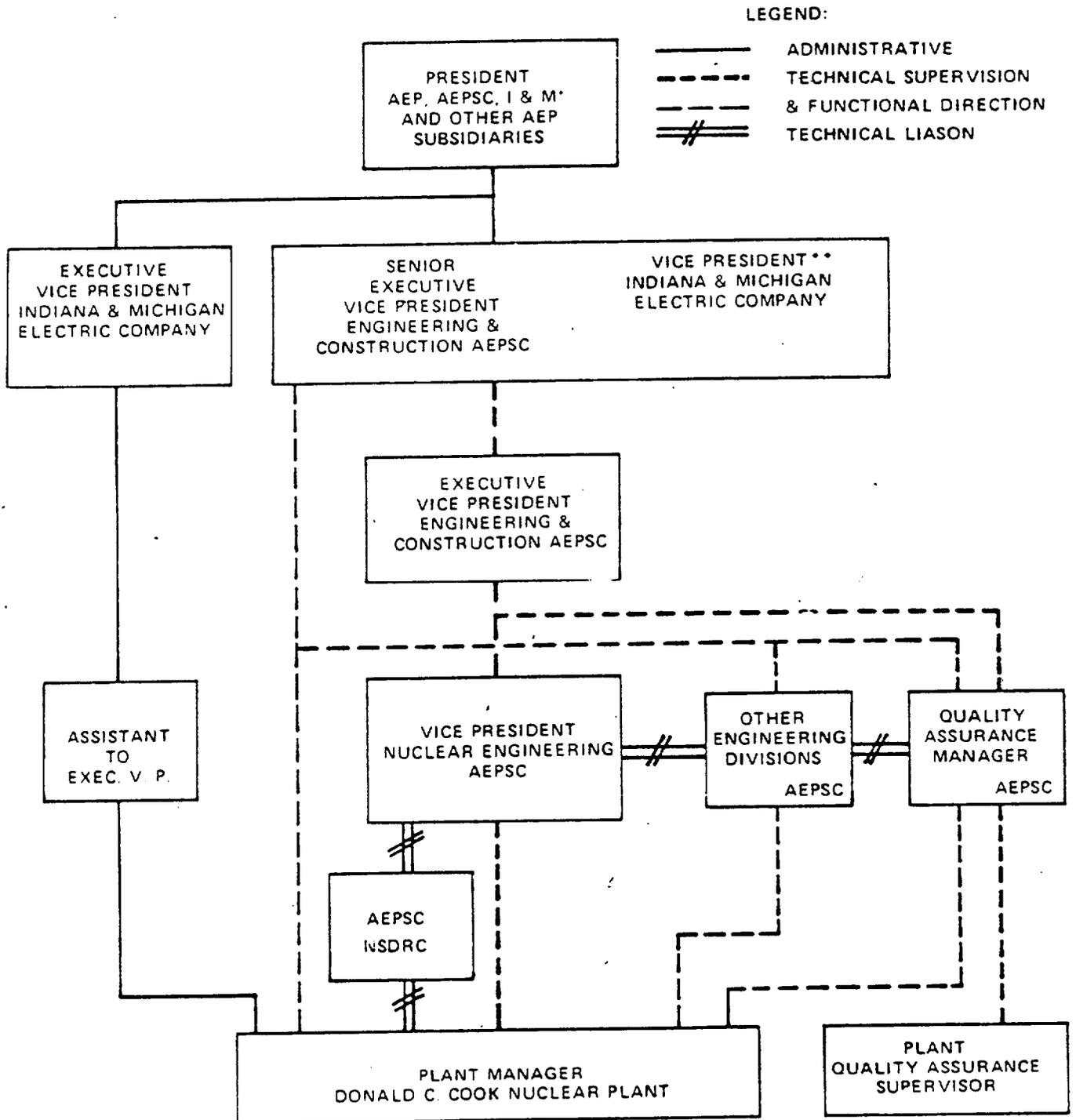
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- e. ALL CORE ALTERATIONS shall be directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation.
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*INDIANA AND MICHIGAN ELECTRIC COMPANY

**Responsible for Fire Protection Program

FIGURE 6.2-1 Offsite Organization for Facility Management and Technical Support

ADMINISTRATIVE CONTROLS

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- b. The complete results of steam generator tube inservice inspections performed during the report period (reference Specification 4.4.5.5.b).
- c. Documentation of all challenges to the pressurizer power operated relief valves (PORVs) or safety valves.

¹ A single submittal may be made for a multiple unit station. The submittal should combine those sections that are common to all units at the station.

² This tabulation supplements the requirements of 20.407 of 10 CFR Part 20.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 77 TO FACILITY OPERATING LICENSE NO. DPR-58
AND AMENDMENT NO. 58 TO FACILITY OPERATING LICENSE NO. DPR-74
INDIANA AND MICHIGAN ELECTRIC COMPANY
DONALD C. COOK NUCLEAR PLANT UNIT NOS. 1 AND 2
DOCKET NOS. 50-315 AND 50-316

I. Introduction

A. Background Information

Following the accident at Three Mile Island, Unit 2, the staff developed an NRC Action Plan, NUREG-0660, to provide a comprehensive and integrated plan to improve safety at power reactors. Specific NUREG-0660 items, approved by the Commission for implementation at power reactors, were issued as NUREG-0737. NUREG-0737 specified that new Technical Specifications would be required for several of the items. Accordingly, on September 20, 1982, the NRC issued Generic Letter 82-16, requesting information on the following NUREG-0737 items:

1. STA Training (I.A.1.1.3).
2. Shift Manning - Overtime Limits (I.A.1.3.1).
3. Short Term Auxiliary Feedwater System (AFWS) Evaluation (II.E.1.1).
4. Safety Grade AFW Initiation and Flow Indication (II.E.1.2).
5. Dedicated Hydrogen Penetrations (II.E.4.1).
6. Containment Pressure Setpoint (II.E.4.2.5).
7. Containment Purge Valves (II.E.4.2.6).
8. Radiation Signal on Purge Valves (II.E.4.2.7).
9. Upgrade B&W AFWS (II.K.2.8).
10. B&W Safety-Grade Anticipatory Reactor Trip (II.K.2.10).
11. B&W Thermal-Mechanical Report (II.K.2.13).
12. Reporting Safety and Relief Valve Failures and Challenges (II.K.3.3).
13. Anticipatory Trip on Turbine Trip (II.K.3.12).

B. Licensee's Response to Generic Letter 82-16

By letter dated April 8, 1983, Indiana and Michigan Electric Company (the licensee) responded to NRC Generic Letter No. 82-16, "NUREG-0737 Technical Specifications." The licensee summarized the results of the requested review. With the inclusion of proposed license amendments for I.A.1.3, Overtime Limits and II.K.3.3, Reporting Safety and Relief Valve Failures, the licensee concluded that all items have been appropriately addressed.

C. Scope of Review

The staff's review consisted of a verification of the status of each of the items and an evaluation of the proposed TSs against the model TSs provided in Generic Letter 82-16 and other guidance related to NUREG-0737. For the D.C. Cook facility, only Item 2 and 12 are evaluated in this SE. The remaining items are not being evaluated in this SE because (1) the item has not been completed at the facility, (2) the item has been previously closed out by the staff for the facility, (3) the staff position has not been finalized for the item, or (4) the item does not apply to the D.C. Cook facility. A summary of each of the items is given below.

1. STA Training (I.A.1.1.3)

Our July 2, 1980 letter provide model TSs and TMI lessons learned category "A" items. Included were TSs that specified the qualifications, training and on-duty requirements for the Shift Technical Advisors (STAs). In a previous submittal of December 10, 1980, the licensee committed to a training program for STAs. Amendments 49 (DPR-58) and 34 (DPR-74) were issued on August 25, 1981 in response to this submittal. By letter of January 8, 1982, the staff advised that their post-implementation review indicated that the licensee's STA training program was acceptable and consistent with all current NRC guidance. The STA training requirements are now under consideration by the Commission, and no action will be taken to amend the TSs until further guidance is provided by the Commission.

2. Shift Manning - Overtime Limits (I.A.1.3.1)

In response to Generic Letter 82-16 dated April 8, 1983, the licensee submitted proposed Technical Specification changes reflecting the Commission's Policy Statement on nuclear power plant staff working hours. The evaluation of this proposed licensee amendment is covered in Section II.

3. Short Term AFWS Evaluation (II.E.1.1)

The licensee made a submittal on February 22, 1980 relative to their auxiliary feedwater system reliability evaluation, and requested TS changes (Amendments 42 and 24). By letter of October 6, 1980, the staff replied, granting the requested TS changes, but indicating that some issues still remained. The open issues were further reviewed and by letter of June 16, 1981, the staff advised the licensee that the review of this item was complete, accepting the licensee's actions.

4. Safety Grade AFW Initiation and Flow Indication (II.E.1.2)

In their submittal of February 22, 1980, the licensee addressed items relative to their evaluation of the Auxiliary Feedwater System, and requested TS changes. By letter of October 6, 1980 the staff advised that the requested TS changes were acceptable, but four items in their review of the AFWS system were not complete at that time. By letter of June 16, 1981, the staff

advised that they had completed their review, and subject to the licensee's commitment to install environmentally qualified auxiliary feedwater flow transmitters, the licensee's responses were acceptable. In their response to GL 82-16 dated April 6, 1983, the licensee advised that it had accomplished this commitment by the installation of the qualified flow rate transmitters before June 5, 1981.

5. Dedicated Hydrogen Penetrations (II.E.4.1)

By letter dated March 20, 1980, the NRC staff forwarded their evaluation of the licensee's compliance with the Category "A" items of NUREG-0576, "TMI Lessons Learned". This letter documented the staff's acceptance of the D.C. Cook design in regard to dedicated hydrogen penetrations. By letter of September 9, 1981, the licensee was advised that the above review satisfied the requirements of item II.E.4.1.

6. Containment Pressure Setpoint (II.E.4.2.5).

By letter dated October 30, 1981, the staff advised the licensee of the status of their review of the related issues of containment pressure setpoint and containment purging and venting. Included with the letter was a Safety Evaluation of the D.C. Cook containment isolation pressure setpoint performed by the EG&G Energy Measurements Group, under contract to the NRC. Their evaluation found that the licensee had met the requirements of item II.E.4.2(5) and the additional guidelines developed by the NRC staff. The NRC staff concurred with the evaluation that a TS change was not necessary.

7. Containment Purge Valves (II.E.4.2.6)

In response to licensee's submittal of August 30, 1982, clarified by subsequent letters of October 29 and November 19, 1982, the staff issued Amendment Nos. 66 and 47 on December 8, 1982, relative to containment purging for safety related reasons and containment isolation valve operability. These amendments reflected the positions taken in NUREG-0737.

8. Radiation Signal on Purge Valves (II.E.4.2.7)

By letters of May 1, June 8, and December 5, 1979, and December 7, 1981; the licensee provided basic information regarding their containment isolation signal initiation. Later, on August 22, 1982, the licensee applied for a TS Change for a new Radiation Monitoring System. The NRC staff issued Amendment No. 60 and 58, dated September 9, 1982 for these TSs changes.

9. Upgrade B&W AFWS (II.K.2.8)

This item pertains to B&W facilities only, and is not applicable to this facility.

10. B&W Safety-Grade Anticipatory Reactor Trip (II.K.2.10)

This item pertains to B&W facilities only, and is not applicable to this facility.

11. B&W Thermal-Mechanical Report (II.K.2.13)

This item pertains to B&W facilities only, and is not applicable to this facility.

12. Reporting Safety and Relief Valve Failures and Changes (II.K.3.3).

By letter dated May 28, 1982, the licensee had previously committed to the reporting requirements of this TMI Action Item. In their response to Generic Letter 82-16 dated April 8, 1983, the licensee submitted proposed Technical Specifications changes covering the reporting requirements. The evaluation of this proposed license amendment is covered in Section II.

13. Anticipatory Trip on Turbine Trip (II.K.3.12)

By letter dated June 20, 1980, the licensee advised that the D.C. Cook facility has an anticipatory (reactor) trip on turbine trip as a part of the licensing basis (original plant design). By letter of September 8, 1981, the staff indicated their acceptance of this item.

II. Safety Evaluation of Licensee's Proposed License Amendment for TMI Action Items I.A.1.3.1 and II.K.3.3

By letter dated April 8, 1983, the licensee submitted proposed NUREG-0737 TS changes to the D.C. Cook Unit Nos. 1 and 2. The staff's evaluation of these proposed TSs changes are as follows:

A. Generic Letter 82-16, Item 2 - Shift Manning - Overtime Limits (I.A.1.3.1)

On June 15, 1982, the staff issued Generic Letter 82-12 which contained a revised version of the Commission's Policy Statement on nuclear power plant staff working hours. Generic Letter 82-16 requested that the administrative section of the Technical Specification be revised to implement the Commission policy. The licensee in their April 8, 1983 response to Generic Letter 82-16, made an application to amend their license incorporating the Commission's policy by referencing Generic Letter 82-12 in Section 6.2.2.g of the Technical Specification.

We have reviewed the licensee's proposed wording to incorporate the Commission policy on overtime limits and find it acceptable.

B. GL 82-16 Item 12. Reporting SV and RV Failures and Challenges
(II.K.3.3)

Generic Letter 82-16 requested that the administrative section of the Technical Specification be revised to require that any failure of the pressurizer power operated relief valves (PORV) or safety valves (SV) be promptly reported to the Commission by telephone and followed up with a written report. Additionally, all challenges to these valves are to be reported to the Commission on a monthly or annual basis.

The licensee, in their April 8, 1983 response to Generic Letter 82-16, proposed a change which will add reporting requirements to TSs Section 6.9.1.5(c) (documenting all challenges to the Pressurizer Power Operated Relief Valves (PORVs) and Safety Valves (SVs) in the Donald C. Cook annual report) and to TS Section 6.9.1.12(1) (prompt notification with written followup) for reporting failures of the pressurizer PORVs or SVs. The changes meet the proposed standard TS format; however, the changes to T.S. Section 6.9.1.12(1) are now considered to be inconsistent with the new rule 10 CFR 50.73, which is to become effective January 1, 1984. That new rule will require failures of the pressurizer PORV's and SV's, among other things, to be reported (see 48 FR 33656 Item (f) under the Licensee Event Report system. In discussion with the licensee it was agreed to delete the proposed Item "1" in T.S. Section 6.9.1.12. We find this and the licensee's proposed T.S. Section 6.9.1.5(c) acceptable.

III. Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendments involve actions which are insignificant from the standpoint of environmental impact, and, pursuant to 10 CFR 51.5(d)(4), that an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

IV. Conclusion

We have concluded, based on the considerations discussed above, that:
(1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and
(2) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Dated: November 23, 1983

Principal Contributors:

J. E. Foster
K. R. Ridgway
D. Wigginton