

July 12, 1985

Docket No. 50-315
and 50-316

Mr. John Dolan, Vice President
Indiana and Michigan Electric Company
c/o American Electric Power Service Corporation
1 Riverside Plaza
Columbus, Ohio 43216

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Dear Mr. Dolan:

SUBJECT: ORDER MODIFYING LICENSE TO CONFIRM ADDITIONAL LICENSEE
COMMITMENTS ON EMERGENCY RESPONSE CAPABILITY (SUPPLEMENT 1
TO NUREG-0737)

Re: Donald C. Cook Nuclear Plant, Unit Nos. 1 and 2

The Commission has issued the enclosed Order confirming your additional commitments on emergency response capability. This Order is based on your letters dated September 28, 1984, February 28 and April 29, 1985 committing to the schedules specified in the Order. Your letters modified your previous schedules in letters dated April 15 and August 2, 1983, January 30 and February 10, 1984. This order pertains to the schedules for the Safety Parameter Display System, the Detailed Control Room Design Review, the Upgrade of Emergency Operating Procedures and the Emergency Response Facilities. The fifth item, Regulatory Guide 1.97 Requirements, will be the subject of separate correspondence.

A copy of this Order is being filed with the Office of the Federal Register for publication.

Sincerely,

/s/SAVarga

Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Enclosure:
Order

cc w/enclosure:
See next page

*SEE PREVIOUS WHITE FOR CONCURRENCES

ORB#1:DL*
CParrish
06/24/85

ORB#1:DL
DWigginton/ts
06/28/85

BC-ORB#1:DL
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06/28/85

BC-ORB#5
JZwolinski
06/23/85

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Lieberman
07/12/85

AD:DR:DL
GLainas
06/10/85

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Mr. John Dolan, Vice President
Indiana and Michigan Electric Company
c/o American Electric Power Service Corporation
1 Riverside Plaza
Columbus, Ohio 43216

Dear Mr. Dolan:

SUBJECT: ORDER MODIFYING LICENSE CONFIRMING ADDITIONAL LICENSEE
COMMITMENTS ON EMERGENCY RESPONSE CAPABILITY (SUPPLEMENT 1
TO NUREG-0737)

Re: Donald C. Cook Nuclear Plant, Unit Nos. 1 and 2

The Commission has issued the enclosed Order confirming your additional commitments on emergency response capability. This Order is based on your letters dated September 28, 1984, February 28 and April 29, 1985 committing to the schedules specified in the Order. Your letters modified your previous schedules in letters dated April 15 and August 2, 1983, January 30 and February 10, 1984. This order pertains to the schedules for the Safety Parameter Display System, the Detailed Control Room Design Review, the Upgrade of Emergency Operating Procedures and the Emergency Response Facilities. The fifth item, Regulatory Guide 1.97 Requirements, will be the subject of separate correspondence. The priority for these tasks is clear. Any deviation from this schedule will be reflected in future SALP inputs.

A copy of this Order is being filed with the Office of the Federal Register for publication.

Sincerely,

Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Enclosure:
Order

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AD:OR:DL
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06/ / 85

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HThompson
06/ /85

Mr. John Dolan
Indiana and Michigan Electric Company Donald C. Cook Nuclear Plant

cc:

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Washington, DC 20036

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Nuclear Facilities and Environmental
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Division of Radiological Health
Department of Public Health
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Lansing, Michigan 48909

The Honorable John E. Grotberg
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J. Feinstein
American Electric Power
Service Corporation
1 Riverside Plaza
Columbus, Ohio 43216

UNITED STATES OF AMERICA
NUCLEAR REGULATORY COMMISSION

| | | |
|-------------------------------------|---|--------------------|
| In the Matter of | } | |
| INDIANA & MICHIGAN ELECTRIC COMPANY | } | Docket Nos. 50-315 |
| | } | and 50-316 |
| (D. C. Cook Nuclear Plant | } | |
| Unit Nos. 1 and 2) | } | |

ORDER MODIFYING LICENSE TO CONFIRM ADDITIONAL LICENSEE COMMITMENTS
ON EMERGENCY RESPONSE CAPABILITY

I.

Indiana and Michigan Electric Company (IMEC) (the licensee) is the holder of Facility Operating License Nos. DPR-58 and DPR-74 which authorizes the operation of the Donald C. Cook Nuclear Plant, Unit Nos. 1 and 2 (the facilities) at steady-state power levels not in excess of 3250 and 3411 megawatts thermal, respectively. The facilities are pressurized water reactors (PWRs) located in Berrien County, Michigan.

II.

Following the accident at Three Mile Island Unit No. 2 (TMI-2) on March 28, 1979, the Nuclear Regulatory Commission (NRC) staff developed a number of proposed requirements to be implemented on operating reactors and on plants under construction. These requirements include Operational Safety, Siting and Design, and Emergency Preparedness and are intended to provide substantial additional protection in the operation of nuclear facilities and significant upgrading of emergency response capability based on the experience from the accident at TMI-2 and the official studies and investigations of the

accident. The requirements are set forth in NUREG-0737, "Clarification of TMI Action Plan Requirements," and in Supplement 1 to NUREG-0737, "Requirements for Emergency Response Capability." Among these requirements are a number of items consisting of emergency response facility operability, emergency procedure implementation, addition of instrumentation, possible control room design modification, and specific information to be submitted.

On December 17, 1982, a letter (Generic Letter 82-33) was sent to all licensees of operating reactors, applicants for operating licenses, and holders of construction permits enclosing Supplement 1 to NUREG-0737. In this letter operating reactor licensees and holders of construction permits were requested to furnish the following information, pursuant to 10 CFR 50.54(f), no later than April 15, 1983:

- (1) A proposed schedule for completing each of the basic requirements for the items identified in Supplement 1 to NUREG-0737, and
- (2) A description of plans for phased implementation and integration of emergency response activities including training.

III.

IMEC responded to Generic Letter 82-33 by letter dated April 15, 1983. By letters dated August 2, 1983, January 30, 1984, and February 10, 1984, IMEC provided additional information and dates as a result of negotiations with the NRC staff. In these submittals, IMEC made commitments to complete the basic requirements by providing either a completion date for the item or an intermediate date for providing a commitment to a completion date. The intermediate dates (status dates) were based on IMEC target completion dates for the item.

On June 12, 1984, the NRC issued "Order Confirming Licensee Commitments on Emergency Response Capability" which confirmed IMEC's Commitments.

IV.

The June 12, 1984, Order stated that for those requirements for which IMEC committed to a schedule for providing implementation dates, those dates would be reviewed, negotiated and confirmed by a subsequent order. In conformance with the milestones in the June 12, 1984 Order, IMEC's letters dated September 28, 1984, February 28 and April 29, 1985 provided completion schedules for the following requirements:

- | | |
|--|--|
| 1. Safety Parameter Display System (SPDS) | 1b. SPDS fully operational and operators trained. |
| 2. Detailed Control Room Design Review (DCRDR) | 2b. Submit a summary report to the NRC including a proposed schedule for implementation. |
| 4. Upgrade Emergency Operating Procedures (EOPs) | 4b. Implement the upgraded EOPs. |
| 5. Emergency Response Facilities | 5a. Technical Support Center |
| Fully Functional | 5b. Operations Staging Area |
| | 5c. Emergency Response Facility |

The attached Table summarizing IMEC's schedular commitments for the above items was developed by the NRC staff from the information provided by IMEC. The NRC staff finds that these dates are reasonable and achievable dates for

meeting the Commission requirements. The NRC staff concludes that the schedule proposed by the licensee will provide timely upgrading of the licensee's emergency response capability.

In view of the foregoing, I have determined that the implementation of IMEC's commitments are required in the interest of the public health and safety and should, therefore, be confirmed by an immediately effective Order.

V.

Accordingly, pursuant to Sections 103, 161i, 161o and 182 of the Atomic Energy Act of 1954, as amended, and the Commission's regulations in 10 CFR 2.204 and 10 CFR Part 50, IT IS HEREBY ORDERED, EFFECTIVE IMMEDIATELY, THAT license DPR-20 is modified to provide that the licensee shall:

Implement the specific items described in the Attachment to this ORDER in the manner described in IMEC's submittals noted in Section IV herein no later than the dates in the Attachment.

Extension of time for completing these items may be granted by the Director, Division of Licensing, for good cause shown.

VI.


Any other person with an adversely affected interest may request a hearing on this Order within 20 days of the date of publication of this Order in the Federal Register. Any request for a hearing should be addressed to the Director, Office of Nuclear Reactor Regulation, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555. A copy should be sent to the Executive Legal Director at the same address. A REQUEST FOR HEARING SHALL NOT STAY THE IMMEDIATE EFFECTIVENESS OF THIS ORDER.

If a hearing is to be held, the Commission will issue an Order designating the time and place of any such hearing.

If a hearing is held concerning this Order, the issue to be considered at the hearing shall be whether the licensee should comply with the requirements set forth in Section V of this Order.

This Order is effective upon issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Hugh L. Thompson, Jr., Director
Division of Licensing
Office of Nuclear Reactor Regulation

Dated in Bethesda, Maryland
this 12th day of July, 1985.

Attachment:
Licensee's Additional Commitments
on Requirements Specified in
Supplement 1 to NUREG-0737

DONALD C. COOK NUCLEAR PLANT

LICENSEE'S ADDITIONAL COMMITMENTS ON SUPPLEMENT 1 TO NUREG-0737

| TITLE | REQUIREMENTS | LICENSEE'S COMPLETION SCHEDULE (OR STATUS) |
|--|--|---|
| 1. Safety Parameter Display System (SPDS) | 1b. SPDS fully operational and operators trained. | December 1985 |
| 2. Detailed Control Room Design Review (DCRDR) | 2b. Submit a summary report to the NRC including a proposed schedule for implementation. | December 1986 |
| 4. Upgrade Emergency Operating Procedures (EOPs) | 4b. Implement the upgraded EOPs. | December 1985 |
| 5. Emergency Response Facilities Fully Functional | 5a. Technical Support Center 5b. Operations Staging Area 5c. Emergency Response Facility | Complete* Complete* Complete* |

*SPDS to be installed and operational December 1985.