April 1, 1987

Docket Nos.: 50-315
and 50-316

Mr. John Dolan, Vice President
Indiana and Michigan Electric Company
c/o American Electric Power Service Corporation
1 Riverside Plaza
Columbus, Ohio 43216

Dear Mr. Dolan:

The Commission has issued the enclosed Amendment No.103 to Facility Operating License No. DPR-58 and Amendment No. 89 to Facility Operating License No. DPR-74 for the Donald C. Cook Nuclear Plant, Unit Nos. 1 and 2. The amendments consist of changes to the Technical Specifications in response to your application transmitted by letter dated November 13, 1986.

These amendments revise the Technical Specifications to clarify the intent of the steam generator Section 3/4.4.5. Within the IMEC proposal to change the intent to "integrity" rather than "operable" and in discussion with your staff, the revision provided by the enclosed amendments adds a footnote which allows crevice flushing of all four steam generators at the same time in Mode 4.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next bi-weekly Federal Register notice.

Sincerely,

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D. L. Wigginton, Project Manager
PWR Project Directorate #4
Division of PWR Licensing-A

Enclosures:
1. Amendment No. 103 to DPR-58
2. Amendment No. 89 to DPR-74
3. Safety Evaluation

cc: w/enclosures
See next page

PWR#4/DPWR-A DWigginton/rad 03/25/87	PWR#4/DPWR-A MDuncan 03/25/87	EB/PWR-A RBallard 03/25/87	FOB/PWR-A VBenaroya 03/26/87
RSB/DPWR-A CBerlinger 03/26/87	PWR#4/DPWR-A BJYoungblood 03/31/87		

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PDR ADOCK 05000315
P PDR

Mr. John Dolan
Indiana and Michigan Electric Company

Donald C. Cook Nuclear Plant

cc:
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Vice President
Nuclear Operations
American Electric Power Service
Corporation
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Columbus, Ohio 43215

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Department of Attorney General
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U.S. Nuclear Regulatory Commission
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Stevensville, Michigan 49127

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Mayor, City of Bridgeman
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Bridgeman, Michigan 49106

Special Assistant to the Governor
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Lansing, Michigan 48909

Nuclear Facilities and Environmental
Monitoring Section Office
Division of Radiological Health
Department of Public Health
3500 N. Logan Street
Post Office Box 30035
Lansing, Michigan 48909

The Honorable John E. Grotberg
United States House of Representatives
Washington, DC 20515

Regional Administrator, Region III
U.S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, Illinois 60137

J. Feinstein
American Electric Power
Service Corporation
1 Riverside Plaza
Columbus, Ohio 43216

Dated: April 1, 1987

AMENDMENT NO. 103 TO FACILITY OPERATING LICENSE NO. DPR-58 - DONALD C. COOK, UNIT 1
AEMEDMENT NO. 89 TO FACILITY OPERATING LICENSE NO. DPR-74 - DONALD C. COOK, UNIT 2

DISTRIBUTION: w/enclosures
Docket File
NRC PDR
Local PDR
PWR#4 Reading
M. Duncan
D. Wigginton
B. J. Youngblood Reading
ACRS (10)
OGC - Bethesda
J. Partlow
B. Grimes
E. Jordan
R. Diggs
T. Barnhart (8)
W. Jones
E. Butcher
L. Harmon
FOB
OPA
LFMB
R. Perch

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

INDIANA AND MICHIGAN ELECTRIC COMPANY

DOCKET NO. 50-315

DONALD C. COOK NUCLEAR PLANT UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 103
License No. DPR-58

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Indiana and Michigan Electric Company (the licensee) dated November 13, 1986, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-58 is hereby amended to read as follows:

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P PDR

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No.103, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. The Technical Specifications are to become effective within 45 days of receipt of this amendment.

4. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/S/

Dave L. Wigginton, Project Manager
PWR Project Directorate #4
Division of PWR Licensing-A

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 1, 1987

PWR#4/DPWR-A	PWR#4/DPWR-A	OGC-Bethesda	PWR#4/DPWR-A
DWigginton/rad	MDuncan		BJYdungblood
~~12/5~~ 3/5/86	~~12/25~~ 3/25/86	~~12/~~ /86 3/27/87	~~12/3~~ 3/3/87

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

INDIANA AND MICHIGAN ELECTRIC COMPANY

DOCKET NO. 50-316

DONALD C. COOK NUCLEAR PLANT UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 89
License No. DPR-74

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Indiana and Michigan Electric Company (the licensee) dated November 13, 1986, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-74 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 89 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. The Technical Specification changes are effective within 45 days of receipt of this amendment.

4. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/S/

Dave L. Wigginton, Project Manager
PWR Project Directorate #4
Division of PWR Licensing-A

Attachment:
Changes to the Technical Specifications

Date of Issuance: April 1, 1987

PWR#4/DPWR-A	PWR#4/DPWR-A	OGC-Bethesda	PWR#4/DPWR-A
DWigginton/rad	MDuncan	MYoung	BJYoungblood
~~12~~/3/25/86	~~12~~/3/25/86	~~3~~/3/27/87	~~3~~/3/1/87

ATTACHMENT TO LICENSE AMENDMENTS

AMENDMENT NO. 103 FACILITY OPERATING LICENSE NO. DPR-58

AMENDMENT NO. 89 FACILITY OPERATING LICENSE NO. DPR-74

DOCKET NOS. 50-315 AND 50-316

Revise Appendix A as follows:

Remove Pages	Insert Pages
Unit 1	
3/4 4-7	3/4 4-7
3/4 4-12	3/4 4-12
B3/4 4-2a	B3/4 4-2a
Unit 2	
3/4 4-7	3/4 4-7
3/4 4-8*	3/4 4-8*
3/4 4-12	3/4 4-12
B3/4 4-2a	B3/4 4-2a

*Included for convenience only

LIMITING CONDITION FOR OPERATION

3.4.5 Each steam generator shall be OPERABLE.

APPLICABILITY: MODES 1, 2, 3 and 4*.

ACTION:

With one or more steam generators inoperable, restore the inoperable generator(s) to OPERABLE status prior to increasing T_{avg} above 200°F.

SURVEILLANCE REQUIREMENTS

4.4.5.0 Each steam generator shall be demonstrated OPERABLE by performance of the following augmented inservice inspection program and the requirement of Specification 4.0.5.

4.4.5.1 Steam Generator Sample Selection and Inspection - Each steam generator shall be determined OPERABLE during shutdown by selecting and inspecting at least the minimum number of steam generators specified in Table 4.4-1.

4.4.5.2 Steam Generator Tube Sample Selection and Inspection - The steam generator tube minimum sample size, inspection result classification, and the corresponding action required shall be as specified in Table 4.4-2. The inservice inspection of steam generator tubes shall be performed at the frequencies specified in Specification 4.4.5.3 and the inspected tubes shall be verified acceptable per the acceptance criteria of Specification 4.4.5.4. The tubes selected for each inservice inspection shall include at least 3% of the total number of tubes in all steam generators; the tubes selected for these inspections shall be selected on a random basis except:

a. Where experience in similar plants with similar water chemistry indicates critical areas to be inspected, then at least 50% of the tubes inspected shall be from these critical areas.

b. The first sample of tubes selected for each inservice inspection (subsequent to the preservice inspection) of each steam generator shall include:

* This Specification does not apply in Mode 4 while performing crevice flushing as long as Limiting Conditions For Operation for Specification 3.4.1.3 are maintained.

TABLE 4.4-1

MINIMUM NUMBER OF STEAM GENERATORS TO BE

INSPECTED DURING INSERVICE INSPECTION

Preservice Inspection	Yes
No. of Steam Generators per Unit	Four
First Inservice Inspection	Two
Second & Subsequent Inservice Inspections	One[2]

Table Notation:

1. The inservice inspection may be limited to one steam generator on a rotating schedule encompassing 3 N % of the tubes (where N is the number of steam generators in the plant) if the results of the first or previous inspections indicate that all steam generators are performing in a like manner. Note that under some circumstances, the operating conditions in one or more steam generators may be found to be more severe than those in other steam generators. Under such circumstances the sample sequence shall be modified to inspect the most severe conditions.

2. The third and fourth steam generators not inspected during the first inservice inspection shall be inspected during the second and third inspections, respectively. The fourth and subsequent inspections shall follow the instructions described in 1 above.

3/4.4.5 STEAM GENERATOR TUBE INTEGRITY

The Surveillance Requirements for inspection of the steam generator tubes ensure that the structural integrity of this portion of the RCS will be maintained. The program for inservice inspection of steam generator tubes is based on a modification of Regulatory Guide 1.83, Revision 1. Inservice inspection of steam generator tubing is essential in order to maintain surveillance of the conditions of the tubes in the event that there is evidence of mechanical damage or progressive degradation due to design, manufacturing errors, or inservice conditions that lead to corrosion. Inservice inspection of steam generator tubing also provides a means of characterizing the nature and cause of any tube degradation so that corrective measures can be taken.

The plant is expected to be operated in a manner such that the secondary coolant will be maintained within those chemistry limits found to result in negligible corrosion of the steam generator tubes. If the secondary coolant chemistry is not maintained within these parameter limits, localized corrosion may likely result in stress corrosion cracking. The extent of cracking during plant operation would be limited by the limitation of steam generator tube leakage between the primary coolant system and the secondary coolant system (primary-to-secondary leakage = 500 gallons per day per steam generator). Cracks having a primary-to-secondary leakage less than this limit during operation will have an adequate margin of safety to withstand the loads imposed during normal operation and by postulated accidents. Operating plants have demonstrated that primary-to-secondary leakage of 500 gallons per day per steam generator can readily be detected by radiation monitors of steam generator blowdown. Leakage in excess of this limit will require plant shutdown and an unscheduled inspection, during which the leaking tubes will be located and plugged. A steam generator while undergoing crevice flushing in Mode 4 is available for decay heat removal and is operable/operating upon reinstatement of auxiliary or main feed flow control and steam control.

Wastage-type defects are unlikely with the all volatile treatment (AVT) of secondary coolant. However, even if a defect of similar type should develop in service, it will be found during scheduled inservice steam generator tube examinations. Plugging will be required for all tubes with imperfections exceeding the plugging limit which, by the definition of Specification 4.4.5.4.a is 40% of the tube nominal wall thickness. Steam generator tube inspections of operating plants have demonstrated the capability to reliably detect degradation that has penetrated 20% of the original tube wall thickness.

Whenever the results of any steam generator tubing inservice inspection fall into Category C-3, these results will be promptly reported to the Commission pursuant to Specification 6.9.1 prior to resumption of plant operation. Such cases will be considered by the Commission on a case-by-case basis and may result in a requirement for analysis, laboratory examinations, tests, additional eddy-current inspection, and revision of the Technical Specifications, if necessary.

LIMITING CONDITION FOR OPERATION

3.4.5 Each steam generator shall be OPERABLE.

APPLICABILITY: MODES 1, 2, 3 and 4.*

ACTION:

With one or more steam generators inoperable, restore the inoperable generator(s) to OPERABLE status prior to increasing T_{avg} above 200°F.

SURVEILLANCE REQUIREMENTS

4.4.5.0 Each steam generator shall be demonstrated OPERABLE by performance of the following augmented inservice inspection program and the requirement of Specification 4.0.5.

4.4.5.1 Steam Generator Sample Selection and Inspection - Each steam generator shall be determined OPERABLE during shutdown by selecting and inspecting at least the minimum number of steam generators specified in Table 4.4-1.

4.4.5.2 Steam Generator Tube Sample Selection and Inspection - The steam generator tube minimum sample size, inspection result classification, and the corresponding action required shall be as specified in Table 4.4-2. The inservice inspection of steam generator tubes shall be performed at the frequencies specified in Specification 4.4.5.3 and the inspected tubes shall be verified acceptable per the acceptance criteria of Specification 4.4.5.4. The tubes selected for each inservice inspection shall include at least 3% of the total number of tubes in all steam generators; the tubes selected for these inspections shall be selected on a random basis except:

a. Where experience in similar plants with similar water chemistry indicates critical areas to be inspected, then at least 50% of the tubes inspected shall be from these critical areas.

b. The first sample of tubes selected for each inservice inspection (subsequent to the preservice inspection) of each steam generator shall include:

* This Specification does not apply in Mode 4 while performing crevice flushing as long as Limiting Conditions For Operation for Specification 3.4.1.3 are maintained.

1. All nonplugged tubes that previously had detectable wall penetrations (>20%).

2. Tubes in those areas where experience has indicated potential problems.

3. A tube inspection (pursuant to Specification 4.4.5.4.a.8) shall be performed on each selected tube. If any selected tube does not permit the passage of the eddy current probe for a tube inspection, this shall be recorded and an adjacent tube shall be selected and subjected to a tube inspection.

c. The tubes selected as the second and third samples (if required by Table 4.4-2) during each inservice inspection may be subjected to a partial tube inspection provided:

1. The tubes selected for these samples include the tubes from those areas of the tube sheet array where tubes with imperfections were previously found.

2. The inspections include those portions of the tubes where imperfections were previously found.

The results of each sample inspection shall be classified into one of the following three categories:

Category	Inspection Results
C-1	Less than 5% of the total tubes inspected are degraded tubes and none of the inspected tubes are defective.
C-2	One or more tubes, but not more than 1% of the total tubes inspected are defective, or between 5% and 10% of the total tubes inspected are degraded tubes.
C-3	More than 10% of the total tubes inspected are degraded tubes or more than 1% of the inspected tubes are defective.

Note: In all inspections, previously degraded tubes must exhibit significant (>10%) further wall penetrations to be included in the above percentage calculations.

TABLE 4.4-1

MINIMUM NUMBER OF STEAM GENERATORS TO BE

INSPECTED DURING INSERVICE INSPECTION

Preservice Inspection	Yes
No. of Steam Generators per Unit	Four
First Inservice Inspection	Two
Second & Subsequent Inservice Inspections	One[2]

Table Notation:

1. The inservice inspection may be limited to one steam generator on a rotating schedule encompassing 3 N % of the tubes (where N is the number of steam generators in the plant) if the results of the first or previous inspections indicate that all steam generators are performing in a like manner. Note that under some circumstances, the operating conditions in one or more steam generators may be found to be more severe than those in other steam generators. Under such circumstances the sample sequence shall be modified to inspect the most severe conditions.

2. The third and fourth steam generators not inspected during the first inservice inspection shall be inspected during the second and third inspections, respectively. The fourth and subsequent inspections shall follow the instructions described in 1 above.

3/4.4.5 STEAM GENERATOR TUBE INTEGRITY

The Surveillance Requirements for inspection of the steam generator tubes ensure that the structural integrity of this portion of the RCS will be maintained. The program for inservice inspection of steam generator tubes is based on a modification of Regulatory Guide 1.83, Revision 1. Inservice inspection of steam generator tubing is essential in order to maintain surveillance of the conditions of the tubes in the event that there is evidence of mechanical damage or progressive degradation due to design, manufacturing errors, or inservice conditions that lead to corrosion. Inservice inspection of steam generator tubing also provides a means of characterizing the nature and cause of any tube degradation so that corrective measures can be taken.

The plant is expected to be operated in a manner such that the secondary coolant will be maintained within those chemistry limits found to result in negligible corrosion of the steam generator tubes. If the secondary coolant chemistry is not maintained within these limits, localized corrosion may likely result in stress corrosion cracking. The extent of cracking during plant operation would be limited by the limitation of steam generator tube leakage between the primary coolant system and the secondary coolant system (primary-to-secondary leakage = 500 gallons per day per steam generator). Cracks having a primary-to-secondary leakage less than this limit during operation will have an adequate margin of safety to withstand the loads imposed during normal operation and by postulated accidents. Operating plants have demonstrated that primary-to-secondary leakage of 500 gallons per day per steam generator can readily be detected by radiation monitors of steam generator blowdown. Leakage in excess of this limit will require plant shutdown and an unscheduled inspection, during which the leaking tubes will be located and plugged. A steam generator while undergoing crevice flushing in Mode 4 is available for decay heat removal and is operable/operating upon reinstatement of auxiliary or main feed flow control and steam control.

Wastage-type defects are unlikely with secondary coolant. However, even if a defect should develop in service, it will be found during scheduled inservice steam generator tube examinations. Plugging will be required for all tubes with imperfections exceeding the plugging limit of 40% of the tube nominal wall thickness. Steam generator tube inspections of operating plants have demonstrated the capability to reliably detect degradation that has penetrated 20% of the original tube wall thickness.

Whenever the results of any steam generator tubing inservice inspection fall into Category C-3, these results will be promptly reported to the Commission pursuant to Specification 6.9.1 prior to resumption of plant operation. Such cases will be considered by the Commission on a case-by-case basis and may result in a requirement for analysis, laboratory examinations, tests, additional eddy-current inspection, and revision of the Technical Specifications, if necessary.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 103TO FACILITY OPERATING LICENSE NO. DPR-58

AND AMENDMENT NO.89 TO FACILITY OPERATING LICENSE NO. DPR-74

INDIANA AND MICHIGAN ELECTRIC COMPANY

DONALD C. COOK NUCLEAR PLANT, UNITS 1 AND 2

DOCKET NOS. 50-315 AND 50-316

INTRODUCTION

By letter dated November 13, 1986, the Indiana and Michigan Electric Company (the licensee) submitted a proposed amendment that would change Section 3/4.4.5 to address steam generator "integrity." The current Technical Specifications address OPERABLE. The licensee also proposed to change footnote 2 to Table 4.4-1 to clarify the requirement that second and subsequent inspections would be on one of two remaining steam generators respectively.

EVALUATION

The Technical Specifications Section 3/4.4.5 currently addresses OPERABLE requirements of the steam generators. The intent of Section 3/4.4.5 as supported by the Bases section appears to be steam generator tube integrity. The licensee first brought this to the staff's attention in early 1986. On May 14, 1986 in a letter from B.J. Youngblood to John Dolan, IMEC, the NRC agreed that the current Technical Specification was inappropriately worded and that the Bases section supported the intent of this specification to address the "integrity" of the steam generators. The licensee was encouraged, by the letter, to modify the Technical Specifications at their next convenience with changes we would find acceptable.

Upon receipt of the licensee's proposed license amendment, the staff undertook a re-evaluation of the position that the Technical Specification 3.4.5 dealt exclusively with integrity and that the question of operable steam generators was restricted to the issue of adequate decay heat removal which is addressed by Technical Specification 3.4.1. It was determined that additional review and consideration would be prudent to proceed with the overall change to Specification 3.4.5, i.e. change operable to integrity. Therefore, in discussions with the licensee it was decided to restrict the changes to only those needed and to defer the overall change proposal to another review. The only operation of concern to the licensee at this time for which steam generator operability is an issue is crevice flushing in Mode 4.

Crevice flushing is a process to remove corrosion causing contaminates from the crevice between the steam generator tubes and the tubesheet. The steam generators are pressurized with a small amount of water on the steam or crevice side. The

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reactor coolant system is heated up with the reactor coolant pumps in operation and with one residual heat removal (RHR) system in operation and the second available per Technical Specification 3.4.1. The coolant temperature must be above 200°F (Mode 4) to provide the heat necessary to allow the steam side to boil. The pressure is released on the steam generators and the crevice area boils, forcing the contaminates out of the crevice. The process is controlled by procedure so that the steam generators are not boiled dry. This allows the contaminates to remain in solution and not return to the crevice and it allows the auxiliary or main feed water to be started without thermal shocking a hot dry steam generator. During crevice flushing, the steam generators are not removing decay heat but they are available and could be reinstated to OPERABLE/operating status with initiation of auxiliary or main feed flow control and steam control, however, the licensee proposes to flush all four steam generators at the same time. This operation is within the licensee's proposed amendment change to remove operable from Technical Specification 3.4.5; other possible operations would require additional considerations not covered by this evaluation. To assure crevice flushing all four steam generators at the same time is acceptable, our evaluation concentrated on the impact to the reactor core.

The process of crevice flushing the D.C. Cook steam generators is expected to produce a decrease in reactor system temperature. Following the flushing procedures the reactor system temperature may increase slightly as the heat load from the reactor coolant pumps and the small amount of decay heat is transferred to the operable RHR train. The licensee evaluated the effect of reactor system temperature changes on core reactivity to ensure that inadvertent criticality cannot occur. Cooldown of the reactor system cannot cause criticality since the coolant will be borated to maintain the required shutdown margin to 68°F. Heatup cannot cause criticality since the reactor will be sufficiently borated so that even with the worst case positive moderator coefficient, the required shutdown margin will be maintained. We therefore, find the crevice flushing of all four steam generators at the same time to be acceptable. In place of the requested change to Technical Specification 3.4.5, we proposed and the licensee agreed to a more limited but acceptable footnote which allows crevice flushing.

In our letter dated May 14, 1986, we further suggested clarifying language to footnote 2, Table 4.4-1 which addresses the second and subsequent inspections of steam generators after the initial inspection which covers two of the four steam generators. The footnote was to assure that the third and fourth steam generators would be covered by the second and subsequent inspections so that after three inspections, all four steam generators would be inspected. The licensee has adopted the proposed changes and we find this acceptable.

ENVIRONMENTAL CONSIDERATION

These amendments involve a change in the installation or use of the facilities' components located within the restricted areas as defined in 10 CFR 20. The staff has determined that these amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that these amendments involve no significant hazards

consideration, and there has been no public comment on such finding. Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR Sec 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors: D. Wigginton
W. Jensen

Dated: April 1, 1987