Docket Nos. 50-315
and 50-316

April 23, 1986

Mr. John Dolan, Vice President
Indiana and Michigan Electric Company
c/o American Electric Power Service Corporation
1 Riverside Plaza
Columbus, Ohio 43216

Dear Mr. Dolan:

The Commission has issued the enclosed Amendment No. 95 to Facility Operating License No. DPR-58 and Amendment No. 81 to Facility Operating License No. DPR-74 for the Donald C. Cook Nuclear Plant, Unit Nos. 1 and 2. The amendments consist of changes to the Technical Specifications in response to your application transmitted by letter dated October 19, 1984, as supplemented by letters dated May 31, 1985, October 18, 1985, and February 7, 1986.

These amendments revise the Technical Specifications for testing certain containment isolation valves.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next bi-weekly Federal Register notice.

Sincerely,

(S)

D. L. Wigginton, Project Manager
PWR Project Directorate #4
Division of PWR Licensing-A, NRR

Enclosures:
1. Amendment No. 95 to DPR-58
2. Amendment No. 81 to DPR-74
3. Safety Evaluation

cc: w/enclosures
See next page

DW			DSH for
PWR#4:DPWR-A	PWR#4:DPWR-A	OELD	PWR#4:DPWR-A
DWigginton:mac	MDuncan	SRT	BJYoungblood
04/16/86	04/16/86	04/17/86	04/16/86

8605020227 860423
PDR ADOCK 05000315
P PDR

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

April 23, 1986

Docket Nos. 50-315
and 50-316

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D. L. Wigginton, Project Manager
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cc: w/enclosures
See next page

Mr. John Dolan
Indiana and Michigan Electric Company

Donald C. Cook Nuclear Plant

cc:
Mr. M. P. Alexich
Vice President
Nuclear Operations
American Electric Power Service
Corporation
1 Riverside Plaza
Columbus, Ohio 43215

Attorney General
Department of Attorney General
525 West Ottawa Street
Lansing, Michigan 48913

Township Supervisor
Lake Township Hall
Post Office Box 818
Bridgman, Michigan 49106

W. G. Smith, Jr., Plant Manager
Donald C. Cook Nuclear Plant
Post Office Box 458
Bridgman, Michigan 49106

U.S. Nuclear Regulatory Commission
Resident Inspectors Office
7700 Red Arrow Highway
Stevensville, Michigan 49127

Gerald Charnoff, Esquire
Shaw, Pittman, Potts and Trowbridge
1800 M Street, N.W.
Washington, DC 20036

Mayor, City of Bridgeman
Post Office Box 366
Bridgman, Michigan 49106

Special Assistant to the Governor
Room 1 - State Capitol
Lansing, Michigan 48909

Nuclear Facilities and Environmental
Monitoring Section Office
Division of Radiological Health
Department of Public Health
3500 N. Logan Street
Post Office Box 30035
Lansing, Michigan 48909

The Honorable John E. Grotberg
United States House of Representatives
Washington, DC 20515

Regional Administrator, Region III
U.S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, Illinois 60137

J. Feinstein
American Electric Power
Service Corporation
1 Riverside Plaza
Columbus, Ohio 43216

April 23, 1986

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AMENDMENT NO. 95 and 81 TO DC COOK UNITS 1 AND 2

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

INDIANA AND MICHIGAN ELECTRIC COMPANY

DOCKET NO. 50-315

DONALD C. COOK NUCLEAR PLANT UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 95
License No. DPR-58

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Indiana and Michigan Electric Company (the licensee) dated October 19, 1984, as supplemented by letters dated May 31, 1985, October 18, 1985, and February 7, 1986, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 2.C.(2) of Facility Operating License No. DPR-58 is hereby amended to read as follows:

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PDR ADOCK 05000315
P PDR

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 95, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. The change in Technical Specifications is to become effective within 60 days of issuance of the amendment. In the period between issuance of the amendment and the effective date of the new Technical Specifications, the licensee shall adhere to the Technical Specifications for the systems, components, or operation existing at the time. The period of time during changeover of systems, components or operation shall be minimized or compensated for by suitable temporary alternatives.

4. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Paul W O'Connor

for B. J. Youngblood, Director
PWR Project Directorate #4
Division of PWR Licensing-A, NRR

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 23, 1986

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 95 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

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FOR THE NUCLEAR REGULATORY COMMISSION

/S/

B. J. Youngblood, Director
PWR Project Directorate #4
Division of PWR Licensing-A, NRR

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 23, 1986

			DSH for
PWR#4:DPWR-A	PWR#4:DPWR-A	OELD	PWR#4:DPWR-A
DWigginton:mac	MDuncan		BJYoungblood
04/16/86	04/16/86	04/17/86	04/16/86

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 81 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. The change in Technical Specifications is to become effective within 60 days of issuance of the amendment. In the period between issuance of the amendment and the effective date of the new Technical Specifications, the licensee shall adhere to the Technical Specifications for the systems, components, or operation existing at the time. The period of time during changeover of systems, components or operation shall be minimized or compensated for by suitable temporary alternatives.

4. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

B. J. Youngblood, Director
PWR Project Directorate #4
Division of PWR Licensing-A, NRR

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 23, 1986

PWR#4:DPWR-A	PWR#4:DPWR-A	OELD	PWR#4:DPWR-A
DWigginton:mac	MDuncan		BJYoungblood
04/16/86	04/16/86	04/17/86	04/16/86

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

INDIANA AND MICHIGAN ELECTRIC COMPANY

DOCKET NO. 50-316

DONALD C. COOK NUCLEAR PLANT UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 81
License No. DPR-74

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Indiana and Michigan Electric Company (the licensee) dated October 19, 1984, as supplemented by letters dated May 31, 1985, October 18, 1985, and February 7, 1986, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 2.C.(2) of Facility Operating License No. DPR-74 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 81 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. The change in Technical Specifications is to become effective within 60 days of issuance of the amendment. In the period between issuance of the amendment and the effective date of the new Technical Specifications, the licensee shall adhere to the Technical Specifications for the systems, components, or operation existing at the time. The period of time during changeover of systems, components or operation shall be minimized or compensated for by suitable temporary alternatives.

4. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Paul W O'Connor

for B. J. Youngblood, Director
PWR Project Directorate #4
Division of PWR Licensing-A, NRR

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 23, 1986

ATTACHMENT TO LICENSE AMENDMENTS

AMENDMENT NO. 95 FACILITY OPERATING LICENSE NO. DPR-58

AMENDMENT NO. 81 FACILITY OPERATING LICENSE NO. DPR-74

DOCKET NOS. 50-315 AND 50-316

Revise Appendix A as follows:

Remove Pages	Insert Pages
UNIT 1	
3/4 6-1	3/4 6-1
3/4 6-2*	3/4 6-2*
3/4 6-14	3/4 6-14
3/4 6-15	3/4 6-15
3/4 6-16	3/4 6-16
3/4 6-17	3/4 6-17
3/4 6-18	3/4 6-18
3/4 6-19	3/4 6-19
3/4 6-20	3/4 6-20
3/4 6-21	3/4 6-21
3/4 6-22	3/4 6-22
UNIT 2	
3/4 6-17	3/4 6-17
3/4 6-18	3/4 6-18
3/4 6-19	3/4 6-19
3/4 6-22	3/4 6-22
3/4 6-23	3/4 6-23
3/4 6-24	3/4 6-24
3/4 6-27*	3/4 6-27*
3/4 6-28	3/4 6-28
3/4 6-29	3/4 6-29
3/4 6-30	3/4 6-30
3/4 6-31	3/4 6-31

* Included for Convenience

3/4.6 CONTAINMENT SYSTEMS

3/4.6.1 PRIMARY CONTAINMENT

CONTAINMENT INTEGRITY

LIMITING CONDITION FOR OPERATION

3.6.1.1 Primary CONTAINMENT INTEGRITY shall be maintained.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

Without primary CONTAINMENT INTEGRITY, restore CONTAINMENT INTEGRITY within one hour or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.6.1.1 Primary CONTAINMENT INTEGRITY shall be demonstrated:

a. At least once per 31 days by verifying that:

1. All penetrations*not capable of being closed by OPERABLE containment automatic isolation valves and required to be closed during accident conditions are closed by valves, blind flanges, or deactivated automatic valves secured in their positions, except as provided in Table 3.6-1 of Specification 3.6.3.1, and

2. All equipment hatches are closed and sealed,

b. By verifying that each containment air lock is OPERABLE per Specification 3.6.1.3.

*Except valves, blind flanges, and deactivated automatic valves which are located inside the containment and are locked, sealed or otherwise secured in the closed position. These penetrations shall be verified closed during each COLD SHUTDOWN except that such verification need not be performed more often than once per 92 days.

CONTAINMENT LEAKAGE

LIMITING CONDITION FOR OPERATION

3.6.1.2 Containment leakage rates shall be limited to:

a. An overall integrated leakage rate of $\leq L_a$, 0.25 percent by weight of the containment air per 24 hours at P_a, 12.0 psig, and

b. A combined leakage rate of $\leq 0.60\ L_a$ for all penetrations and valves subject to Type B and C tests when pressurized to P_a.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

With either (a) the measured overall integrated containment leakage rate exceeding $0.75\ L_a$ or (b) with the measured combined leakage rate for all penetrations and valves subject to Types B and C tests exceeding $0.60\ L_a$ restore the leakage rate(s) to within the limit(s) prior to increasing the Reactor Coolant System temperature above 200°F.

SURVEILLANCE REQUIREMENTS

4.6.1.2 The containment leakage rates shall be demonstrated at the following test schedule and shall be determined in conformance with the criteria specified in Appendix J of 10 CFR 50 using the methods and provisions of ANSI N45.4-1972:

a. Three Type A tests (Overall Integrated Containment Leakage Rate) shall be conducted at 40 $\pm$ 10 month intervals during shutdown at P_a, 12.0 psig, during each 10-year service period. The third test of each set shall be conducted during the shutdown for the 10-year plant inservice inspection.

3/4.6.3 CONTAINMENT ISOLATION VALVES

LIMITING CONDITION FOR OPERATION

3.6.3.1 The containment isolation valves specified in Table 3.6-1 shall be OPERABLE with isolation times as shown in Table 3.6-1. The ACTION statement of T/S 3/4.6.3 is not applicable to the containment purge supply and exhaust isolation valves, VCR-101 through 106 and VCR-201 through 206, listed in Table 3.6-1. The Limiting Condition for Operation and its associated ACTION statement for these valves is given in Technical Specification 3/4.6.1.7.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

With one or more of the isolation valve(s) specified in Table 3.6-1 inoperable, either:

a. Restore the inoperable valve(s) to OPERABLE status within 4 hours, or

b. Isolate each affected penetration within 4 hours by use of at least one deactivated automatic valve secured in the isolation position, or

c. Isolate each affected penetration within 4 hours by use of at least one closed manual valve or blind flange, or

d. Be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.6.3.1.1 The isolation valves specified in Table 3.6-1 shall be demonstrated OPERABLE prior to returning the valve to service after maintenance, repair or replacement work is performed on the valve or its associated actuator, control or power circuit by performance of a cycling test and verification of isolation time.

SURVEILLANCE REQUIREMENTS (Continued)

valve or its associated actuator, control or power circuit by performance of the cycling test, above, and verification of isolation time.

4.6.3.1.2 Each isolation valve specified in Table 3.6-1 shall be demonstrated OPERABLE during the COLD SHUTDOWN or REFUELING MODE at least once per 18 months by:

a. Verifying that on a Phase A containment isolation test signal, each Phase A isolation valve actuates to its isolation position.

b. Verifying that on a Phase B containment isolation test signal, each Phase B isolation valve actuates to its isolation position.

c. Verifying that on a Containment Purge and Exhaust isolation signal, each Purge and Exhaust valve actuates to its isolation position.

4.6.3.1.3 The isolation time of each power operated or automatic valve of Table 3.6-1 shall be determined to be within its limit when tested pursuant to Specification 4.0.5.

TABLE 3.6-1

CONTAINMENT ISOLATION VALVES

VALVE NUMBER		FUNCTION	ISOLATION TIME IN SECONDS
A.	PHASE "A" ISOLATION		
1.	CCR-455	CCW to Reactor Supports	10
2.	CCR-456	CCW from Reactor Supports	10
3.	CCR-457	CCW from Reactor Supports	10
4.	CCR-460	CCW from Excess ltdn. Hx.	10
5.	CCR-462	CCW to Excess ltdn. Hx.	10
6.	DCR-201	R.C. Drain Tank to Vent. Hdr.	10
7.	DCR-202	R.C. Drain Tank to Gas Analyzer	10
8.	DCR-203	R.C. Drain Tank to Vent. Hdr.	10
9.	DCR-204	R.C. Drain Tank to Gas Analyzer	10
10.	DCR-205	R.C. Drain Pump Suction Isolation	10
11.	DCR-206	R.C. Drain Pump Suction Isolation	10
12.	DCR-207	N_2 Supply to R.C. Drain Tank	10
13.	DCR-301	Steam Generator Blowdown Sample #1	10
14.	DCR-302	Steam Generator Blowdown Sample #2	10
15.	DCR-303	Steam Generator Blowdown Sample #3	10
16.	DCR-304	Steam Generator Blowdown Sample #4	10
17.	DCR-310	Steam Generator Blowdown Lines #1	10
18.	DCR-320	Steam Generator Blowdown Lines #2	10
19.	DCR-330	Steam Generator Blowdown Lines #3	10
20.	DCR-340	Steam Generator Blowdown Lines #4	10
21.	DCR-600	Containment Sump to Waste Holdup	10
22.	DCR-601	Containment Sump to Waste Holdup	10
23.	DCR-610	Ice Condenser Drain to Drain Hdr.	10
24.	DCR-611	Ice Condenser Drain to Drain Hdr.	10
25.	DCR-620	Continuous Ventilation Drain to Holdup	10
26.	DCR-621	Continuous Ventilation Drain to Holdup	10

TABLE 3.6-1 (Continued)

VALVE NUMBER		FUNCTION	ISOLATION TIME IN SECONDS
A. PHASE "A" ISOLATION (Continued)			
27.	ECR-10	Cont. H_2 Sample Return	10
28.	ECR-11	Cont. H_2 Sample - Air to Rec. E	10
29.	ECR-12	Cont. H_2 Sample - Air from Rec. E	10
30.	ECR-13	Cont. H_2 Sample - Low. Cont. Vol.	10
31.	ECR-14	Cont. H_2 Sample - Low. Cont. Vol.	10
32.	ECR-15	Cont. H_2 Sample - Up Cont. Vol.	10
33.	ECR-16	Cont. H_2 Sample - Up Cont. Vol.	10
34.	ECR-17	Cont. H_2 Sample - Air to Rec. W	10
35.	ECR-18	Cont. H_2 Sample - Air from Rec. W	10
36.	ECR-19	Cont. H_2 Sample - Cont. Dome Vol.	10
37.	ECR-20	Cont. H_2 Sample - Return	10
38.	ECR-21	Cont. H_2 Sample - Air to Rec. E.	10
39.	ECR-22	Cont. H_2 Sample - Air fr. Rec. E	10
40.	ECR-23	Cont. H_2 Sample - Low Cont. Vol.	10
41.	ECR-24	Cont. H_2 Sample - Low Cont. Vol.	10
42.	ECR-25	Cont. H_2 Sample - Up Cont. Vol.	10
43.	ECR-26	Cont. H_2 Sample - Up Cont. Vol.	10
44.	ECR-27	Cont. H_2 Sample - Air to Rec. W.	10
45.	ECR-28	Cont. H_2 Sample - Air Fr. Rec. W.	10
46.	ECR-29	Cont. H_2 Sample - Cont. Dome Vol.	10
47.	ECR-416	PAS Containment Sump Sample	10
48.	ECR-417	PAS Containment Sump Sample	10
49.	ECR-496	PAS Waste Liquid and Gas Return	10
50.	ECR-497	PAS Waste Liquid and Gas Return	10
51.	ECR-535	PAS Containment Gas Sample	10
52.	ECR-536	PAS Containment Gas Sample	10
53.	GCR-301	N_2 Supply to Pressurizer Relief Tank	10
54.	GCR-314	N_2 Supply to Accumulators	10
55.	ICR-5	Accumulators Sample	10
56.	ICR-6	Accumulators Sample	10
57.	MCR-251	Sample Line from Steam Gen. Outlet #1	10
58.	MCR-252	Sample Line from Steam Gen. Outlet #2	10
59.	MCR-253	Sample Line from Steam Gen. Outlet #3	10
60.	MCR-254	Sample Line form Steam Gen. Outlet #4	10
61.	NCR-105	Hot Leg Sample	10
62.	NCR-106	Hot Leg Sample	10

TABLE 4.6-1 (Continued)

VALVE NUMBER	FUNCTION	ISOLATION TIME IN SECONDS
A. PHASE "A" ISOLATION (Continued)		
63. NCR-107	PRZ Liquid Sample	10
64. NCR-108	PRZ Liquid Sample	10
65. NCR-109	PRZ Steam Sample	10
66. NCR-110	PRZ Steam Sample	10
67. NCR-252	Primary Water to Pressure Relief Tank	10
68. PCR-40	Containment Service Air	10
69. QCM-250	RCP Seal Water Discharge	15
70. QCM-350	RCP Seal Water Discharge	15
71. QCR-300	Letdown to Letdown Hx.	10
72. QCR-301	Letdown to Letdown Hx.	10
73. QCR-919	Demineralized Water Supply for Refueling Cavity	10
74. QCR-920	Demineralized Water Supply for Refueling Cavity	10
75. RCR-100	PRZ Relief Tank to Gas Anal.	10
76. RCR-101	PRZ Relief Tank to Gas Anal.	10
77. VCR-10	Glycol Supply to Fan Cooler	10
78. VCR-11	Glycol Supply to Fan Cooler	10
79. VCR-20	Glycol Supply from Fan Cooler	10
80. VCR-21	Glycol Supply from Fan Cooler	10
81. XCR-100	Control Air to Containment	10
82. XCR-101	Control Air to Containment Isolation	10
83. XCR-102	Control Air to Containment Isolation	10
84. XCR-103	Control Air to Containment	10
B. PHASE "B" ISOLATION		
1. CCM-451	CCW from RCP Oil Coolers	60
2. CCM-452	CCW from RCP Oil Coolers	60
3. CCM-453	CCW from RCP Thermal Barrier	30
4. CCM-454	CCW from RCP Thermal Barrier	30
5. CCM-458	CCW to RCP Oil Coolers & Thermal Barrier	60
6. CCM-459	CCW to RCP Oil Coolers & Thermal Barrier	60
7. ECR-31	Containment Airborne Radiation Monitor	10
8. ECR-32	Containment Airborne Radiation Monitor	10
9. ECR-33	Containment Airborne Radiation Monitor	10
10. ECR-35	Containment Airborne Radiation Monitor	10
11. ECR-36	Containment Airborne Radiation Monitor	10

TABLE 3.6-1 (Continued)

VALVE NUMBER	FUNCTION	ISOLATION TIME IN SECONDS
B. PHASE "B" ISOLATION (Continued)		
12. WCR-901	NESW to Low Containment Vent #1	10
13. WCR-903	NESW from Low Containment Vent #1	10
14. WCR-905	NESW to Low Containment Vent #2	10
15. WCR-907	NESW from Low Containment Vent #2	10
16. WCR-909	NESW to Low Containment Vent #3	10
17. WCR-911	NESW from Low Containment Vent #3	10
18. WCR-913	NESW to Low Containment Vent #4	10
19. WCR-915	NESW from Low Containment Vent #4	10
20. WCR-921	NESW to Up Containment Vent #1	10
21. WCR-923	NESW from Up Containment Vent #1	10
22. WCR-925	NESW to Up Containment Vent #2	10
23. WCR-927	NESW from Up Containment Vent #2	10
24. WCR-929	NESW to Up Containment Vent #3	10
25. WCR-931	NESW from Up Containment Vent #3	10
26. WCR-933	NESW to Up Containment Vent #4	10
27. WCR-935	NESW from Up Containment Vent #4	10
28. WCR-941	NESW to RCP Motor Air Cooler	10
29. WCR-942	NESW to RCP Motor Air Cooler	10
30. WCR-943	NESW to RCP Motor Air Cooler	10
31. WCR-944	NESW to RCP Motor Air Cooler	10
32. WCR-945	NESW from RCP Motor Air Cooler	10
33. WCR-946	NESW from RCP Motor Air Cooler	10
34. WCR-947	NESW from RCP Motor Air Cooler	10
35. WCR-948	NESW from RCP Motor Air Cooler	10
36. WCR-951	NESW to RCP Motor Air Cooler Vent #1	10
37. WCR-952	NESW to RCP Motor Air Cooler Vent #2	10
38. WCR-953	NESW to RCP Motor Air Cooler Vent #3	10
39. WCR-954	NESW to RCP Motor Air Cooler Vent #4	10
40. WCR-955	NESW from RCP Motor Air Cooler Vent #1	10
41. WCR-956	NESW from RCP Motor Air Cooler Vent #2	10
42. WCR-957	NESW from RCP Motor Air Cooler Vent #3	10
43. WCR-958	NESW from RCP Motor Air Cooler Vent #4	10

TABLE 3.6-1 (Continued)

VALVE NUMBER	FUNCTION	ISOLATION TIME IN SECONDS
B. PHASE "B" ISOLATION (Continued)		
44. WCR-961	NESW to Instr. Rm. East Vent	10
45. WCR-963	NESW from Instr. Rm. West Vent	10
46. WCR-965	NESW to Instr. Rm. East Vent	10
47. WCR-967	NESW from Instr. Rm. West Vent	10
48. WCR-900	NESW to RCP Lower Containment Vent #1	10
49. WCR-902	NESW from Lower Containment Vent #1	10
50. WCR-904	NESW to RCP Lower Containment Vent #2	10
51. WCR-906	NESW from Lower Containment Vent #2	10
52. WCR-908	NESW to RCP Lower Containment Vent #3	10
53. WCR-910	NESW from Lower Containment Vent #3	10
54. WCR-912	NESW to RCP Lower Containment Vent #4	10
55. WCR-914	NESW from Lower Containment Vent #4	10
56. WCR-920	NESW to RCP Upper Containment Vent #1	10
57. WCR-922	NESW from Upper Containment Vent #1	10
58. WCR-924	NESW to RCP Upper Containment Vent #2	10
59. WCR-926	NESW from Upper Containment Vent #2	10
60. WCR-928	NESW to RCP Upper Containment Vent #3	10
61. WCR-930	NESW from Upper Containment Vent #3	10
62. WCR-932	NESW to RCP Upper Containment Vent #4	10
63. WCR-934	NESW from Upper Containment Vent #4	10
64. WCR-960	NESW to Instrument Room East Vent	10
65. WCR-962	NESW from Instrument Room East Vent	10
66. WCR-964	NESW to Instrument Room West Vent	10
67. WCR-966	NESW from Instrument Room West Vent	10
C. CONTAINMENT PURGE AND EXHAUST **		
1. VCR-101	Instr. Room Purge Air Inlet	5
2. VCR-102	Instr. Room Purge Air Outlet	5
3. VCR-103	Lower Comp. Purge Air Inlet	5
4. VCR-104	Lower Comp. Purge Air Outlet	5
5. VCR-105	Upper Comp. Purge Air Inlet	5
6. VCR-106	Upper Comp. Purge Air Outlet	5
7. VCR-107*	Cont. Press. Relief Fan Isolation	5
8. VCR-201	Instr. Room Purge Air Inlet	5
9. VCR-202	Instr. Room Purge Air Outlet	5
10. VCR-203	Lower Comp. Purge Air Inlet	5
11. VCR-204	Lower Comp. Purge Air Outlet	5

TABLE 3.6-1 (Continued)

VALVE NUMBER	FUNCTION	ISOLATION TIME IN SECONDS
C. CONTAINMENT PURGE EXHAUST (Continued)**		
12. VCR-205	Upper Comp. Purge Air Inlet	5
13. VCR-206	Upper Comp. Purge Air Outlet	5
14. VCR-207*	Cont. Press Relief Fan Isolation	5
D. MANUAL ISOLATION VALVES(1)		
1. ICM-111	RHR to RC Cold Legs	NA
2. ICM-129	RHR Inlet to Pumps	NA
3. ICM-250	Boron Injection Inlet	NA
4. ICM-251	Boron Injection Inlet	NA
5. ICM-260	Safety Injection Inlet	NA
6. ICM-265	Safety Injection Inlet	NA
7. ICM-305	RHR Suction from Sump	NA
8. ICM-306	RHR Suction from Sump	NA
9. ICM-311	RHR to RC Hot Legs	NA
10. ICM-321	RHR to RC Hot Legs	NA
11. NPX 151 VI	Dead Weight Tester	NA
12. PA-343	Containment Service Air	NA
13. SF-151	Refueling Water Supply	NA
14. SF-153	Refueling Water Supply	NA
15. SF-159	Refueling Cavity Drain to Purification System	NA
16. SF-160	Refueling Cavity Drain to Purification System	NA
17. SI-171	Safety Injection Test Line	NA
18. SI-172	Accumulator Test Line	NA

TABLE 3.6-1 (Continued)

VALVE NUMBER	FUNCTION	ISOLATION TIME IN SECONDS
D. MANUAL ISOLATION VALVES (1) (Continued)		
19. CCR-440	CCW from Main Steam Penetration	NA
20. CCR-441	CCW from Main Steam Penetration	NA
21. MCM-221	Main Steam to Auxiliary Feed Pump	NA
22. MCM-231	Main Steam to Auxiliary Feed Pump	NA
23. CCM-430	CCW to East Pressure Equalization Fan	NA
24. CCM-431	CCW from East Pressure Equalization Fan	NA
25. CCM-432	CCW to West Pressure Equalization Fan	NA
26. CCM-433	CCW from West Pressure Equalization Fan	NA
27. SM-8*	Upper Containment Sample	NA
28. SM-10*	Upper Containment Sample	NA
29. SM-4*	Instrument Room Sample	NA
30. SM-6*	Instrument Room Sample	NA

NA - Manual Valve-Isolation time not applicable.
(1) - Includes motor operated valves which do not isolate automatically.
* - May be opened on an intermittent basis under administrative control.
** - Containment purge supply and exhaust isolation valves (VCR-101 through 106 and VCR-201 through 206) may be opened under Technical Specification 3/4.6.1.7.

TABLE 3.6-1 (Continued)

CONTAINMENT ISOLATION VALVES

VALVE NUMBER	FUNCTION	ISOLATION TIME IN SECONDS
A. PHASE "A" ISOLATION (Continued)		
25. DCR-620	Continuous Ventilation Drain to Holdup	$\leq$ 10
26. DCR-621	Continuous Ventilation Drain to Holdup	$\leq$ 10
27. ECR-10	Cont. H_2 Sample Return	$\leq$ 10
28. ECR-11	Cont. H_2 Sample - Air to Rec. E	$\leq$ 10
29. ECR-12	Cont. H_2 Sample - Air From Rec. E	$\leq$ 10
30. ECR-13	Cont. H_2 Sample - Low. Cont. Vol.	$\leq$ 10
31. ECR-14	Cont. H_2 Sample - Low. Cont. Vol.	$\leq$ 10
32. ECR-15	Cont. H_2 Sample - Up. Cont. Vol.	$\leq$ 10
33. ECR-16	Cont. H_2 Sample - Up. Cont. Vol.	$\leq$ 10
34. ECR-17	Cont. H_2 Sample - Air to Rec. W	$\leq$ 10
35. ECR-18	Cont. H_2 Sample - Air from Rec. W	$\leq$ 10
36. ECR-19	Cont. H_2 Sample - Cont. Dome Vol.	$\leq$ 10

TABLE 3.6-1 (Continued)

CONTAINMENT ISOLATION VALVES

VALVE NUMBER		FUNCTION	ISOLATION TIME IN SECONDS
A.	PHASE "A" ISOLATION (Continued)		
37.	ECR-20	Cont. H_2 Sample-Return	≤10
38.	ECR-21	Cont. H_2 Sample - Air to Rec. E.	≤10
39.	ECR-22	Cont. H_2 Sample - Air Fr. Rec. E.	≤10
40.	ECR-23	Cont. H_2 Sample - Low. Cont. Vol.	≤10
41.	ECR-24	Cont. H_2 Sample - Low. Cont. Vol.	≤10
42.	ECR-25	Cont. H_2 Sample - Up. Cont. Vol.	≤10
43.	ECR-26	Cont. H_2 Sample - Up. Cont. Vol.	≤10
44.	ECR-27	Cont. H_2 Sample - Air to Rec. W.	≤10
45.	ECR-28	Cont. H_2 Sample - Air Fr. Rec. W.	≤10
46.	ECR-29	Cont. H_2 Sample - Cont. Dome Vol.	≤10
47.	ECR-416	PAS Containment Sump Sample	≤10
48.	ECR-417	PAS Containment Sump Sample	≤10
49.	ECR-496	PAS Waste Liquid and Gas Return	≤10
50.	ECR-497	PAS Waste Liquid and Gas Return	≤10
51.	ECR-535	PAS Containment Gas Sample	≤10
52.	ECR-536	PAS Containment Gas Sample	≤10
53.	GCR-301	N_2 Supply to Pressurizer Relief Tank	≤10
54.	GCR-314	N_2 Supply to Accumulators	≤10

TABLE 3.6-1 (Continued)

CONTAINMENT ISOLATION VALVES

VALVE NUMBER	FUNCTION	ISOLATION TIME IN SECONDS
A. PHASE "A" ISOLATION (Continued)		
55. 1CR-5	Accumulators Sample	≤ 10
56. 1CR-6	Accumulators Sample	≤ 10
57. MCR-251#	Sample Line from Steam Gen. Outlet #1	≤ 10
58. MCR-252#	Sample Line from Steam Gen. Outlet #1	≤ 10
59. MCR-253#	Sample Line from Steam Gen. Outlet #3	≤ 10
60. MCR-254#	Sample Line from Steam Gen. Outlet #4	≤ 10
61. NCR-105	Hot Leg Sample	≤ 10
62. NCR-106	Hot Leg Sample	≤ 10
63. NCR-107	PRZ Liquid Sample	≤ 10
64. NCR-108	PRZ Liquid Sample	≤ 10
65. NCR-109	PRZ Steam Sample	≤ 10
66. NCR-110	PRZ Steam Sample	≤ 10

TABLE 3.6-1 (Continued)

CONTAINMENT ISOLATION VALVES

VALVE NUMBER	FUNCTION	ISOLATION TIME IN SECONDS
B. PHASE "B" ISOLATION (Continued)		
12. WCR-901	NESW to Low. Containment Vent #1	≤10
13. WCR-903	NESW from Low. Containment Vent #1	≤10
14. WCR-905	NESW to Low. Containment Vent #2	≤10
15. WCR-907	NESW from Low. Containment Vent #2	≤10
16. WCR-909	NESW to Low. Containment Vent #3	≤10
17. WCR-911	NESW from Low. Containment Vent #3	≤10
18. WCR-913	NESW to Low. Containment Vent #4	≤10
19. WCR-915	NESW from Low Containment Vent #4	≤10
20. WCR-921	NESW to Up. Containment Vent #1	≤10
21. WCR-923	NESW from Up. Containment Vent #1	≤10
22. WCR-925	NESW to Up. to Containment Vent #2	≤10
23. WCR-927	NESW from Up. Containment Vent #2	≤10

TABLE 3.6-1 (Continued)

CONTAINMENT ISOLATION VALVES

VALVE NUMBER	FUNCTION	ISOLATION TIME IN SECONDS
B. PHASE "B" ISOLATION (Continued)		
24. WCR-929	NESW to Up. Containment Vent #3	≤10
25. WCR-931	NESW from Up. Containment Vent #3	≤10
26. WCR-933	NESW to Up. Containment Vent #4	≤10
27. WCR-935	NESW from Up. Containment Vent #4	≤10
28. WCR-941	NESW to RCP Motor Air Cooler	≤10
29. WCR-942	NESW to RCP Motor Air Cooler	≤10
30. WCR-943	NESW to RCP Motor Air Cooler	≤10
31. WCR-944	NESW to RCP Motor Air Cooler	≤10
32. WCR-945	NESW from RCP Motor Air Cooler	≤10
33. WCR-946	NESW from RCP Motor Air Cooler	≤10
34. WCR-947	NESW from RCP Motor Air Cooler	≤10
35. WCR-948	NESW from RCP Motor Air Cooler	≤10
36. WCR-951	NESW to RCP Motor Air Cooler Vent #1	≤10
37. WCR-952	NESW to RCP Motor Air Cooler Vent #2	≤10
38. WCR-953	NESW to RCP Motor Air Cooler Vent #3	≤10
39. WCR-954	NESW to RCP Motor Air Cooler Vent #4	≤10

TABLE 3.6-1 (Continued)

CONTAINMENT ISOLATION VALVES

VALVE NUMBER	FUNCTION	ISOLATION TIME IN SECONDS
B. PHASE "B" ISOLATION (Continued)		
40. WCR-955	NESW from RCP Motor Air Cooler Vent #1	≤10
41. WCR-956	NESW from RCP Motor Air Cooler Vent #2	≤10
42. WCR-957	NESW from RCP Motor Air Cooler Vent #3	≤10
43. WCR-958	NESW from RCP Motor Air Cooler Vent #4	≤10
44. WCR-961	NESW to Instr. Rm. East Vent	≤10
45. WCR-963	NESW from Instr. Rm. West Vent	≤10
46. WCR-965	NESW to Instr. Rm. East Vent	≤10
47. WCR-967	NESW from Instr. Rm. West Vent	≤10
48. WCR-900	NESW to RCP Lower Containment Vent #1	≤10
49. WCR-902	NESW from Lower Containment Vent #1	≤10
50. WCR-904	NESW to RCP Lower Containment Vent #2	≤10
51. WCR-906	NESW from Lower Containment Vent #2	≤10
52. WCR-908	NESW to RCP Lower Containment Vent #3	≤10
53. WCR-910	NESW from Lower Containment Vent #3	≤10
54. WCR-912	NESW to RCP Lower Containment Vent #4	≤10
55. WCR-914	NESW from Lower Containment Vent #4	≤10

TABLE 3.6-1 (Continued)

CONTAINMENT ISOLATION VALVES

VALVE NUMBER	FUNCTION	ISOLATION TIME IN SECONDS
D. MANUAL ISOLATION VALVES[1] (Continued)		
3. 1CM-250	Boron Injection Inlet	NA
4. 1CM-251	Boron Injection Inlet	NA
5. 1CM-260	Safety Injection Inlet	NA
6. 1CM-265	Safety Injection Inlet	NA
7. 1CM-305	RHR Suction From Sump	NA
8. 1CM-306	RHR Suction From Sump	NA
9. 1CM-311#	RHR to RC Hot Legs	NA
10. 1CM-321#	RHR to RC Hot Legs	NA
E. OTHER		
1. CS-442-1	Seal Wtr. to RCP #1	NA
2. CS-442-2	Seal Wtr. to RCP #2	NA
3. CS-442-3	Seal Wtr. to RCP #3	NA
4. CS-442-4	Seal Wtr. to RCP #4	NA

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TABLE 3.6-1 (Continued)

CONTAINMENT ISOLATION VALVES

VALVE NUMBER		FUNCTION	ISOLATION TIME IN SECONDS
E. OTHER (Continued)			
5.	SI-189	R. C. Relief Valve Vent Hole	NA
6.	PA-342	Service Air to Containment	NA
7.	NPX-151 VI	Dead Weight Calibrator	NA
8.	N-160	N_2 to R. C. Drain Tank	NA
9.	SM-1	Air Particle/Radio Gas Detect Return	NA
10.	N-102	N_2 To Accumulators	NA
11.	SI-171	Safety Injection Test Line	NA
12.	SI-172	Safety Injection Test Line	NA
13.	SI-194	Safety Injection Test Line	NA
14.	PW-275	Primary Wtr. to Pre. Relief Tank	NA
15.	CS-321	R.C.S. Charging	NA

TABLE 3.6-1 (Continued)

CONTAINMENT ISOLATION VALVES

VALVE NUMBER	FUNCTION	ISOLATION TIME IN SECONDS
E. OTHER (Continued)		
16. SF-152	Refueling Wtr. to Refuel Cavity	NA
17. SF-154	Refueling Wtr. to Refuel Cavity	NA
18. SF-159	Refueling Cavity Drain	NA
19. SF-160	Refueling Cavity Drain	NA
20. N-159	N_2 to Prz. Relief Tank	NA
21. CCW-135	CCW to Reactor Supports	NA
22. CA-181-N	Weld Channel Supply Air	NA
23. CA-181-S	Weld Channel Supply Air	NA
24. SM-8*	Upper Cont. Grab Sample	NA
25. SM-10*	Upper Cont. Grab Sample	NA

TABLE 3.6-1 (Continued)

CONTAINMENT ISOLATION VALVES

VALVE NUMBER	FUNCTION	ISOLATION TIME IN SECONDS
E. OTHER (Continued)		
26. PPP-300	Instrument Penetration	NA
27. PPP-301	Instrument Penetration	NA
28. PPP-302	Instrument Penetration	NA
29. PPP-303	Instrument Penetration	NA
30. PPA-310 and PPA-311	Instrument Penetration	NA
31. PPA-312 and PPA-313	Instrument Penetration	NA
32. Blind Flange	Fuel Transfer Penetration	NA
33. Blind Flange	Ice Condenser Ice Supply	NA
34. Blind Flange	Ice Condenser Ice Return	NA
35. Blind Flange	In-Core Flux Thimble Access	NA

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 95 TO FACILITY OPERATING LICENSE NO. DPR-58

AND AMENDMENT NO. 81 TO FACILITY OPERATING LICENSE NO. DPR-74

INDIANA AND MICHIGAN ELECTRIC COMPANY

DONALD C. COOK NUCLEAR PLANT, UNIT NOS. 1 AND 2

DOCKET NOS. 50-315 AND 50-316

INTRODUCTION

By letters dated October 19, 1984, and May 31, 1985, the licensee requested revisions to the Technical Specifications for the D. C. Cook Nuclear Plant, Units 1 and 2. Additional justification for the changes was provided by letters dated October 18, 1985 and February 7, 1986. A description of each proposed change is provided below, along with our assessment of the change.

Unit 1 Changes

3/4.6 Containment Systems

The licensee has proposed to add a footnote to Surveillance Requirement 4.6.1.1.a.1 to specify that valves, blind flanges, and deactivated automatic valves which are located inside the containment and are locked, sealed, or otherwise secured in the closed position, shall be verified closed during each cold shutdown, rather than once per 31 days as currently required. The licensee claims that access to these valves inside containment is undesirable on a monthly basis when the unit is operating due to ALARA considerations. The footnote the licensee proposes to add was not included in early drafts of the Standard Technical Specifications (STSs) on which Unit 1 Technical Specifications (TSs) were based, but has been incorporated in more recent revisions of the STSs. This change would make this Unit 1 TS consistent with the Unit 2 TS as well as the STS (NUREG-0452, Revision 4.)

The staff has considered the proposed change and finds it to be acceptable. We note that while the change can result in some increase in the probability or consequences of a previously analyzed accident due to the decreased surveillance frequency for the affected valves, the reduction in safety margin is within acceptable limits. This is evidenced by consistency with the STS. On this basis, we find the change to be acceptable.

3/4.6.3 Containment Isolation Valves

The licensee has proposed to delete the present requirements to (1) demonstrate the operability of each power operated or automatic valve testable

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during plant operation at least once per 92 days (Surveillance Requirement 4.6.3.1.1.a) and (2) demonstrate the operability of each power operated or automatic valve, and certain manual valves during cold shutdown or refueling at least once per 18 months (Surveillance Requirements 4.6.3.1.2.d and e, respectively). The licensee would, instead, test the affected valves pursuant to TS 4.0.5, i.e., in accordance with Section XI of the ASME Boiler and Pressure Vessel Code. The testing requirements for each valve under Specification 4.0.5 have been incorporated in a new section 4.6.3.1.3. The proposed change would make the Unit 1 TS consistent with the Unit 2 TS, as well as the STS.

The staff has assessed the impact of this change on the method and frequency of valve testing. We find that the full stroke valve test performed under ASME Section XI (the IST program) is equivalent to that required by the current TS with the following exceptions: (1) valves ICM-250 and 251 (Boron injection inlet) are tested once every 92 days under the current TS but are tested only during cold shutdown under the IST program, and (2) valves ICM-111 (RHR to RC cold legs) and NPX-151-VI (dead weight tester) are tested once every 92 days under the current TS but are not tested at all under the IST program. The licensee has justified the less frequent test interval for the boron injection inlet valves on the basis that these valves cannot be cycled during normal plant operation without introducing boron into a non-heat traced line. (The boron could crystalize and plug the line.) This justification has been accepted by the staff in its review of the IST program for D. C. Cook Unit 1. Omission of valve ICM-111 from the IST program was also considered acceptable by the staff on the basis that the valve is closed during plant operation and is only used for normal shutdown cooling (use of a parallel flow path is assumed in plant accident analyses). We consider this to be adequate justification for deleting the current TS requirement to test these valves once every 92 days. With regard to valve NPX-151-VI, this valve is an instrument valve which performs a containment isolation function. The valve is normally closed and is required to remain closed to perform its safety function. The licensee has stated that NPX-151-VI is tested per Appendix J, and that the valve has been included in the second 10 year interval IST program. We consider this to constitute acceptable testing requirements for the valve.

In summary, the staff considers the reduction in safety margin due to the decreased testing requirements for the aforementioned valves to be within acceptable limits. On this basis, we find the proposed change to be acceptable.

The licensee has also proposed to delete the column entitled "Testable during Plant Operation" from the table of containment isolation valves (Table 3.6-1) on the basis that this information is now given in the IST program for D. C. Cook, and is no longer needed in the TS. The proposed change would make the format of the Unit 1 TS consistent with the Unit 2 TS as well as the STS. The purpose of indicating which valves are testable during operation is to identify which valves are to be tested under Surveillance Requirement 4.6.3.1.1.a. As discussed above, the licensee has proposed, and staff considers acceptable, the deletion of this surveillance requirement. We therefore find the deletion of the subject column to be acceptable.

Unit 2 Changes

3/4.6.3 Containment Isolation Valves

The licensee has proposed to modify Table 3.6-1 by adding asterisks for the containment isolation valves in the upper containment grab sample line (valves SM-8 and SM-10), to denote that these valves may be opened on an intermittent basis under administrative controls. As the TS is currently written, the opening of these valves requires entry into an action statement and associated administrative tasks. The licensee has stated that rather than enter the action statement, plant practice has been to send personnel into upper containment when it is necessary to obtain grab samples, even if the unit is at power. The proposed change would allow upper containment samples to be taken without having personnel access upper containment, thereby enhancing the ALARA program. The proposed change would also make the Unit 1 and Unit 2 TS consistent.

The staff has considered the proposed change and finds it to be acceptable on the basis that the increase in risk posed by intermittent use of the valves is small compared to the reduction in occupational exposure which could be achieved by reducing personnel entries to obtain air samples.

The licensee has also proposed to delete from Table 3.6-1 14 valves inadvertently left in the TS in Amendment No. 42, and to sequentially renumber certain other valves. The valves to be deleted are NSW-415-1, 2, 3, 4; NSW-419-1, 2, 3, 4; NSW-224-1, 2, 3, 4 and NSW-417-3, 4. These valves are already identified elsewhere in Table 3.6-1 by revised valve numbers. The staff has reviewed the proposed change and concurs in the licensee's assessment that the change is administrative in nature. On this basis, we find it to be acceptable.

Finally, the licensee has proposed to delete from certain pages of Table 3.6-1 a footnote stating "This Technical Specification will not be effective until after the 1982 refueling outage." This footnote reflected a planned installation of replacement valves during the 1982 refueling outage. The valve replacement was completed at that time, eliminating the need for the footnote. On this basis, we consider deletion of the footnote to be acceptable.

ENVIRONMENTAL CONSIDERATION

These amendments involve a change in the installation or use of the facilities' components located within the restricted areas as defined in 10 CFR 20 and a change in surveillance requirements. The staff has determined that these amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that these amendments involve no significant hazards consideration, and there has been no public comment on such finding. Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR Sec 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

CONCLUSION

The Commission made a proposed determination that the amendments involve no significant hazards consideration which was published in the Federal Register (49 FR 50805) on December 31, 1984, and consulted with the state of Michigan. No public comments were received, and the state of Michigan did not have any comments.

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors: D. Wigginton, PWR#4
R. Palla

Dated: April 23, 1986