Docket Nos. 50-315 and 50-316

> Mr. John Dolan, Vice President Indiana and Michigan Electric Company c/o American Electric Power Service Corporation 1 Riverside Plaza Columbus, Ohio 43216

Dear Mr. Dolan:

The Commission has issued the enclosed Amendment No.96 to Facility Operating License No. DPR-58 and Amendment No. 83 to Facility Operating License No. DPR-74 for the Donald C. Cook Nuclear Plant, Unit Nos. 1 and 2. The amendments consist of changes to the Technical Specifications in response to your application transmitted by letter dated November 4, 1985.

These amendments revise the Technical Specification by adding provisions for the containment hydrogen monitor in response to NUREG-0737 Item II.F.1.6.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next bi-weekly Federal Register notice.

Sincerely,

151

D. L. Wigginton, Project Manager PWR Project Directorate #4 Division of PWR Licensing-A, NRR

PDR

1 1 JUN 1986

Enclosures:

- 1. Amendment No. 96 to DPR-58
- 2. Amendment No. 83 to DPR-74
- 3. Safety Evaluation

cc: w/enclosures See next page

PWR#4:DPWR-A	PWR#4; DPWR-A	0ELD PWR#4:1	PWR#4:DPWR-A			
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Mr. John Dolan Indiana and Michigan Electric Company

cc:

Mr. M. P. Alexich
Vice President
Nuclear Operations
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Special Assistant to the Governor Room 1 - State Capitol Lansing, Michigan 48909

Nuclear Facilities and Environmental Monitoring Section Office Division of Radiological Health Department of Public Health 3500 N. Logan Street Post Office Box 30035 Lansing, Michigan 48909 Donald C. Cook Nuclear Plant

The Honorable John E. Grotberg United States House of Representatives Washington, DC 20515

Regional Administrator, Region III U.S. Nuclear Regulatory Commission 799 Roosevelt Road Glen Ellyn, Illinois 60137

J. Feinstein American Electric Power Service Corporation 1 Riverside Plaza Columbus, Ohio 43216

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DISTRIBUTION: Docket File NRC PDR L PDR NSIC PRC System PWR#4 R/F MDuncan DWigginton BJYoungblood R/F EJordan BGrimes JPartlow ACRS (10) OELD LHarmon TBarnhart (8) WJones JMilhoan OPA LFMB EButcher	AMENDMENT	NO.	96 TO 1	DC COOK	UNITS	1 AND	2

UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555



INDIANA AND MICHIGAN ELECTRIC COMPANY

DOCKET NO. 50-315

DONALD C. COOK NUCLEAR PLANT UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 96 License No. DPR-58

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Indiana and Michigan Electric Company (the licensee) dated November 4, 1985, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
- Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 2.C.(2) of Facility Operating License No. DPR-58 is hereby amended to read as follows:

8606230523 860611 PDR ADOCK 05000315 PDR (2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 83, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

- 3. The Technical Specifications are to be effective within 60 days of issuance.
- 4. This license amendment is effective as of the date of its issuance.

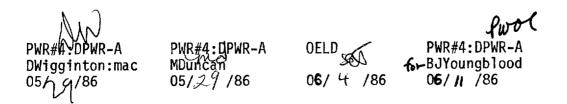
FOR THE NUCLEAR REGULATORY COMMISSION

151

B. J. Youngblood, Director PWR Project Directorate #4 Division of PWR Licensing-A, NRR

Attachment: Changes to the Technical Specifications

Date of Issuance: June 11, 1986



- 2 -

UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

UCLEAR REGULA



DOCKET NO. 50-316

DONALD C. COOK NUCLEAR PLANT UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 83 License No. DPR-74

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Indiana and Michigan Electric Company (the licensee) dated November 4, 1985, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
- Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 2.C.(2) of Facility Operating License No. DPR-74 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 96, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

- 3. The Technical Specifications are to be effective within 60 days of issuance.
- 4. This license amendment is effective as of the date of its issuance.

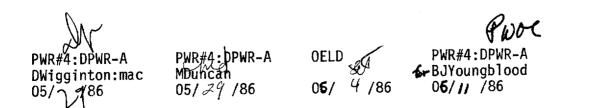
FOR THE NUCLEAR REGULATORY COMMISSION

151

B. J. Youngblood, Director PWR Project Directorate #4 Division of PWR Licensing-A, NRR

Attachment: Changes to the Technical Specifications

Date of Issuance: June 11, 1986



- 2 -

ATTACHMENT TO LICENSE AMENDMENTS AMENDMENT NO. 96 FACILITY OPERATING LICENSE NO. DPR-58 AMENDMENT NO. 83 FACILITY OPERATING LICENSE NO. DPR-74 DOCKET NOS. 50-315 AND 50-316

Revise Appendix A as follows:

Remove Pages	Insert Pages
<u>Unit 1</u>	<u>Unit 1</u>
3/4 6-23	3/4 6 23

 Unit 2
 Unit 2

 3/4 6-33
 3/4 6 33

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CONTAINMENT SYSTEMS

3/4.6.4 COMBUSTIBLE GAS CONTROL

HYDROGEN ANALYZERS

LIMITING CONDITION FOR OPERATION

3.6.4.1 Two containment hydrogen analyzers shall be OPERABLE.

APPLICABILITY: MODES 1 and 2

ACTION:

a. With one hydrogen analysis device inoperable, restore the inoperable analysis device to OPERABLE status within 30 days or be in at least HOT STANDBY within the next 6 hours.

b. With both hydrogen analysis devices inoperable, restore at least one analysis device to OPERABLE status within 72 hours or be in at least HOT STANDBY within the next 6 hours.

SURVEILLANCE REQUIREMENTS

4.6.4.1 Each hydrogen analysis device shall be demonstrated OPERABLE at least once per 92 days on a STAGGERED TEST BASIS by performing a CHANNEL CALIBRATION using a four percent and fifteen percent nominal hydrogen gas, balance nitrogen.

3/4 6-23

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4.6.4.1 Each hydrogen analysis device shall be demonstrated OPERABLE at least once per 92 days on a STAGGERED TEST BASIS by performing a CHANNEL CALIBRATION using a four percent and fifteen percent nominal hydrogen gas, balance nitrogen.

3/4 6-33



UNITED STATES

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION RELATED TO AMENDMENT NO. 96 TO FACILITY OPERATING LICENSE NO. DPR-58 AND AMENDMENT NO. 83 TO FACILITY OPERATING LICENSE NO. DPR-74 INDIANA AND MICHIGAN ELECTRIC COMPANY

DONALD C. COOK NUCLEAR PLANT, UNIT NOS. 1 AND 2

DOCKET NOS. 50-315 AND 50-316

INTRODUCTION

By letter dated November 4, 1985, the licensee proposed certain changes to Technical Specification (TS) 3/4.6.4.1 concerning containment hydrogen monitors. This change request constitutes the licensee's response to NUREG-0737 Item II.F.1.6, Containment Hydrogen Monitors, and the guidance provided in Generic Letter (GL) 83-37 which recommended that licensees provide continuous indication of hydrogen concentration in the control room with a capability to measure hydrogen concentrations over a range of O to 10%. Generic Letter 83-37 provided guidance on technical specifications related to containment hydrogen monitors as noted below:

"Two independent containment hydrogen monitors should be operable at all times when the reactor is operating in Power Operation or Startup modes. LCO for these monitors should include the requirement that with one hydrogen monitor inoperable, the monitor should be restored to operable status within 30 days or the plant should be brought to at least hot standby condition within the next 6 hours. If both monitors are inoperable, at least one monitor should be restored to operable status within 72 hours or the plant should be brought to at least hot standby condition within the next 6 hours.

Generic letter 83-37 also provides surveillance requirements for calibration testing of hydrogen monitors at 92 day intervals with two sample gas mixtures, one containing one volume percent hydrogen (balance nitrogen) and the other containing four volume percent hydrogen (balance nitrogen).

Additional guidance on the matter of containment hydrogen monitoring is provided in Regulatory Guide 1.97 which recommends a measurement capability over a range of 0-30% hydrogen for ice condenser containments.

EVALUATION & CONCLUSIONS

In accordance with the recommendations of Item II.F.1.6 and Regulatory Guide 1.97 the licensee has installed hydrogen monitors to provide continuous indication of hydrogen concentrations inside containment following an

accident. The hydrogen monitors have a range of 0 to 30 volume percent hydrogen. The licensee has also proposed technical specifications which fully comply with the guidance of Generic Letter 83-37 insofar as the limiting conditions for operation and action items are concerned.

However, with regard to the surveillance requirements for the hydrogen monitors, the licensee has proposed to calibrate the monitors using sample gas mixtures containing four and fifteen volume percent hydrogen (balance nitrogen) rather than the recommended compositions of one and four volume percent hydrogen. The licensee's rationale for choosing alternative sample gas mixtures is that the new hydrogen monitors have a wider operating range and would be better calibrated using mixtures spanning a wider range of hydrogen concentrations than by the samples recommended in GL 83-37 and the Westinghouse Standard Technical Specifications. The staff has considered the licensee's proposal and has discussed this matter with the vendor for the hydrogen monitors, Comsip Inc., and concludes that the calibration approach suggested by the licensee is acceptable. Therefore, the staff finds the proposed revisions to TS 3/4.6.4.1 to be acceptable.

ENVIRONMENTAL CONSIDERATION

These amendments involve a change in the installation or use of the facilities' components located within the restricted areas as defined in 10 CFR 20. The staff has determined that these amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that these amendments involve no significant hazards consideration, and there has been no public comment on such finding. Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR Sec 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

CONCLUSION

The Commission made a proposed determination that the amendments involve no significant hazards consideration which was published in the <u>Federal Register</u> (50 FR 51625) on December 18, 1985, and consulted with the state of Michigan. No public comments were received, and the state of Michigan did not have any comments.

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors: C. Tinkler

Dated: June 11, 1986