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Docket Nos. 50-315
and 50-316

Mr. John Dolan, Vice President
Indiana and Michigan Electric Company
Post Office Box 18
Bowling Green Station
New York, New York 10004

Dear Mr. Dolan:

The Commission has issued the enclosed Amendment No. 72 to Facility Operating License No. DPR-58 and Amendment No. 54 to Facility Operating License No. DPR-74 for the Donald C. Cook Nuclear Plant, Unit Nos. 1 and 2, respectively. The amendments consist of changes to the Technical Specifications in response to your application transmitted by letter dated April 14, 1983.

These amendments revise the Technical Specifications for oxygen content in the waste decay tanks and makes several administrative corrections which were inadvertently overlooked in the Technical Specifications issued by Amendment Nos. 69 and 51 dated February 7, 1983 for Unit Nos. 1 and 2, respectively.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

ORIGINAL SIGNED

David L. Wigginton, Project Manager
Operating Reactors Branch #1
Division of Licensing

Enclosures:
1. Amendment No. 72 to DPR-58
2. Amendment No. 54 to DPR-74
3. Safety Evaluation
4. Notice of Issuance

cc w/enclosures:
See next page

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PDR ADOCK 05000315
P PDR

* see AE of Amendment and p.2 of Notice | No legal objection
OELD W.F.P. *
04/22/83

OFFICE	ORB#1:DL	ORB#1:DL	ETSB	ETSB	AD/RP	ORB#1:DL	AD/OR:DL
SURNAME	CParrish	DWigginton:dm	CWillis	WGammill	DMuller	for SVarga	GLainas
DATE	04/21/83	04/21/83	04/21/83	04/21/83	04/22/83	04/22/83	04/22/83

Mr. John Dolan
Indiana and Michigan Electric Company

cc: Mr. M. P. Alexich
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Donald C. Cook Nuclear Plant
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U. S. Nuclear Regulatory Commission
Resident Inspectors Office
7700 Red Arrow Highway
Stevensville, Michigan 49127

Gerald Charnoff, Esquire
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1800 M Street, N.W.
Washington, D. C. 20036

Honorable James Bemenek, Mayor
City of Bridgman, Michigan 49106

U.S. Environmental Protection Agency
Region V Office
ATTN: EIS COORDINATOR
230 South Dearborn Street
Chicago, Illinois 60604

Maurice S. Reizen, M.D.
Director
Department of Public Health
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The Honorable Tom Corcoran
United States House of Representatives
Washington, D. C. 20515

James G. Keppler
Regional Administrator - Region III
U. S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, Illinois 60137

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

INDIANA AND MICHIGAN ELECTRIC COMPANY

DOCKET NO. 50-315

DONALD C. COOK NUCLEAR PLANT UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 72
License No. DPR-58

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Indiana and Michigan Electric Company (the licensee) dated April 14, 1983, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-58 is hereby amended to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 72 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 25, 1983

ATTACHMENT TO LICENSE AMENDMENTS

AMENDMENT NO. 72 TO FACILITY OPERATING LICENSE NO. DPR-58

DOCKET NO. 50-315

Revise Appendix A as follows:

Remove Pages	Insert Pages
3/4 3-63	3/4 3-63
3/4 3-65	3/4 3-65
3/4 11-13	3/4 11-13
1-9	1-9
6-12	6-12
6-13	6-13

TABLE 3.3-13

Radioactive Gaseous Effluent Monitoring Instrumentation

Instrument	Minimum Channels Operable	Applicability	Action
1. Waste Gas Holdup System Explosive Gas Monitoring System			
a. Hydrogen Monitor	(1)	**	30
b. Oxygen Monitor	(2)	**	29
2. Condenser Evacuation System			
a. Noble Gas Activity Monitor	(1)	****	28
b. Flow Rate Monitor	(1)	****	27
3. Unit Vent, Auxiliary Building Ventilation System			
a. Noble Gas Activity Monitor	(1)	*	28
b. Iodine Sampler Cartridge	(1)	*	32
c. Particulate Sampler Filter	(1)	*	32
d. Effluent System Flow Rate Measuring Device	(1)	*	27
e. Sampler Flow Rate Measuring Device	(1)	*	27
4. Containment Purge System ***			
a. Noble Gas Activity Monitor	(1)	****[1]	31
b. Particulate Sampler	(1)	****[1]	32
5. Waste Gas Holdup System			
a. Noble Gas Activity Monitor Providing Alarm and Termination of Gas Decay Tank Releases	(1)	****[2]	33
6. Gland Seal Exhaust			
a. Noble Gas Activity Monitor	(1)	****	28
b. Flow Rate Monitor	(1)	****	27

TABLE 3.3-13 (Cont)

TABLE NOTATION

Action 27 With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, effluent releases via this pathway may continue for up to 30 days provided the flow rate is estimated at least once per 4 hours.

Action 28 With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, effluent releases via this pathway may continue for up to 30 days provided grab samples are taken at least once per 8 hours and these samples are analyzed for gross activity within 24 hours..

Action 29 With the number of channels OPERABLE one less than required by the Minimum Channels OPERABLE requirement, operation of this system may continue for up to 30 days. With 2 channels inoperable, operation of this system may continue for up to 30 days, provided grab samples are taken and analyzed every 12 hours.

Action 30 With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, operation of this system may continue for up to 14 days, provided grab samples are taken and analyzed every 12 hours.

Action 31 With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirements, immediately suspend PURGING of radioactive effluents via this pathway.

Action 32 With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, effluent releases via the affected pathway may continue for up to 30 days provided samples required for weekly analysis are continuously collected with auxiliary sampling equipment as required in Table 4.11-2.

Action 33 With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, the contents of the tank(s) may be released to the environment for up to 14 days provided that prior to initiating the release:

a. At least two independent samples of the tank's contents are analyzed and,

b. At least two technically qualified members of the Facility Staff independently verify the release rate calculations and discharge valve lineups;

otherwise, suspend release of radioactive effluents via this pathway.

RADIOACTIVE EFFLUENTS

EXPLOSIVE GAS MIXTURE

LIMITING CONDITION FOR OPERATION

3.11.2.5 The concentration of oxygen in the waste gas holdup system shall be limited to $\leq$ 3% by volume if the hydrogen in the system is $\geq$ 4% by volume.

APPLICABILITY: At all times.

ACTION:

a. With the concentration of oxygen in the waste gas holdup system > 3% by volume but $\leq$ 4% by volume and containing $\geq$ 4% hydrogen, restore the concentration of oxygen to $\leq$ 3% or reduce the hydrogen concentration to < 4% within 96 hours.

b. With the concentration of oxygen in the waste gas holdup system or tank > 4% by volume and > 4% hydrogen by volume without delay suspend all additions of waste gases to the system or tank and reduce the concentration of oxygen to $\leq$ 3% or the concentration of hydrogen to $\leq$ 4% within 96 hours in the system or tank.

c. The provisions of Specification 3.0.3, 3.0.4 and 6.9.1.13 are not applicable.

SURVEILLANCE REQUIREMENTS

4.11.2.5 The concentration of oxygen in the waste gas holdup system shall be determined to within the above limits by continuously monitoring the waste gases in the waste gas holdup system with the oxygen monitors required OPERABLE by Table 3.3-13 of Specification 3.3.3.10.

TABLE 1.2

FREQUENCY NOTATION

NOTATION	FREQUENCY
S	At least once per 12 hours.
D	At least once per 24 hours.
W	At least once per 7 days.
M	At least once per 31 days
Q	At least once per 92 days.
SA	At least once per 184 days.
R	At least once per 549 days.
S/U	Prior to each reactor startup.
P	Completed prior to each release.
N.A.	Not applicable.

m. The PROCESS CONTROL PROGRAM and implementing procedures for modification of radioactive wastes at least once per 24 months.

n. The performance of activities required by the Quality Assurance Program to meet the criteria of Regulatory Guide 1.21, Rev. 1, June 1974 and Regulatory Guide 4.1, Rev. 1, April 1975 at least once per 12 months.

AUTHORITY

6.5.2.9 The NSDRC shall report to and advise the Vice Chairman, Engineering and Construction, AEPSC, on those areas of responsibility specified in Sections 6.5.2.7 and 6.5.2.8.

RECORDS

6.5.2.10 Records of NSDRC activities shall be prepared, approved and distributed as indicated below:

a. Minutes of each NSDRC meeting shall be prepared, approved and forwarded to the Vice Chairman, Engineering and Construction, AEPSC, within 14 days following each meeting.

b. Reports of reviews encompassed by Section 6.5.2.7 above, shall be prepared, approved and forwarded to the Vice Chairman, Engineering and Construction, AEPSC, within 14 days following completion of the review.

c. Audit reports encompassed by Section 6.5.2.8 above, shall be forwarded to the Vice Chairman, Engineering and Construction, AEPSC, and to the management positions responsible for the areas audited within 30 days after completion of the audit.

6.6 REPORTABLE OCCURRENCE ACTION

6.6.1 The following actions shall be taken for REPORTABLE OCCURRENCES:

a. The Commission shall be notified and/or a report submitted pursuant to the requirements of Specification 6.9.

b. Each REPORTABLE OCCURRENCE requiring 24 hour notification to the Commission shall be reviewed by the PNSRC and submitted to the NSDRC and the Chief, Nuclear Engineer.

6.7 SAFETY LIMIT VIOLATION

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

a. The facility shall be placed in at least HOT STANDBY within one hour.

b. The Safety Limit violation shall be reported to the Commission and to the Chairman of the NSDRC within 24 hours.

c. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the PNSRC. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems or structures, and (3) corrective action taken to prevent recurrence.

d. The Safety Limit Violation Report shall be submitted to the Commission, the Chairman of the NSDRC and the Vice President, Nuclear Engineering within 14 days of the violation.

6.8 PROCEDURES

6.8.1 Written procedures shall be established, implemented and maintained covering the activities referenced below:

a. The applicable procedures recommended in Appendix "A" of Regulatory Guide 1.33, November, 1972.

b. Refueling operations.

c. Surveillance and test activities of safety related equipment.

d. Security Plan implementation.

e. Emergency Plan implementation.

f. Fire Protection Program implementation.

g. PROCESS CONTROL PROGRAM implementation.

h. OFFSITE DOSE CALCULATION MANUAL implementation.

i. Quality Assurance Program for effluent and environmental monitoring using the guidance in Regulatory Guide 1.21, Rev. 1, June 1974 and Regulatory Guide 4.1, Rev. 1, April 1975.

6.8.2 Each procedure and administrative policy of 6.8.1 above, and changes thereto, shall be reviewed by the PNSRC and approved by the Plant Manager prior to implementation and reviewed periodically as set forth in administrative procedures.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

INDIANA AND MICHIGAN ELECTRIC COMPANY

DOCKET NO. 50-316

DONALD C. COOK NUCLEAR PLANT UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 54
License No. DPR-74

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Indiana and Michigan Electric Company (the licensee) dated April 14, 1983, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-74 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 54 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 25, 1983

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 54 TO FACILITY OPERATING LICENSE NO. DPR-74

DOCKET NO. 50-316

Revise Appendix A as follows:

Remove Pages	Insert Pages
3/4 3-59	3/4 3-59
3/4 3-61	3/4 3-61
3/4 11-13	3/4 11-13
XVI	XVI

TABLE 3.3-13

Radioactive Gaseous Effluent Monitoring Instrumentation

	Instrument	Minimum Channels Operable	Applicability	Action
1.	Waste Gas Holdup System Explosive Gas Monitoring System			
	a. Hydrogen Monitor	(1)	**	30
	b. Oxygen Monitor	(2)	**	29
2.	Condenser Evacuation System			
	a. Noble Gas Activity Monitor	(1)	****	28
	b. Flow Rate Monitor	(1)	****	27
3.	Unit Vent, Auxiliary Building Ventilation System			
	a. Noble Gas Activity Monitor	(1)	*	28
	b. Iodine Sampler Cartridge	(1)	*	32
	c. Particulate Sampler Filter	(1)	*	32
	d. Effluent System Flow Rate Measuring Device	(1)	*	27
	e. Sampler Flow Rate Measuring Device	(1)	*	27
4.	Containment Purge System ***			
	a. Noble Gas Activity Monitor	(1)	****[1]	31
	b. Particulate Sampler	(1)	****[1]	32
5.	Waste Gas Holdup System			
	a. Noble Gas Activity Monitor Providing Alarm and Termination of Gas Decay Tank Releases	(1)	****[2]	33
6.	Gland Seal Exhaust			
	a. Noble Gas Activity Monitor	(1)	****	28
	b. Flow Rate Monitor	(1)	****	27

TABLE 3.3-13 (Cont)

TABLE NOTATION

Action 27 With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, effluent releases via this pathway may continue for up to 30 days provided the flow rate is estimated at least once per 4 hours.

Action 28 With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, effluent releases via this pathway may continue for up to 30 days provided grab samples are taken at least once per 8 hours and these samples are analyzed for gross activity within 24 hours.

Action 29 With the number of channels OPERABLE one less than required by the Minimum Channels OPERABLE requirement, operation of this system may continue for up to 30 days. With 2 channels inoperable, operation of this system may continue for up to 30 days, provided grab samples are taken and analyzed every 12 hours.

Action 30 With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, operation of this system may continue for up to 14 days, provided grab samples are taken and analyzed every 12 hours.

Action 31 With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirements, immediately suspend PURGING of radioactive effluents via this pathway.

Action 32 With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, effluent releases via the affected pathway may continue for up to 30 days provided samples required for weekly analysis are continuously collected with auxiliary sampling equipment as required in Table 4.11-2.

Action 33 With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, the contents of the tank(s) may be released to the environment for up to 14 days provided that prior to initiating the release:

a. At least two independant samples of the tank's contents are analyzed and,

b. At least two technically qualified members of the Facility Staff independently verify the release rate calculations and discharge valve lineups;

otherwise, suspend release of radioactive effluents via this pathway.

RADIOACTIVE EFFLUENTS

EXPLOSIVE GAS MIXTURE

LIMITING CONDITION FOR OPERATION

3.11.2.5 The concentration of oxygen in the waste gas holdup system shall be limited to $\leq$ 3% by volume if the hydrogen in the system is $\geq$ 4% by volume.

APPLICABILITY: At all times.

ACTION:

a. With the concentration of oxygen in the waste gas holdup system > 3% by volume but $\leq$ 4% by volume and containing $\geq$ 4% hydrogen, restore the concentration of oxygen to $\leq$ 3% or reduce the hydrogen concentration to < 4% within 96 hours.

b. With the concentration of oxygen in the waste gas holdup system or tank > 4% by volume and > 4% hydrogen by volume without delay suspend all additions of waste gases to the system or tank and reduce the concentration of oxygen to $\leq$ 3% or the concentration of hydrogen to $\leq$ 4% within 96 hours in the system or tank..

c. The provisions of Specification 3.0.3, 3.0.4 and 6.9.1.13 are not applicable.

SURVEILLANCE REQUIREMENTS

4.11.2.5 The concentration of oxygen in the waste gas holdup system shall be determined to within the above limits by continuously monitoring the waste gases in the waste gas holdup system with the oxygen monitors required OPERABLE by Table 3.3-13 of Specification 3.3.3.10.

ADMINISTRATIVE CONTROLS

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 72 TO FACILITY OPERATING LICENSE NO. DPR-58

AND AMENDMENT NO. 54 TO FACILITY OPERATING LICENSE NO. DPR-74

INDIANA AND MICHIGAN ELECTRIC COMPANY

DONALD C. COOK NUCLEAR PLANT UNIT NOS. 1 AND 2

DOCKET NOS. 50-315 AND 50-316

Introduction

By letter dated April 14, 1983, Indiana and Michigan Electric Company requested Radiological Effluent Technical Specification changes to Facility Operating License Nos. DPR-58 and DPR-74. The subject changes involve Sections 3.3.3.10 and 3.11.2.5 of the Technical Specifications for Unit Nos. 1 and 2. The licensee has proposed to amend Section 3.3.3.10 of the Technical Specifications by changing Table 3.3-13, as follows:

(a) Provide for operation of the waste gas holdup system for up to 30 days (rather than the present 14 days) with less than two oxygen monitors operable.

(b) Include for clarity the term "Unit Vent" in the heading to read "Unit Vent, Auxiliary Building Ventilation System," since the Auxiliary Building Ventilation System is part of the Unit Vent System.

(c) Clarify the ACTION regarding sampling and analysis when iodine sampler cartridges and particulate samplers are not operable to indicate only sampling and analyis of charcoal samples and particulate samples, respectively, of the auxiliary building vent.

Section 3.11.2.5 of the Technical Specifications provides requirements concerning the concentrations of oxygen and hydrogen in the waste gas holdup system. The specification is provided to ensure that the concentration of potentially explosive gas mixtures contained in the waste gas holdup system is maintained below the flammability limits of hydrogen and oxygen. Maintaining the concentration of hydrogen and oxygen below the flammability limit provides assurance that the release of radioactive materials will be controlled in conformance with the requirements of General Design Criterion 60 of Appendix A to 10 CFR Part 50.

Evaluation

The model Radiological Effluent Technical Specifications (RETS) described in NUREG-0472 are based on systems that meet the requirements of the Standard Review Plan (SRP), NUREG-0800. The Donald C. Cook Nuclear Plant Explosive Gas Monitoring Systems does not meet the requirements of SPR 11.3 of NUREG-0800 for dual hydrogen analyzers with automatic control functions to preclude the formation or buildup of explosive hydrogen-oxygen mixtures. Therefore, Technical Specification 3.11.2.5 and the part of Technical Specification 3.3.3.10 that addresses the Explosive Gas Monitoring System are designed to afford a degree of protection against a hydrogen-oxygen explosion similar to the provisions of the model RETS described in NUREG-0472.

Technical Specification 3.11.2.5 provides that the concentration in the waste gas holdup system is to be limited at all times to less than or equal to 2% oxygen if the hydrogen in the system is greater than or equal to 4% by volume, and that if the concentration in the waste gas holdup system is outside this limit the concentration is to be reduced to within this limit within 48 hours. If the concentration of either hydrogen or oxygen is kept below a concentration of 4% by volume, any gas mixture within the system will be below the flammability limit. The proposed change to Technical Specification 3.11.2.5 would limit the concentration in the waste gas holdup system to less than or equal to 3% oxygen by volume if the hydrogen in the system is greater than or equal to 4% by volume. This provides a margin between 3% and 4% oxygen by volume outside the flammability limit. The proposed change to Technical Specification 3.11.2.5 would also provide that if the concentration in the waste gas holdup system is outside the limit, the concentration is to be reduced to within the limit within 96 hours rather than within 48 hours.

Technical Specification 3.3.3.10 provides, in part, that the waste gas holdup system may be operated for up to 14 days with less than two oxygen monitors operable, but that with no oxygen monitor operable grab samples must be taken and analyzed every 12 hours. The proposed change to Technical Specification would allow for operation of the waste gas holdup system as described above for 30 days rather than for 14 days. With the proposed limit, as determined under the proposed amendment by operable hydrogen and oxygen monitors or with no operable hydrogen or oxygen monitor, the prescribed periodic sampling and analysis, adequate protection is afforded against a hydrogen-oxygen explosion and assurance is provided that the release of radioactive materials will be controlled in conformance with the requirements of General Design Criteria 60 of Appendix A to 10 CFR Part 50.

The other proposed amendments to Section 3.3.3.10 are for clarification only.

Summary

In view of the above considerations, we have concluded that the proposed amendment to Sections 3.3.3.10 and 3.11.2.5 of the Technical Specifications for Unit Nos. 1 and 2 are acceptable.

Environmental Consideration

We have determined that the amendments do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendments involve an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of these amendments.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendments do not involve a significant increase in the probability or consequences of an accident previously evaluated, do not create the possibility of an accident of a type different from any evaluated previously, and do not involve a significant reduction in a margin of safety, the amendments do not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Dated: April 25, 1983

Principal Contributor:

C. Nichols

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NOS. 50-315 AND 50-316

INDIANA AND MICHIGAN ELECTRIC COMPANY

NOTICE OF ISSUANCE OF AMENDMENTS TO FACILITY OPERATING LICENSES

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 72 to Facility Operating License No. DPR-58, and Amendment No. 54 to Facility Operating License No. DPR-74 issued to Indiana and Michigan Electric Company (the licensee), which revised Technical Specifications for operation of the Donald C. Cook Nuclear Plant, Unit Nos. 1 and 2 (the facilities) located in Berrien County, Michigan. The amendments are effective as of the date of issuance.

The amendments revise the Technical Specifications for oxygen content in the waste decay tanks and makes several administrative corrections which were inadvertently overlooked in the Technical Specifications issued by Amendment Nos. 69 and 51 dated February 7, 1983 for Units 1 and 2, respectively.

The application for the amendments complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments. Prior public notice of these amendments was not required since the amendments do not involve a significant hazards consideration.

The Commission has determined that the issuance of these amendments will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of these amendments.

For further details with respect to this action, see (1) the application for amendments dated April 14, 1983, (2) Amendment Nos. 72 and 54 to License Nos. DPR-58 and DPR-74, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C. and at the Maude Reston Palenske Memorial Library, 500 Market Street, St. Joseph, Michigan 49085. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 25th day of April, 1983.

FOR THE NUCLEAR REGULATORY COMMISSION

Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing