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Docket No. 50-315

Mr. John Dolan, Vice President
Indiana and Michigan Electric Company
Post Office Box 18
Bowling Green Station
New York, New York 10004

Dear Mr. Dolan:

The Commission has issued the enclosed Amendment No. 56 to Facility Operating License No. DPR-58 for the Donald C. Cook Nuclear Plant, Unit No. 1. The amendment consists of changes to the Technical Specifications in response to your request transmitted by letter dated July 21, 1982.

This amendment redefines the Containment Purge and Exhaust Isolation System during the current refueling outage to allow modifications to upgrade the system.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

Original signed by:
S. A. Varga

Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Enclosures:

- 1. Amendment No. 56 to DPR-58
- 2. Safety Evaluation
- 3. Notice of Issuance

cc w/enclosures:
See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

INDIANA AND MICHIGAN ELECTRIC COMPANY

DOCKET NO. 50-315

DONALD C. COOK NUCLEAR PLANT UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 56
License No. DPR-58

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The request for amendment by Indiana and Michigan Electric Company (the licensee) dated July 21, 1982, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the request, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

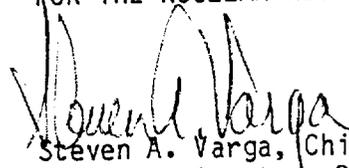
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-58 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 56, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: July 23, 1982

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 56 TO FACILITY OPERATING LICENSE NO. DPR-58

DOCKET NO. 50-315

Revise Appendix A as follows:

Remove Page

3/4 9-10

Insert Page

3/4 9-10

REFUELING OPERATIONS

COOLANT CIRCULATION

LIMITING CONDITION FOR OPERATION

3.9.8 At least one residual heat removal loop shall be in operation.

APPLICABILITY: MODE 6.

ACTION:

- a. With less than one residual heat removal loop in operation, except as provided in b. below, suspend all operations involving an increase in the reactor decay heat load or a reduction in boron concentration of the Reactor Coolant System. Close all containment penetrations providing direct access from the containment atmosphere to the outside atmosphere within 4 hours.
- b. The residual heat removal loop may be removed from operation for up to 1 hour per 8 hour period during the performance of CORE ALTERATIONS in the vicinity of the reactor pressure vessel hot legs.
- c. The provisions of Specification 3.0.3 are not applicable.

SURVEILLANCE REQUIREMENTS

4.9.8 A residual heat removal loop shall be determined to be in operation and circulating reactor coolant at a flow rate of ≥ 3000 gpm at least once per 24 hours.

REFUELING OPERATIONS

CONTAINMENT PURGE AND EXHAUST ISOLATION SYSTEM

LIMITING CONDITION FOR OPERATION

3.9.9 The Containment Purge and Exhaust isolation system*shall be OPERABLE.

APPLICABILITY: MODE 6.

ACTION:

With the Containment Purge and Exhaust isolation system*inoperable, close each of the Purge and Exhaust penetrations providing direct access from the containment atmosphere to the outside atmosphere. The provision of Specification 3.0.3 are not applicable.

SURVEILLANCE REQUIREMENTS

4.9.9 The Containment Purge and Exhaust isolation system shall be demonstrated OPERABLE within 100 hours prior to the start of and at least once per 7 days during CORE ALTERATIONS by verifying that containment Purge and Exhaust isolation occurs on manual initiation and on a high radiation signal from each of the containment radiation monitoring instrumentation channels.

*During the refueling outage commencing about July 1982, the Containment Purge and Exhaust Isolation System shall be defined to include (1) a radiation signal to either the inner or outer valve or a provision for immediate manual closure of the valve in the event the valve closure may be required, and (2) for the Lower Containment and Instrument Room penetrations, any one penetration out of service to replace a valve with the remaining valve locked or sealed closed or closed by a blind flange. This definition is temporary and it and this footnote shall no longer apply to following the refueling outage commencing about July 1982.

D. C. COOK - UNIT 1

3/4 9-10



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 56 TO FACILITY OPERATING LICENSE NO. DPR-58

INDIANA AND MICHIGAN ELECTRIC COMPANY

DONALD C. COOK NUCLEAR PLANT UNIT NO. 1

DOCKET NO. 50-315

Introduction

On July 21, 1982, in a letter telecopied to the NRC, the Indiana and Michigan Electric Company requested a temporary change to the D. C. Cook Facility Operating License Technical Specification 3.9.9 for Unit 1 regarding the Containment Purge and Exhaust System operable during Mode 6 (refueling). The change would allow the isolation valves to be replaced one at a time with valves qualified to satisfy the leak test requirements of Generic Task B-24, "Containment Purge and Exhaust," and would allow the containment radiation monitors to be upgraded to the requirements of TMI Item II.F.1, "Additional Accident Monitoring Instrumentation."

Discussion and Evaluation

The containment penetrations to be modified are in the lower containment and instrument room. Each of the four penetrations has two isolation valves; either 24 inches and 30 inches in diameter. There is currently one area radiation monitor supplying the signal to both the inner and outer valves for all purge and exhaust penetrations. This monitor will be replaced by two monitors, each to provide a signal to separate trains which supply a closure signal to either the inner valves or outer valves. One of the replacement monitors is already installed. For the Containment Purge and Exhaust System to be operable, both valves on each penetration must be operable and each is to be isolable from a signal from the containment radiation monitors. Technical Specification 3.9.9 requires the Containment Purge and Exhaust System to be operable during refueling.

In order to change the valves and radiation monitors, a temporary change is required to redefine the system for the short time necessary to cut out the old valve and weld in the new valve for the lower containment and instrument room penetration and to replace the containment monitors. As the area radiation monitor is replaced, the signal to both the inner and outer valve will not be available for a short period of time. A signal from the one newly installed monitor will be available to one of the valves in all the penetrations. The modifications will be accomplished within about a three or four week period with the actual down time for a penetration to be considerable less. At least one radiation monitor signal will be available to one of the valves on each purge and exhaust penetration at all times. During this refueling outage, as discussed with the staff, the licensee proposes to modify the Technical Specification as follows:

Add as footnote to page 3/4 9-10 of Unit 1 Technical Specifications.

"During the refueling outage commencing about July 1982, the Containment Purge and Exhaust Isolation System shall be defined to include (1) a radiation signal to either the inner or outer valve or a provision for immediate manual closure of the valve in the event the valve closure may be required, and (2) for the Lower Containment and Instrument Room penetrations, any one penetration out of service to replace a valve with the remaining valve locked or sealed closed or closed by a blind flange. This definition is temporary and it and this footnote shall no longer apply following the refueling outage commencing about July 1982."

This change discussed with the licensee will assure that any penetration being modified by valve replacement will be placed in a safe mode by either the remaining valve being locked or sealed closed so that it cannot open or a blind flange installed which would block the penetration completely until the flange was removed. In this manner, the penetration would not allow the transfer of air between the containment and the atmosphere. The current radiation monitor is set up to provide a signal to all the valves. As the current radiation monitor is changed, the signal will be available to only one of the valves on all the purge and exhaust penetrations for a short period of time. One monitor has already been installed and its signal will be applied to either the outer or inner valves. Other signals on both the inner and outer valves will not be disturbed; the radiation signal supplements those other signals. In the event the radiation signals are not available to either valve, the licensee will station an operator in the control room to manually close the valves if the need arises and to assure they remain closed.

During the refueling mode, the only accident that would significantly affect the containment atmosphere and require the closing of the Containment Purge and Exhaust System is the fuel handling accident where an element was dropped or an object was dropped on the elements in the open reactor vessel. In such an event, the irradiated fuel might release radioactive gasses to the containment.

The provisions to be undertaken by the licensee will assure that any penetration being modified will be unable to release any such radioactive gasses from the containment and that, should the gasses be detected, the existing radiation monitor signals will close the valves or alert the operators to manually close the valves on the other penetrations. The refueling personnel are in contact with the control room and any such accident will be known by the control room operators in sufficient time to close or verify closed all penetration valves. The licensee has determined that there will be adequate assurance of containment of any released radioactive gasses from a fuel handling accident during the modifications with the redefined operation of the Containment Purge and Exhaust System and we agree. We have further determined that the health and safety of the public will not be adversely affected by the modification or operation during the modifications and, therefore, the proposed temporary change to the D. C. Cook Unit No. 1 Technical Specification 3.9.9 is acceptable.

Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that this amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of an accident previously evaluated, does not create the possibility of an accident of a type different from any evaluated previously, and does not involve a significant reduction in a margin of safety, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulation and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: July 23, 1982

Principal Contributors:
D. Wigginton
G. Holahan

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-315INDIANA AND MICHIGAN ELECTRIC COMPANYNOTICE OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 56 to Facility Operating License No. DPR-58, to Indiana and Michigan Electric Company (the licensee), which revised Technical Specifications for operation of Donald C. Cook Nuclear Plant, Unit No. 1 (the facility) located in Berrien County, Michigan. The amendment is effective as of the date of issuance.

The amendment redefines the Containment Purge and Exhaust Isolation System during the current refueling outage to allow modifications to upgrade the system.

The request for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since this amendment does not involve a significant hazards consideration.

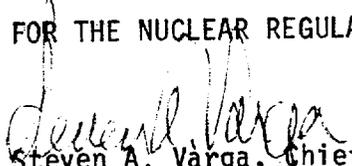
The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

- 2 -

For further details with respect to this action, see (1) the request for amendment dated July 21, 1982, (2) Amendment No. 56 to License No. DPR-58, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C. and at the Maude Reston Palenske Memorial Library, 500 Market Street, St. Joseph, Michigan 49085. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 23rd day of July, 1982.

FOR THE NUCLEAR REGULATORY COMMISSION


Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing