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SEP 09 1982

Docket No. 50-315

Mr. John Dolan, Vice President  
 Indiana and Michigan Electric Company  
 Post Office Box 18  
 Bowling Green Station  
 New York, New York 10004

Dear Mr. Dolan:

The Commission has issued the enclosed Amendment No. 58 to Facility Operating License No. DPR-58 for the Donald C. Cook Nuclear Plant, Unit No. 1. The amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated September 7, 1982.

This amendment permits operation in modes 2 and 3 with one Safety Injection pump inoperable for seven days beginning when the Unit enters mode 3 during startup from the 1982 refueling outage. In addition, the previous footnote to Specification 3.5.2 added under Amendment No. 55 has been deleted as it is no longer valid.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

**ORIGINAL SIGNED**  
 Ramon L. Cilimberg, Project Manager  
 Operating Reactors Branch #1  
 Division of Licensing

Enclosures:

1. Amendment No. 58 to DPR-58
2. Safety Evaluation
3. Notice of Issuance

cc w/enclosures:  
 See next page

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SURNAME	CParrish	RCilimberg	SVarsa	GLinas	R BLACK	GHOLAHAN	
DATE	09/9/82	09/9/82:ds	09/9/82	09/9/82	09/9/82	09/9/82	

Mr. John Dolan  
Indiana and Michigan Electric Company

cc: Mr. Robert W. Jurgensen  
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U. S. Nuclear Regulatory Commission  
Resident Inspectors Office  
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City of Bridgman, Michigan 49106

Regional Radiation Representative  
EPA Region V  
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The Honorable Tom Corcoran  
United States House of Representatives  
Washington, D. C. 20515

James G. Keppler  
Regional Administrator - Region III  
U. S. Nuclear Regulatory Commission  
799 Roosevelt Road  
Glen Ellyn, Illinois 60137



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

INDIANA AND MICHIGAN ELECTRIC COMPANY

DOCKET NO. 50-315

DONALD C. COOK NUCLEAR PLANT UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 58  
License No. DPR-58

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Indiana and Michigan Electric Company (the licensee) dated September 7, 1982, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;  
and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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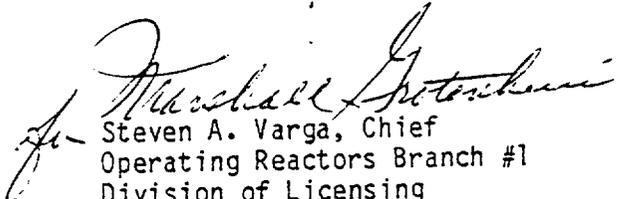
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-58 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 58, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

  
Steven A. Varga, Chief  
Operating Reactors Branch #1  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: September 9, 1982

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 58 TO FACILITY OPERATING LICENSE NO. DPR-58

DOCKET NO. 50-315

Revise Appendix A as follows:

Remove Page

3/4 5-3

Insert Page

3/4 5-3

EMERGENCY CORE COOLING SYSTEMS

ECCS SUBSYSTEMS -  $T_{avg} \geq 350^{\circ}\text{F}$

LIMITING CONDITION FOR OPERATION

3.5.2 Two independent ECCS subsystems shall be OPERABLE with each subsystem comprised of:

- a. One OPERABLE centrifugal charging pump,
- b. One OPERABLE safety injection pump,\*
- c. One OPERABLE residual heat removal heat exchanger,
- d. One OPERABLE residual heat removal pump, and
- e. An OPERABLE flow path capable of taking suction from the refueling water storage tank on a safety injection signal and transferring suction to the containment sump during the recirculation phase of operation.

APPLICABILITY: MODES 1, 2 and 3.

ACTION:

- a. With one ECCS subsystem inoperable, restore the inoperable subsystem to OPERABLE status within 72 hours or be in HOT SHUTDOWN within the next 12 hours.
- b. In the event the ECCS is actuated and injects water into the Reactor Coolant System, a Special Report shall be prepared and submitted to the Commission pursuant to Specification 6.9.2 within 90 days describing the circumstances of the actuation and the total accumulated actuation cycles to date.

\*Only one safety injection pump is required to be operable in Modes 2 and 3 for a period of time not to exceed 7 days beginning when the Unit enters Mode 3 during startup from the 1982 refueling outage. This note constitutes a one time exemption to the requirements of the Technical Specification.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 58 TO FACILITY OPERATING LICENSE NO. DPR-58

INDIANA AND MICHIGAN ELECTRIC COMPANY

DONALD C. COOK NUCLEAR PLANT UNIT NO. 1

DOCKET NO. 50-315

Introduction

By letter dated September 7, 1982, the Indiana and Michigan Electric Company (the licensee) proposed changes to the Technical Specifications appended to Facility Operating License No. DPR-58 for the Donald C. Cook Nuclear Plant Unit No. 1. The requested change would allow operation in modes 2 and 3 with one Safety Injection pump inoperable for seven days.

Discussion

The D. C. Cook Unit 1 ECCS includes low pressure pumps, Safety Injection (SI) pumps (~1500 psig shut off head) and high head pumps (centrifugal charging pumps). The plants FSAR small break LOCA analysis included credit for both the SI pumps and the high head ECCS pumps. For very small breaks (pressure remaining above 1500 psig) the SI pumps are not called upon. For larger, small break (1200 to 600 psig) both the SI pumps and the high head pumps are used; with approximately 650 gpm from the SI pumps and 550 gpm from the high head pumps. For large breaks the low head pumps and the accumulators provide the required ECCS flow.

In the event of a small LOCA, resulting in a pressure between 1200 and 600 psig, and a single failure of the other train of ECCS, there would be only 550 gpm available rather than the 1200 gpm (i.e., 650 gpm + 550 gpm) assumed in the FSAR analysis.

Evaluation

American Electric Power has provided information from Westinghouse sensitivity studies which indicate that reduction in ECCS flow would result in less than a 550°F increase in Peak Clad Temperature (PCT) assuming operation at full power. Since the FSAR small break LOCA analysis indicates a PCT of 1493°F, the 550°F increase would result in a PCT of 2043°F which is below the 10 CFR 50.46 limit of 2200°F.

There are three additional considerations favoring the approval of the requested change. These are: 1) American Electric Power has indicated that more recent Westinghouse ECCS evaluation models would demonstrate additional margin to the PCT criteria; 2) the request is for operation without the SI pump for only 7 days and 3) the analyses are for full power while operation will be limited to 5% power with very little decay heat, since the plant is presently returning to power from a refueling outage.

In addition, the previous footnote to Specification 3.5.2 added under Amendment No. 55 has been deleted as it is no longer valid. The footnote reflected a one-time relief from the requirements of the Action Statement of Specification 3.5.2. The temporary relief granted under Amendment No. 55 expired on June 10, 1982. Therefore, deletion of the previous footnote as part of this amendment is considered to be an administrative change.

#### Summary

Based on the information provided by the licensee, we conclude that operation in the proposed manner would not result in exceeding the Commission's regulations (10 CFR 50.46) in the event of a LOCA with a single failure. We, therefore, find the proposed change acceptable and conclude that it would not result in an undue risk to the health and safety of the public.

#### Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

#### Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of an accident previously evaluated, does not create the possibility of an accident of a type different from any evaluated previously, and does not involve a significant reduction in a margin of safety, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: September 9, 1982

Principal Contributors:

G. Holahan

R. Cilimberg

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-315INDIANA AND MICHIGAN ELECTRIC COMPANYNOTICE OF ISSUANCE OF AMENDMENT TO FACILITY  
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 58 to Facility Operating License No. DPR-58, issued to Indiana and Michigan Electric Company (the licensee), which revised Technical Specifications for operation of Donald C. Cook Nuclear Plant, Unit No. 1 (the facility) located in Berrien County, Michigan. The amendment is effective as of the date of issuance.

The amendment permits operation in Modes 2 and 3 with one Safety Injection pump inoperable for seven days beginning when the Unit enters Mode 3 during startup from the 1982 refueling outage. In addition, the previous footnote to Specification 3.5.2 added under Amendment No. 55 has been deleted as it is no longer valid.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

- 2 -

For further details with respect to this action, see (1) the application for amendment dated September 7, 1982, (2) Amendment No. 58 to License No. DPR-58, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C. and at the Maude Reston Palenske Memorial Library, 500 Market Street, St. Joseph, Michigan 49085. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 9th of September, 1982.

FOR THE NUCLEAR REGULATORY COMMISSION



Marshall Grotenhuis, Acting Chief  
Operating Reactors Branch #1  
Division of Licensing