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SEP 09 1982

Docket Nos. 50-315
and 50-316

Mr. John Dolan, Vice President
Indiana and Michigan Electric Company
Post Office Box 18
Bowling Green Station
New York, New York 10004

Dear Mr. Dolan:

The Commission has issued the enclosed Amendment No. 59 to Facility Operating License No. DPR-58 and Amendment No. 42 to Facility Operating License No. DPR-74 for the Donald C. Cook Nuclear Plant, Unit Nos. 1 and 2, respectively. The amendments consist of changes to the Technical Specifications in response to your application transmitted by letter dated July 13, 1982.

These amendments revise the Technical Specifications to reflect installation of replacement of valves to promote improved operation, and will not be effective until after the 1982 refueling outages for Units 1 and 2.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

ORIGINAL SIGNED

Ramon L. Cilimberg, Project Manager
Operating Reactors Branch #1
Division of Licensing

Enclosures:
1. Amendment No. 59 to DPR-58
2. Amendment No. 42 to DPR-74
3. Safety Evaluation
4. Notice of Issuance

cc w/enclosures:
See next page

OFFICE	ORB#1:DL	ORB#1:DL	ORB#1:DL	AD/OR:DL	OELD		
SURNAME	CParrish	RCilimberg	SVarga	GLainas	RBlack		
DATE	09/9/82	09/8/82:ds	09/9/82	09/9/82	09/9/82		

Mr. John Dolan
Indiana and Michigan Electric Company

cc: Mr. Robert W. Jurgensen
Chief Nuclear Engineer
American Electric Power
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Donald C. Cook Nuclear Plant
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U. S. Nuclear Regulatory Commission
Resident Inspectors Office
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Stevensville, Michigan 49127

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Lake Township
Baroda, Michigan 49101

Mr. William R. Rustem (2)
Office of the Governor
Room 1 - Capitol Building
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Honorable James Bemenek, Mayor
City of Bridgman, Michigan 49106

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The Honorable Tom Corcoran
United States House of Representatives
Washington, D. C. 20515

James G. Keppler
Regional Administrator - Region III
U. S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, Illinois 60137

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

INDIANA AND MICHIGAN ELECTRIC COMPANY

DOCKET NO. 50-315

DONALD C. COOK NUCLEAR PLANT UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 59
License No. DPR-58

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Indiana and Michigan Electric Company (the licensee) dated July 13, 1982, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-58 is hereby amended to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 59 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Marshall Grotenhuis for

Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: September 9, 1982

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 59 TO FACILITY OPERATING LICENSE NO. DPR-58

DOCKET NO. 50-315

Revise Appendix A as follows:

Remove Pages	Insert Pages
3/4 6-17	3/4 6-17
3/4 6-18	3/4 6-18
3/4 6-19	3/4 6-19
3/4 6-20	3/4 6-20
3/4 6-21	3/4 6-21
3/4 6-22	3/4 6-22

TABLE 3.6-1 (Continued)

VALVE NUMBER	FUNCTION	TESTABLE DURING PLANT OPERATION	ISOLATION TIME IN SECONDS
A. PHASE "A" ISOLATION (Continued)			
27. ECR-10	Cont. H_2 Sample Return	Yes	10
28. ECR-11	Cont. H_2 Sample - Air to Rec. E	Yes	10
29. ECR-12	Cont. H_2 Sample - Air from Rec. E	Yes	10
30. ECR-13	Cont. H_2 Sample - Low. Cont. Vol.	Yes	10
31. ECR-14	Cont. H_2 Sample - Low. Cont. Vol.	Yes	10
32. ECR-15	Cont. H_2 Sample - Up Cont. Vol.	Yes	10
33. ECR-16	Cont. H_2 Sample - Up Cont. Vol.	Yes	10
34. ECR-17	Cont. H_2 Sample - Air to Rec. W	Yes	10
35. ECR-18	Cont. H_2 Sample - Air from Rec. W	Yes	10
36. ECR-19	Cont. H_2 Sample - Cont. Dome Vol.	Yes	10
37. ECR-20	Cont. H_2 Sample - Return	Yes	10
38. ECR-21	Cont. H_2 Sample - Air to Rec. E.	Yes	10
39. ECR-22	Cont. H_2 Sample - Air fr. Rec. E	Yes	10
40. ECR-23	Cont. H_2 Sample - Low Cont. Vol.	Yes	10
41. ECR-24	Cont. H_2 Sample - Low Cont. Vol.	Yes	10
42. ECR-25	Cont. H_2 Sample - Up Cont. Vol.	Yes	10
43. ECR-26	Cont. H_2 Sample - Up Cont. Vol.	Yes	10
44. ECR-27	Cont. H_2 Sample - Air to Rec. W.	Yes	10
45. ECR-28	Cont. H_2 Sample - Air Fr. Rec. W.	Yes	10
46. ECR-29	Cont. H_2 Sample - Cont. Dome Vol.	Yes	10
47. ECR-416	PAS Containment Sump Sample	Yes	10
48. ECR-417	PAS Containment Sump Sample	Yes	10
49. ECR-496	PAS Waste Liquid and Gas Return	Yes	10
50. ECR-497	PAS Waste Liquid and Gas Return	Yes	10
51. ECR-535	PAS Containment Gas Sample	Yes	10
52. ECR-536	PAS Containment Gas Sample	Yes	10
53. GCR-301	N_2 Supply to Pressurizer Relief Tank	Yes	10
54. GCR-314	N_2 Supply to Accumulators	Yes	10
55. ICR-5	Accumulators Sample	Yes	10
56. ICR-6	Accumulators Sample	Yes	10
57. MCR-251	Sample Line from Steam Gen. Outlet #1	Yes	10
58. MCR-252	Sample Line from Steam Gen. Outlet #2	Yes	10
59. MCR-253	Sample Line from Steam Gen. Outlet #3	Yes	10
60. MCR-254	Sample Line form Steam Gen. Outlet #4	Yes	10
61. NCR-105	Hot Leg Sample	Yes	10
62. NCR-106	Hot Leg Sample	Yes	10

This Technical Specification will not be effective until after the 1982 refueling outage.

TABLE 3.6-1 (continued)

VALVE NUMBER	FUNCTION	TESTABLE DURING PLANT OPERATION	ISOLATION TIME IN SECONDS
A. PHASE "A" ISOLATION (Continued)			
63. NCR-107	PRZ Liquid Sample	Yes	10
64. NCR-108	PRZ Liquid Sample	Yes	10
65. NCR-109	PRZ Steam Sample	Yes	10
66. NCR-110	PRZ Steam Sample	Yes	10
67. NCR-252	Primary Water to Pressure Relief Tank	Yes	10
68. QCM-250	RCP Seal Water Discharge	No	15
69. QCM-350	RCP Seal Water Discharge	No	15
70. QCR-300	Letdown to Letdown Hx.	No	10
71. QCR-301	Letdown to Letdown Hx.	No	10
72. QCR-919	Demineralized Water Supply for Refueling Cavity	Yes	10
73. QCR-920	Demineralized Water Supply for Refueling Cavity	Yes	10
74. RCR-100	PRZ Relief Tank to Gas Anal.	Yes	10
75. RCR-101	PRZ Relief Tank to Gas Anal.	Yes	10
76. VCR-10	Glycol Supply to Fan Cooler	Yes	10
77. VCR-11	Glycol Supply to Fan Cooler	Yes	10
78. VCR-20	Glycol Supply from Fan Cooler	Yes	10
79. VCR-21	Glycol Supply from Fan Cooler	Yes	10
80. XCR-100	Control Air to Containment	No	10
81. XCR-101	Control Air to Containment Isolation	No	10
82. XCR-102	Control Air to Containment Isolation	No	10
83. XCR-103	Control Air to Containment	No	10
B. PHASE "B" ISOLATION			
1. CCM-451	CCW from RCP Oil Coolers	No	60
2. CCM-452	CCW from RCP Oil Coolers	No	60
3. CCM-453	CCW from RCP Thermal Barrier	No	30
4. CCM-454	CCW from RCP Thermal Barrier	No	30
5. CCM-458	CCW to RCP Oil Coolers & Thermal Barrier	No	60
6. CCM-459	CCW to RCP Oil Coolers & Thermal Barrier	No	60
7. ECR-31	Containment Airborne Radiation Monitor	No	10
8. ECR-32	Containment Airborne Radiation Monitor	No	10
9. ECR-33	Containment Airborne Radiation Monitor	No	10
10. ECR-35	Containment Airborne Radiation Monitor	No	10
11. ECR-36	Containment Airborne Radiation Monitor	No	10

This Technical Specification will not be effective until after the 1982 refueling outages.

TABLE 3.6-1 (Continued)

VALVE NUMBER	FUNCTION	TESTABLE DURING PLANT OPERATION	ISOLATION TIME IN SECONDS
B. PHASE "B" ISOLATION (Continued)			
12. WCR-901	NESW to Low Containment Vent #1	Yes	10
13. WCR-903	NESW from Low Containment Vent #1	Yes	10
14. WCR-905	NESW to Low Containment Vent #2	Yes	10
15. WCR-907	NESW from Low Containment Vent #2	Yes	10
16. WCR-909	NESW to Low Containment Vent #3	Yes	10
17. WCR-911	NESW from Low Containment Vent #3	Yes	10
18. WCR-913	NESW to Low Containment Vent #4	Yes	10
19. WCR-915	NESW from Low Containment Vent #4	Yes	10
20. WCR-921	NESW to Up Containment Vent #1	Yes	10
21. WCR-923	NESW from Up Containment Vent #1	Yes	10
22. WCR-925	NESW to Up Containment Vent #2	Yes	10
23. WCR-927	NESW from Up Containment Vent #2	Yes	10
24. WCR-929	NESW to Up Containment Vent #3	Yes	10
25. WCR-931	NESW from Up Containment Vent #3	Yes	10
26. WCR-933	NESW to Up Containment Vent #4	Yes	10
27. WCR-935	NESW from Up Containment Vent #4	Yes	10
28. WCR-941	NESW to RCP Motor Air Cooler	Yes	10
29. WCR-942	NESW to RCP Motor Air Cooler	Yes	10
30. WCR-943	NESW to RCP Motor Air Cooler	Yes	10
31. WCR-944	NESW to RCP Motor Air Cooler	Yes	10
32. WCR-945	NESW from RCP Motor Air Cooler	Yes	10
33. WCR-946	NESW from RCP Motor Air Cooler	Yes	10
34. WCR-947	NESW from RCP Motor Air Cooler	yes	10
35. WCR-948	NESW from RCP Motor Air Cooler	Yes	10
36. WCR-951	NESW to RCP Motor Air Cooler Vent #1	Yes	10
37. WCR-952	NESW to RCP Motor Air Cooler Vent #2	Yes	10
38. WCR-953	NESW to RCP Motor Air Cooler Vent #3	Yes	10
39. WCR-954	NESW to RCP Motor Air Cooler Vent #4	Yes	10
40. WCR-955	NESW from RCP Motor Air Cooler Vent #1	Yes	10
41. WCR-956	NESW from RCP Motor Air Cooler Vent #2	Yes	10
42. WCR-957	NESW from RCP Motor Air Cooler Vent #3	Yes	10
43. WCR-958	NESW from RCP Motor Air Cooler Vent #4	Yes	10

This Technical Specification will not be effective until after the 1982 refueling outages.

TABLE 3.6-1 (Continued)

VALVE NUMBER	FUNCTION	TESTABLE DURING PLANT OPERATION	ISOLATION TIME IN SECONDS
B. PHASE "B" ISOLATION (Continued)			
44. WCR-961	NESW to Instr. Rm. East Vent	Yes	10
45. WCR-963	NESW from Instr. Rm. West Vent	Yes	10
46. WCR-965	NESW to Instr. Rm. East Vent	Yes	10
47. WCR-967	NESW from Instr. Rm. West Vent	Yes	10
48. WCR-900	NESW to RCP Lower Containment Vent #1	Yes	10
49. WCR-902	NESW from Lower Containment Vent #1	Yes	10
50. WCR-904	NESW to RCP Lower Containment Vent #2	Yes	10
51. WCR-906	NESW from Lower Containment Vent #2	Yes	10
52. WCR-908	NESW to RCP Lower Containment Vent #3	Yes	10
53. WCR-910	NESW from Lower Containment Vent #3	Yes	10
54. WCR-912	NESW to RCP Lower Containment Vent #4	Yes	10
55. WCR-914	NESW from Lower Containment Vent #4	Yes	10
56. WCR-920	NESW to RCP Upper Containment Vent #1	Yes	10
57. WCR-922	NESW from Upper Containment Vent #1	Yes	10
58. WCR-924	NESW to RCP Upper Containment Vent #2	Yes	10
59. WCR-926	NESW from Upper Containment Vent #2	Yes	10
60. WCR-928	NESW to RCP Upper Containment Vent #3	Yes	10
61. WCR-930	NESW from Upper Containment Vent #3	Yes	10
62. WCR-932	NESW to RCP Upper Containment Vent #4	Yes	10
63. WCR-934	NESW from Upper Containment Vent #4	Yes	10
64. WCR-960	NESW to Instrument Room East Vent	Yes	10
65. WCR-962	NESW from Instrument Room East Vent	Yes	10
66. WCR-964	NESW to Instrument Room West Vent	Yes	10
67. WCR-966	NESW from Instrument Room West Vent	Yes	10
C. CONTAINMENT PURGE AND EXHAUST			
1. VCR-101	Instr. Room Purge Air Inlet	Yes	5
2. VCR-102	Instr. Room Purge Air Outlet	Yes	5
3. VCR-103	Lower Comp. Purge Air Inlet	Yes	5
4. VCR-104	Lower Comp. Purge Air Outlet	Yes	5
5. VCR-105	Upper Comp. Purge Air Inlet	Yes	5
6. VCR-106	Upper Comp. Purge Air Outlet	Yes	5
7. VCR-107*	Cont. Press. Relief Fan Isolation	Yes	5
8. VCR-201	Instr. Room Purge Air Inlet	Yes	5
9. VCR-202	Instr. Room Purge Air Outlet	Yes	5
10. VCR-203	Lower Comp. Purge Air Inlet	Yes	5
11. VCR-204	Lower Comp. Purge Air Outlet	Yes	5

This Technical Specification will not be effective until after the 1982 refueling outage.

TABLE 3.6-1 (Continued)

VALVE NUMBER	FUNCTION	TESTABLE DURING PLANT OPERATION	ISOLATION TIME IN SECONDS
C. CONTAINMENT PURGE EXHAUST (Continued)			
12. VCR-205	Upper Comp. Purge Air Inlet	Yes	5
13. VCR-206	Upper Comp. Purge Air Outlet	Yes	5
14. VCR-207*	Cont. Press Relief Fan Isolation	Yes	5
D. MANUAL ISOLATION VALVES(1)			
1. ICM-111	RHR to RC Cold Legs	Yes	NA
2. ICM-129	RHR Inlet to Pumps	No	NA
3. ICM-250	Boron Injection Inlet	Yes	NA
4. ICM-251	Boron Injection Inlet	Yes	NA
5. ICM-260	Safety Injection Inlet	Yes	NA
6. ICM-265	Safety Injection Inlet	Yes	NA
7. ICM-305	RHR Suction from Sump	Yes	NA
8. ICM-306	RHR Suction from Sump	Yes	NA
9. ICM-311	RHR to RC Hot Legs	Yes	NA
10. ICM-321	RHR to RC Hot Legs	Yes	NA
11. NPX 151 VI	Dead Weight Tester	Yes	NA
12. PA 145	Containment Service Air	No	NA
13. SF-151	Refueling Water Supply	Yes	NA
14. SF-153	Refueling Water Supply	Yes	NA
15. SF-159	Refueling Cavity Drain to Purification System	Yes	NA
16. SF-160	Refueling Cavity Drain to Purification System	Yes	NA
17. SI-171	Safety Injection Test Line	Yes	NA
18. SI-172	Accumulator Test Line	Yes	NA

This Technical Specification will not be effective until after the 1982 refueling outage.

TABLE 3.6-1 (Continued)

VALVE NUMBER	FUNCTION	TESTABLE DURING PLANT OPERATION	ISOLATION TIME IN SECONDS
D. MANUAL ISOLATION VALVES (1) (Continued)			
19. CCR-440	CCW from Main Steam Penetration	Yes	NA
20. CCR-441	CCW from Main Steam Penetration	Yes	NA
21. MCM-221	Main Steam to Auxiliary Feed Pump	No	NA
22. MCM-231	Main Steam to Auxiliary Feed Pump	No	NA
23. CCM-430	CCW to East Pressure Equalization Fan	Yes	NA
24. CCM-431	CCW from East Pressure Equalization Fan	Yes	NA
25. CCM-432	CCW to West Pressure Equalization Fan	Yes	NA
26. CCM-433	CCW from West Pressure Equalization Fan	Yes	NA
27. SM-8*	Upper Containment Sample	Yes	NA
28. SM-10*	Upper Containment Sample	Yes	NA
29. SM-4*	Instrument Room Sample	Yes	NA
30. SM-6*	Instrument Room Sample	Yes	NA

NA - Manual Valve-Isolation time not applicable.
(1) - Includes motor operated valves which do not isolate automatically.
* - May be opened on an intermittent basis under administrative control.

This Technical Specification will not be effective until after the 1982 refueling outage.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

INDIANA AND MICHIGAN ELECTRIC COMPANY

DOCKET NO. 50-316

DONALD C. COOK NUCLEAR PLANT UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 42
License No. DPR-74

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Indiana and Michigan Electric Company (the licensee) dated July 13, 1982, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-74 is hereby amended to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 42 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Marshall Grotenhuis

for Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: September 9, 1982

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 42 TO FACILITY OPERATING LICENSE NO. DPR-74

DOCKET NO. 50-316

Revise Appendix A as follows:

Remove Pages	Insert Pages
3/4 6-17*	3/4 6-17*
3/4 6-18	3/4 6-18
3/4 6-19	3/4 6-19
3/4 6-20	3/4 6-20
3/4 6-21	3/4 6-21
3/4 6-22	3/4 6-22
3/4 6-23	3/4 6-23
3/4 6-24	3/4 6-24
3/4 6-25	3/4 6-25
3/4 6-26*	3/4 6-26*
3/4 6-29	3/4 6-29
3/4 6-30	3/4 6-30
3/4 6-31	3/4 6-31
3/4 6-32*	3/4 6-32*

*Included for convenience

TABLE 3.6-1 (Continued)

CONTAINMENT ISOLATION VALVES

VALVE NUMBER	FUNCTION	ISOLATION TIME IN SECONDS
A. PHASE "A" ISOLATION (Continued)		
25. DCR-620	Continuous Ventilation Drain to Holdup	$\leq$ 10
26. DCR-621	Continuous Ventilation Drain to Holdup	$\leq$ 10
27. ECR-10	Cont. H_2 Sample Return	$\leq$ 10
28. ECR-11	Cont. H_2 Sample - Air to Rec. E	$\leq$ 10
29. ECR-12	Cont. H_2 Sample - Air From Rec. E	$\leq$ 10
30. ECR-13	Cont. H_2 Sample - Low. Cont. Vol.	$\leq$ 10
31. ECR-14	Cont. H_2 Sample - Low. Cont. Vol.	$\leq$ 10
32. ECR-15	Cont. H_2 Sample - Up. Cont. Vol.	$\leq$ 10
33. ECR-16	Cont. H_2 Sample - Up. Cont. Vol.	$\leq$ 10
34. ECR-17	Cont. H_2 Sample - Air to Rec. W	$\leq$ 10
35. ECR-18	Cont. H_2 Sample - Air from Rec. W	$\leq$ 10
36. ECR-19	Cont. H_2 Sample - Cont. Dome Vol.	$\leq$ 10

This Technical Specification will not be effective until after the 1982 refueling outage.

TABLE 3.6-1 (Continued)

CONTAINMENT ISOLATION VALVES

VALVE NUMBER		FUNCTION	ISOLATION TIME IN SECONDS
A. PHASE "A" ISOLATION (Continued)			
37.	ECR-20	Cont. H_2 Sample-Return	≤10
38.	ECR-21	Cont. H_2 Sample - Air to Rec. E.	≤10
39.	ECR-22	Cont. H_2 Sample - Air Fr. Rec. E.	≤10
40.	ECR-23	Cont. H_2 Sample - Low. Cont. Vol.	≤10
41.	ECR-24	Cont. H_2 Sample - Low. Cont. Vol.	≤10
42.	ECR-25	Cont. H_2 Sample - Up. Cont. Vol.	≤10
43.	ECR-26	Cont. H_2 Sample - Up. Cont. Vol.	≤10
44.	ECR-27	Cont. H_2 Sample - Air to Rec. W.	≤10
45.	ECR-28	Cont. H_2 Sample - Air Fr. Rec. W.	≤10
46.	ECR-29	Cont. H_2 Sample - Cont. Dome Vol.	≤10
47.	ECR-416	PAS Containment Sump Sample	≤10
48.	ECR-417	PAS Containment Sump Sample	≤10
49.	ECR-496	PAS Waste Liquid and Gas Return	≤10
50.	ECR-497	PAS Waste Liquid and Gas Return	≤10
51.	ECR-535	PAS Containment Gas Sample	≤10
52.	ECR-536	PAS Containment Gas Sample	≤10
53.	GCR-301	N_2 Supply to Pressurizer Relief Tank	≤10
54.	GCR-314	N_2 Supply to Accumulators	≤10

This Technical Specification will not be effective until after the 1982 refueling outage.

TABLE 3.6-1 (Continued)

CONTAINMENT ISOLATION VALVES

VALVE NUMBER	FUNCTION	ISOLATION TIME IN SECONDS
A. PHASE "A" ISOLATION (Continued)		
55. 1CR-5	Accumulators Sample	≤ 10
56. 1CR-6	Accumulators Sample	≤ 10
57. MCR-251#	Sample Line from Steam Gen. Outlet #1	≤ 10
58. MCR-252#	Sample Line from Steam Gen. Outlet #1	≤ 10
59. MCR-253#	Sample Line from Steam Gen. Outlet #3	≤ 10
60. MCR-254#	Sample Line from Steam Gen. Outlet #4	≤ 10
61. NCR-105	Hot Leg Sample	≤ 10
62. NCR-106	Hot Leg Sample	≤ 10
63. NCR-107	PRZ Liquid Sample	≤ 10
64. NCR-108	PRZ Liquid Sample	≤ 10
65. NCR-109	PRZ Steam Sample	≤ 10
66. NCR-110	PRZ Steam Sample	≤ 10

D. C. COOK - UNIT 2 3/4 6-19 Amendment No. 42

This Technical Specification will not be effective until after the 1982 refueling outage.

TABLE 3.6-1 (Continued)

CONTAINMENT ISOLATION VALVES

VALVE NUMBER	FUNCTION	ISOLATION TIME IN SECONDS
A. PHASE "A" ISOLATION (Continued)		
67. NCR-252	Primary Water to Pressurizer Relief Tank	≤10
68. QCM-250	RCP Seal Water Discharge	≤15
69. QCM-350	RCP Seal Water Discharge	≤15
70. QCR-300	Letdown to Letdown Hx.	≤10
71. QCR-301	Letdown to Letdown Hx.	≤10
72. QCR-919	Demin Wtr. Supply for Refueling Cavity	≤10
73. QCR-920	Demin Wtr. Supply for Refueling Cavity	≤10
74. RCR-100	PRZ Relief Tank to Gas Anal.	≤10
75. RCR-101	PRZ Relief Tank to Gas Anal.	≤10
76. VCR-10	Glycol Supply to Fan Cooler	≤10
77. VCR-11	Glycol Supply to Fan Cooler	≤10
78. VCR-20	Glycol Supply from Fan Cooler	≤10
79. VCR-21	Glycol Supply from Fan Cooler	≤10
80. XCR-100	Control Air to Containment	≤10
81. XCR-101	Control Air to Containment Isolation	≤10

This Technical Specification will not be effective until after the 1982 refueling outage.

TABLE 3.6-1 (Continued)

CONTAINMENT ISOLATION VALVES

VALVE NUMBER	FUNCTION	ISOLATION TIME IN SECONDS
A. PHASE "A" ISOLATION (Continued)		
82. XCR-102	Control Air to Containment Isolation	≤ 10
83. XCR-103	Control Air to Containment	≤ 10
B. PHASE "B" ISOLATION		
1. CCM-451	CCW from RCP Oil Coolers	≤ 60
2. CCM-452	CCW from RCP Oil Coolers	≤ 60
3. CCM-453	CCW from RCP Thermal Barrier	≤ 30
4. CCM-454	CCW from RCP Thermal Barrier	≤ 30
5. CCM-458	CCW to RCP Oil Coolers & Thermal Barrier	≤ 60
6. CCM-459	CCW to RCP Oil Coolers & Thermal Barrier	≤ 60
7. ECR-31	Containment Airborne Rad Monitor	≤ 10
8. ECR-32	Containment Airborne Rad Monitor	≤ 10
9. ECR-33	Containment Airborne Rad Monitor	≤ 10
10. ECR-35	Containment Airborne Rad Monitor	≤ 10
11. ECR-36	Containment Airborne Rad Monitor	≤ 10

This Technical Specification will not be effective until after the 1982 refueling outage.

TABLE 3.6-1 (Continued)

CONTAINMENT ISOLATION VALVES

VALVE NUMBER	FUNCTION	ISOLATION TIME IN SECONDS
B. PHASE "B" ISOLATION (Continued)		
12. WCR-901	NESW to Low. Containment Vent #1	≤10
13. WCR-903	NESW from Low. Containment Vent #1	≤10
14. WCR-905	NESW to Low. Containment Vent #2	≤10
15. WCR-907	NESW from Low. Containment Vent #2	≤10
16. WCR-909	NESW to Low. Containment Vent #3	≤10
17. WCR-911	NESW from Low. Containment Vent #3	≤10
18. WCR-913	NESW to Low. Containment Vent #4	≤10
19. WCR-915	NESW from Low Containment Vent #4	≤10
20. WCR-921	NESW to Up. Containment Vent #1	≤10
21. WCR-923	NESW from Up. Containment Vent #1	≤10
22. WCR-925	NESW to Up. to Containment Vent #2	≤10
23. WCR-927	NESW from Up. Containment Vent #2	≤10

This Technical Specification will not be effective until after the 1982 refueling outage.

TABLE 3.6-1 (Continued)

CONTAINMENT ISOLATION VALVES

VALVE NUMBER	FUNCTION	ISOLATION TIME IN SECONDS
B. PHASE "B" ISOLATION (Continued)		
24. WCR-929	NESW to Up. Containment Vent #3	≤10
25. WCR-931	NESW from Up. Containment Vent #3	≤10
26. WCR-933	NESW to Up. Containment Vent #4	≤10
27. WCR-935	NESW from Up. Containment Vent #4	≤10
28. WCR-941	NESW to RCP Motor Air Cooler	≤10
29. WCR-942	NESW to RCP Motor Air Cooler	≤10
30. WCR-943	NESW to RCP Motor Air Cooler	≤10
31. WCR-944	NESW to RCP Motor Air Cooler	≤10
32. WCR-945	NESW from RCP Motor Air Cooler	≤10
33. WCR-946	NESW from RCP Motor Air Cooler	≤10
34. WCR-947	NESW from RCP Motor Air Cooler	≤10
35. WCR-948	NESW from RCP Motor Air Cooler	≤10
36. WCR-951	NESW to RCP Motor Air Cooler Vent #1	≤10
37. WCR-952	NESW to RCP Motor Air Cooler Vent #2	≤10
38. WCR-953	NESW to RCP Motor Air Cooler Vent #3	≤10
39. WCR-954	NESW to RCP Motor Air Cooler Vent #4	≤10

This Technical Specification will not be effective until after the 1982 refueling outage.

TABLE 3.6-1 (Continued)

CONTAINMENT ISOLATION VALVES

VALVE NUMBER	FUNCTION	ISOLATION TIME IN SECONDS
B. PHASE "B" ISOLATION (Continued)		
40. WCR-955	NESW from RCP Motor Air Cooler Vent #1	≤10
41. WCR-956	NESW from RCP Motor Air Cooler Vent #2	≤10
42. WCR-957	NESW from RCP Motor Air Cooler Vent #3	≤10
43. WCR-958	NESW from RCP Motor Air Cooler Vent #4	≤10
44. WCR-961	NESW to Instr. Rm. East Vent	≤10
45. WCR-963	NESW from Instr. Rm. West Vent	≤10
46. WCR-965	NESW to Instr. Rm. East Vent	≤10
47. WCR-967	NESW from Instr. Rm. West Vent	≤10
48. WCR-900	NESW to RCP Lower Containment Vent #1	≤10
49. WCR-902	NESW from Lower Containment Vent #1	≤10
50. WCR-904	NESW to RCP Lower Containment Vent #2	≤10
51. WCR-906	NESW from Lower Containment Vent #2	≤10
52. WCR-908	NESW to RCP Lower Containment Vent #3	≤10
53. WCR-910	NESW from Lower Containment Vent #3	≤10
54. WCR-912	NESW to RCP Lower Containment Vent #4	≤10
55. WCR-914	NESW from Lower Containment Vent #4	≤10

This Technical Specification will not be effective until after the 1982 refueling outage.

TABLE 3.6-1 (Continued)

CONTAINMENT ISOLATION VALVES

VALVE NUMBER	FUNCTION	ISOLATION TIME IN SECOND
B. PHASE "B" ISOLATION (Continued)		
56. WCR-920	NESW to RCP Upper Containment Vent #1	≤10
57. WCR-922	NESW from Upper Containment Vent #1	≤10
58. WCR-924	NESW to RCP Upper Containment Vent #2	≤10
59. WCR-926	NESW from Upper Containment Vent #2	≤10
60. WCR-928	NESW to RCP Upper Containment Vent #3	≤10
61. WCR-930	NESW from Upper Containment Vent #3	≤10
62. WCR-932	NESW to RCP Upper Containment Vent #4	≤10
63. WCR-934	NESW from Upper Containment Vent #4	≤10
64. WCR-960	NESW to Instrument Room East Vent	≤10
65. WCR-962	NESW from Instrument Room East Vent	≤10
66. WCR-964	NESW to Instrument Room West Vent	≤10
67. WCR-966	NESW from Instrument Room West Vent	≤10
C. CONTAINMENT PURGE AND EXHAUST		
1. VCR-101	Instr. Room Purge Air Inlet	≤5
2. VCR-102	Instr. Room Purge Air Outlet	≤5
3. VCR-103	Lower Comp. Purge Air Inlet	≤5
4. VCR-104	Lower Comp. Purge Air Outlet	≤5
5. VCR-105	Upper Comp. Purge Air Inlet	≤5

This Technical Specification will not be effective until after the 1982 refueling outage.

TABLE 3.6-1 (Continued)

CONTAINMENT ISOLATION VALVES

VALVE NUMBER		FUNCTION	ISOLATION TIME IN SECONDS
C.	CONTAINMENT PURGE AND EXHAUST (Continued)		
6.	VCR-106	Upper Comp. Purge Air Outlet	≤ 5
7.	VCR-107*	Cont. Press. Relief Fan Isolation	≤ 5
8.	VCR-201	Instr. Room Purge Air Inlet	≤ 5
9.	VCR-202	Instr. Room Purge Air Outlet	≤ 5
10.	VCR-203	Lower Comp. Purge Air Inlet	≤ 5
11.	VCR-204	Lower Comp. Purge Air Outlet	≤ 5
12.	VCR-205	Upper Comp. Purge Air Outlet	≤ 5
13.	VCR-206	Upper Comp. Purge Air Outlet	≤ 5
14.	VCR-207*	Cont. Press Relief Fan Isolation	≤ 5
D.	MANUAL ISOLATION VALVES (1)		
1.	1CM-111#	RHR to RC Cold Legs	NA
2.	1CM-129	RHR Inlet to Pumps	NA

This Technical Specification will not be effective until after the 1982 refueling outage.

TABLE 3.6-1 (Continued)

CONTAINMENT ISOLATION VALVES

VALVE NUMBER	FUNCTION	ISOLATION TIME IN SECONDS
E. OTHER (Continued)		
18. PA-243	Service Air to Containment	NA
19. NPX-151 VI	Dead Weight Calibrator	NA
20. N-160	N_2 to R. C. Drain Tank	NA
21. SM-1	Air Particle/Radio Gas Detect Return	NA
22. N-102	N_2 To Accumulators	NA
23. SI-171	Safety Injection Test Line	NA
24. SI-172	Safety Injection Test Line	NA
25. SI-194	Safety Injection Test Line	NA
26. PW-275	Primary Wtr. to Pre. Relief Tank	NA
27. CS-321	R.C.S. Charging	NA

This Technical Specification will not be effective until after the 1982 refueling outage.

TABLE 3.6-1 (Continued)

CONTAINMENT ISOLATION VALVES

VALVE NUMBER	FUNCTION	ISOLATION TIME IN SECONDS
E. OTHER (Continued)		
28. SF-152	Refueling Wtr. to Refuel. Cavity	NA
29. SF-154	Refueling Wtr. to Refuel. Cavity	NA
30. SF-159	Refueling Cavity Drain	NA
31. SF-160	Refueling Cavity Drain	NA
32. NSW-417-3	NESW to Instr. Rm. Vent 'W'	NA
33. NSW-417-4	NESW to Instr. Rm. Vent 'E'	NA
34. N-159	N_2 to Prz. Relief Tank	NA
35. CCW-135	CCW to Reactor Supports	NA
36. CA-181-N	Weld Channel Supply Air	NA
37. CA-181-S	Weld Channel Supply Air	NA
38. SM-8	Upper Cont. Grab Sample	NA
39. SM-10	Upper Cont. Grab Sample	NA

This Technical Specification will not be effective until after the 1982 refueling outage.

TABLE 3.6-1 (Continued)

CONTAINMENT ISOLATION VALVES

VALVE NUMBER	FUNCTION	ISOLATION TIME IN SECONDS
E. OTHER (Continued)		
40. PPP-300	Instrument Penetration	NA
41. PPP-301	Instrument Penetration	NA
42. PPP-302	Instrument Penetration	NA
43. PPP-303	Instrument Penetration	NA
44. PPA-310 and PPA-311	Instrument Penetration	NA
45. PPA-312 and PPA-313	Instrument Penetration	NA
46. Blind Flange	Fuel Transfer Penetration	NA
47. Blind Flange	Service Air to Containment	NA
48. Blind Flange	Ice Condenser Ice Supply	NA
49. Blind Flange	Ice Condenser Ice Return	NA
50. Blind Flange	In-Core Flux Thimble Access	NA

This Technical Specification will not be effective until after the 1982 refueling outage.

TABLE 3.6-1 (Continued)

CONTAINMENT ISOLATION VALVES

TABLE NOTATION

* May be opened on an intermittent basis under administrative control.

\# Not subject to Type "B" or "C" Leak Tests.

NA Check valves, blind flanges on normally closed valves which do not receive containment isolation signals; isolation time not applicable.

(1) Includes motor operated valves which do not isolate automatically.

This Technical Specification will not be effective until after the 1982 refueling outage.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 59 TO FACILITY OPERATING LICENSE NO. DPR-58

AND AMENDMENT NO. 42 TO FACILITY OPERATING LICENSE NO. DPR-74

INDIANA AND MICHIGAN ELECTRIC COMPANY

DONALD C. COOK PLANT UNIT NOS. 1 AND 2

DOCKET NOS. 50-315 AND 50-316

Introduction

By letter dated July 13, 1982, the Indiana and Michigan Electric Company (the licensee) proposed changes to the Technical Specifications appended to Facility Operating License Nos. DPR-58 and DPR-74 for the Donald C. Cook Nuclear Power Plant Unit Nos. 1 and 2. The proposed changes reflect the addition or replacement of containment isolation valves to improve operations, and will not be effective until after the 1982 refueling outages for Units 1 and 2. The acceptability of these changes from a safety standpoint are discussed below.

Discussion and Evaluation

Indiana and Michigan Electric Company has proposed revision of Technical Specifications to change the list of containment isolation valves in Table 3.6-1 to reflect installation of valves made during the 1982 refueling outages as follows:

1. Three valves have been added to reflect installation of the post-accident monitoring system required by Item II.F.1 of NUREG-0737.

2. Fourteen check valves in the non-essential service water system have been replaced with valves of superior closing characteristics.

3. Six valves have been added to reflect installation of the post-accident sampling system required by Item II.B.3 of NUREG-0737.

4. Locked closed manaul valves on the demineralized water supply for the refueling cavity were replaced with remote air operated valves. The manual valves were replaced in order to remove the administrative controls on the demineralized water system. Previously, the system was required to

be isolated through the employment of locked closed manual valves prior to the reactor coolant system entering mode 4 (Hot Shutdown). With the installation of the automatic isolation valves, the capability of containment isolation prior to entering mode 4 is demonstrated, allowing the use of demineralized water inside containment during modes 1, 2, 3, and 4. This valve replacment will enable a more expedious plant recovery and will thereby reduce plant downtime.

We have reviewed the proposed changes to Table 3.6-1 and conclude that the modifications are acceptable.

Environmental Consideration

We have determined that the amendments do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendments involve an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of these amendments.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendments do not involve a significant increase in the probability or consequences of an accident previously evaluated, do not create the possibility of an accident of a type different from any evaluated previously, and do not involve a significant reduction in a margin of safety, the amendments do not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Dated: September 9, 1982

Principal Contributors:
P. Hearn
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UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NOS. 50-315 AND 50-316

INDIANA AND MICHIGAN ELECTRIC COMPANY

NOTICE OF ISSUANCE OF AMENDMENTS TO FACILITY OPERATING LICENSES

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 59 to Facility Operating License No. DPR-58, and Amendment No. 42 to Facility Operating License No. DPR-74 issued to Indiana and Michigan Electric Company (the licensee), which revised Technical Specifications for operation of Donald C. Cook Nuclear Plant, Unit Nos. 1 and 2 (the facilities) located in Berrien County, Michigan. The amendments are effective as of the date of issuance.

The amendments revise the Technical Specifications to reflect installation or replacement of specified containment isolation valves to promote improved operation.

The application for the amendments complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments. Prior public notice of these amendments was not required since the amendments do not involve a significant hazards consideration.

The Commission has determined that the issuance of these amendments will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of these amendments.

For further details with respect to this action, see (1) the application for amendments dated July 13, 1982, (2) Amendment Nos. 59 and 42 to License Nos. DPR-58 and DPR-74, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C. and at the Maude Reston Maude Memorial Library, 500 Market Street, St. Joseph, Michigan 49085. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 9th day of September, 1982.

FOR THE NUCLEAR REGULATORY COMMISSION

Marshall Grotenhuis

Marshall Grotenhuis, Acting Chief
Operating Reactors Branch #1
Division of Licensing