

DECEMBER 7 1979

Docket Nos. 50-315
and 50-316

Mr. John Dolan, Vice President
Indiana and Michigan Electric Company
Post Office Box 18
Bowling Green Station
New York, New York 10004

REGULATORY DOCKET FILE COPY

Dear Mr. Dolan:

The Commission has issued the enclosed Amendment No. 34 to Facility Operating License No. DPR-58 and Amendment No. 15 to Facility Operating License No. DPR-74 for the Donald C. Cook Nuclear Plant, Unit Nos. 1 and 2, respectively. The amendments consist of changes to the Technical Specifications in response to your applications transmitted by letters dated February 3, 1978 and March 7, 1979.

These amendments require the maximum allowable containment purge isolation valve closure times to be reduced from 10 seconds to 5 seconds.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

The enclosed Safety Evaluation deals primarily with our evaluation of a postulated fuel handling accident inside containment. With the issuance of these amendments we consider that matter resolved.

Sincerely,

A. Schwencer

12/6/79

A. Schwencer, Chief
Operating Reactors Branch #1
Division of Operating Reactors

Enclosures:

- 1. Amendment No. 34 to DPR-58
- 2. Amendment No. 15 to DPR-74
- 3. Safety Evaluation
- 4. Notice of Issuance

cc: w/enclosures
See next page

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and 50-316

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

December 7, 1979

Docket Nos. 50-315
and 50-316

Mr. John Dolan, Vice President
Indiana and Michigan Electric Company
Post Office Box 18
Bowling Green Station
New York, New York 10004

Dear Mr. Dolan:

The Commission has issued the enclosed Amendment No. 34 to Facility Operating License No. DPR-58 and Amendment No. 15 to Facility Operating License No. DPR-74 for the Donald C. Cook Nuclear Plant, Unit Nos. 1 and 2, respectively. The amendments consist of changes to the Technical Specifications in response to your applications transmitted by letters dated February 3, 1978 and March 7, 1979.

These amendments require the maximum allowable containment purge isolation valve closure times to be reduced from 10 seconds to 5 seconds.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

The enclosed Safety Evaluation deals primarily with our evaluation of a postulated fuel handling accident inside containment. With the issuance of these amendments we consider that matter resolved.

Sincerely,

A handwritten signature in cursive script, appearing to read "A. Schwencer".

A. Schwencer, Chief
Operating Reactors Branch #1
Division of Operating Reactors

Enclosures:

1. Amendment No. 34 to DPR-58
2. Amendment No. 15 to DPR-74
3. Safety Evaluation
4. Notice of Issuance

cc: w/enclosures
See next page

Mr. John Dolan
Indiana and Michigan Electric Company - 2 -

December 7, 1979

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P. O. Box 30035
Lansing, Michigan 48909



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

INDIANA AND MICHIGAN ELECTRIC COMPANY

DOCKET NO. 50-315

DONALD C. COOK NUCLEAR PLANT UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 34
License No. DPR-58

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The applications for amendment by Indiana and Michigan Electric Company (the licensee) dated February 3, 1978 and March 7, 1979, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-58 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 34, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



A. Schwencer, Chief
Operating Reactors Branch #1
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 7, 1979

ATTACHMENT TO LICENSE AMENDMENT NO. 34

FACILITY OPERATING LICENSE NO. DPR-58

DOCKET NO. 50-315

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided for document completeness.

Pages

3/4 6-20

3/4 6-21

TABLE 3.6-1 (Continued)

<u>VALVE NUMBER</u>	<u>FUNCTION</u>	<u>TESTABLE DURING PLANT OPERATION</u>	<u>ISOLATION TIME IN SECONDS</u>
B. PHASE "B" ISOLATION (Continued)			
9.	WCR-901	NESW to Low Containment Vent #1	Yes 10
10.	WCR-903	NESW from Low Containment Vent #1	Yes 10
11.	WCR-905	NESW to Low Containment Vent #2	Yes 10
12.	WCR-907	NESW from Low Containment Vent #2	Yes 10
13.	WCR-909	NESW to Low Containment Vent #3	Yes 10
14.	WCR-911	NESW from Low Containment Vent #3	Yes 10
15.	WCR-913	NESW to Low Containment Vent #4	Yes 10
16.	WCR-915	NESW from Low Containment Vent #4	Yes 10
17.	WCR-921	NESW to Up Containment Vent #1	Yes 10
18.	WCR-923	NESW from Up Containment Vent #1	Yes 10
19.	WCR-925	NESW to Up Containment Vent #2	Yes 10
20.	WCR-927	NESW from Up Containment Vent #2	Yes 10
21.	WCR-929	NESW to Up Containment Vent #3	Yes 10
22.	WCR-931	NESW from Up Containment Vent #3	Yes 10
23.	WCR-933	NESW to Up Containment Vent #4	Yes 10
24.	WCR-935	NESW from Up Containment Vent #4	Yes 10
25.	WCR-945	NESW from RCP Motor Air Cooler	Yes 10
26.	WCR-946	NESW from RCP Motor Air Cooler	Yes 10
27.	WCR-947	NESW from RCP Motor Air Cooler	Yes 10
28.	WCR-948	NESW from RCP Motor Air Cooler	Yes 10
29.	WCR-951	NESW to RCP Motor Air Cooler Vent #1	Yes 10
30.	WCR-952	NESW to RCP Motor Air Cooler Vent #2	Yes 10
31.	WCR-953	NESW to RCP Motor Air Cooler Vent #3	Yes 10
32.	WCR-954	NESW to RCP Motor Air Cooler Vent #4	Yes 10
33.	WCR-955	NESW from RCP Motor Air Cooler Vent #1	Yes 10
34.	WCR-956	NESW from RCP Motor Air Cooler Vent #2	Yes 10
35.	WCR-957	NESW from RCP Motor Air Cooler Vent #3	Yes 10
36.	WCR-958	NESW from RCP Motor Air Cooler Vent #4	Yes 10

TABLE 3.6-1 (Continued)

<u>VALVE NUMBER</u>	<u>FUNCTION</u>	<u>TESTABLE DURING PLANT OPERATION</u>	<u>ISOLATION TIME IN SECONDS</u>
<u>B. PHASE "B" ISOLATION (Continued)</u>			
37. WCR-961	NESW to Instr. Rm. East Vent	Yes	10
38. WCR-963	NESW from Instr. Rm. West Vent	Yes	10
39. WCR-965	NESW to Instr. Rm. East Vent	Yes	10
40. WCR-967	NESW from Instr. Rm. West Vent	Yes	10
41. WCR-902	NESW from Lower Containment Vent #1	Yes	10
42. WCR-906	NESW from Lower Containment Vent #2	Yes	10
43. WCR-910	NESW from Lower Containment Vent #3	Yes	10
44. WCR-914	NESW from Lower Containment Vent #4	Yes	10
45. WCR-922	NESW from Upper Containment Vent #1	Yes	10
46. WCR-926	NESW from Upper Containment Vent #2	Yes	10
47. WCR-930	NESW from Upper Containment Vent #3	Yes	10
48. WCR-934	NESW from Upper Containment Vent #4	Yes	10
49. WCR-962	NESW from Instrument Room East Vent	Yes	10
50. WCR-966	NESW from Instrument Room West Vent	Yes	10
<u>C. CONTAINMENT PURGE AND EXHAUST</u>			
1. VCR-101	Instr. Room Purge Air Inlet	Yes	5
2. VCR-102	Instr. Room Purge Air Outlet	Yes	5
3. VCR-103	Lower Comp. Purge Air Inlet	Yes	5
4. VCR-104	Lower Comp. Purge Air Outlet	Yes	5
5. VCR-105	Upper Comp. Purge Air Inlet	Yes	5
6. VCR-106	Upper Comp. Purge Air Outlet	Yes	5
7. VCR-107*	Cont. Press. Relief Fan Isolation	Yes	5
8. VCR-201	Instr. Room Purge Air Inlet	Yes	5
9. VCR-202	Instr. Room Purge Air Outlet	Yes	5
10. VCR-203	Lower Comp. Purge Air Inlet	Yes	5
11. VCR-204	Lower Comp. Purge Air Outlet	Yes	5

D. C. COOK-UNIT 1

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Attachment No. 34

TABLE 3.6-1 (Continued)

<u>VALVE NUMBER</u>	<u>FUNCTION</u>	<u>TESTABLE DURING PLANT OPERATION</u>	<u>ISOLATION TIME IN SECONDS</u>
<u>C. CONTAINMENT PURGE AND EXHAUST (Continued)</u>			
12. VCR-205	Upper Comp. Purge Air Inlet	Yes	5
13. VCR-206	Upper Comp. Purge Air Outlet	Yes	5
14. VCR-207*	Cont. Press Relief Fan Isolation	Yes	5
<u>D. MANUAL ISOLATION VALVES⁽¹⁾</u>			
1. ICM-111	RHR to RC Cold Legs	Yes	NA
2. ICM-129	RHR Inlet to Pumps	No	NA
3. ICM-250	Boron Injection Inlet	Yes	NA
4. ICM-251	Boron Injection Inlet	Yes	NA
5. ICM-260	Safety Injection Inlet	Yes	NA
6. ICM-265	Safety Injection Inlet	Yes	NA
7. ICM-305	RHR Suction from Sump	Yes	NA
8. ICM-306	RHR Suction from Sump	Yes	NA
9. ICM-311	RHR to RC Hot Legs	Yes	NA
10. ICM-321	RHR to RC Hot Legs	Yes	NA
11. DW-209	Demineralized Water Supply for Refueling Cavity	Yes	NA
12. DW-210	Demineralized Water Supply for Refueling Cavity	Yes	NA
13. NPX 151 VI	Dead Weight Tester	Yes	NA
14. PA 145*	Containment Service Air	No	NA
15. SF-151*	Refueling Water Supply	Yes	NA
16. SF-153*	Refueling Water Supply	Yes	NA
17. SF-159	Refueling Cavity Drain to Purification System	Yes	NA
18. SF-160	Refueling Cavity Drain to Purification System	Yes	NA
19. SI-171	Safety Injection Test Line	Yes	NA
20. SI-172	Accumulator Test Line	Yes	NA

TABLE 3.6-1 (Continued)

<u>VALVE NUMBER</u>	<u>FUNCTION</u>	<u>TESTABLE DURING PLANT OPERATION</u>	<u>ISOLATION TIME IN SECONDS</u>	
<u>D. MANUAL ISOLATION VALVES⁽¹⁾ (Continued)</u>				
21.	CCR-440	CCW from Main Steam Penetration	Yes	NA
22.	CCR-441	CCW from Main Steam Penetration	Yes	NA
23.	MCM-221	Main Steam to Auxiliary Feed Pump	No	NA
24.	MCM-231	Main Steam to Auxiliary Feed Pump	No	NA
25.	CCM-430	CCW to East Pressure Equalization Fan	Yes	NA
26.	CCM-431	CCW from East Pressure Equalization Fan	Yes	NA
27.	CCM-432	CCW to West Pressure Equalization Fan	Yes	NA
28.	CCM-433	CCW from West Pressure Equalization Fan	Yes	NA
29.	SM-8*	Upper Containment Sample	Yes	NA
30.	SM-10*	Upper Containment Sample	Yes	NA
31.	SM-4*	Instrument Room Sample	Yes	NA
32.	SM-6*	Instrument Room Sample	Yes	NA

NA - Manual Valve-Isolation time not applicable.

(1) - Includes motor operated valves which do not isolate automatically.

* - May be opened on an intermittent basis under administrative control.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

INDIANA AND MICHIGAN ELECTRIC COMPANY

DOCKET NO. 50-316

DONALD C. COOK NUCLEAR PLANT UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 15
License No. DPR-74

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The applications for amendment by Indiana and Michigan Electric Company (the licensee) dated February 3, 1978 and March 7, 1979, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;
and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-74 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 15, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



A. Schwencer, Chief
Operating Reactors Branch #1
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 7, 1979

ATTACHMENT TO LICENSE AMENDMENT NO. 15

FACILITY OPERATING LICENSE NO. DPR-74

DOCKET NO. 50-316

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. Revised pages are identified by Amendment number and contain vertical lines indicating the area of change. Corresponding overleaf pages are also provided to maintain document completeness.

Pages

3/4 6-25

3/4 6-26

TABLE 3.6-1 (Continued)
CONTAINMENT ISOLATION VALVES

<u>VALVE NUMBER</u>	<u>FUNCTION</u>	<u>ISOLATION TIME IN SECONDS</u>
B. <u>PHASE "B" ISOLATION (Continued)</u>		
45. WCR-922	NESW from Upper Containment Vent #1	< 10
46. WCR-926	NESW from Upper Containment Vent #2	< 10
47. WCR-930	NESW from Upper Containment Vent #3	< 10
48. WCR-934	NESW from Upper Containment Vent #4	< 10
49. WCR-962	NESW from Instrument Room East Vent	< 10
50. WCR-966	NESW from Instrument Room West Vent	< 10
C. <u>CONTAINMENT PURGE AND EXHAUST</u>		
1. VCR-101	Instr. Room Purge Air Inlet	< 5
2. VCR-102	Instr. Room Purge Air Outlet	< 5
3. VCR-103	Lower Comp. Purge Air Inlet	< 5
4. VCR-104	Lower Comp. Purge Air Outlet	< 5
5. VCR-105	Upper Comp. Purge Air Inlet	< 5

D. C. COOK - 2000 2

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Amendment No. 15

TABLE 3.6-1 (Continued)
CONTAINMENT ISOLATION VALVES

<u>VALVE NUMBER</u>	<u>FUNCTION</u>	<u>ISOLATION TIME IN SECONDS</u>
C. <u>CONTAINMENT PURGE AND EXHAUST</u> (Continued)		
6. VCR-106	Upper Comp. Purge Air Outlet	≤ 5
7. VCR-107*	Cont. Press. Relief Fan Isolation	≤ 5
8. VCR-201	Instr. Room Purge Air Inlet	≤ 5
9. VCR-202	Instr. Room Purge Air Outlet	≤ 5
10. VCR-203	Lower Comp. Purge Air Inlet	≤ 5
11. VCR-204	Lower Comp. Purge Air Outlet	≤ 5
12. VCR-205	Upper Comp. Purge Air Outlet	≤ 5
13. VCR-206	Upper Comp. Purge Air Outlet	≤ 5
14. VCR-207*	Cont. Press Relief Fan Isolation	≤ 5
D. <u>MANUAL ISOLATION VALVES</u> (1)		
1. 1CM-111#	RHR to RC Cold Legs	NA
2. 1CM-129	RHR Inlet to Pumps	NA



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 34 TO FACILITY OPERATING LICENSE NO. DPR-58
AND AMENDMENT NO. 15 TO FACILITY OPERATING LICENSE NO. DPR-74

INDIANA AND MICHIGAN ELECTRIC COMPANY

DONALD C. COOK NUCLEAR PLANT UNIT NOS. 1 AND 2

DOCKET NOS. 50-315 AND 50-316

Introduction

The Indiana and Michigan Electric Company and the Indiana and Michigan Power Company, the licensees for the D. C. Cook Nuclear Plant, Unit Nos. 1 and 2, at our request have performed an analysis for these units of the potential consequences of a postulated fuel handling accident inside containment (FHAIC) with the reactor in the cold shutdown condition with the pressure vessel cover removed.

In our December 1977 Supplement No. 7 to the D. C. Cook Unit No. 2 Safety Evaluation Report, we found that the FHAIC had been appropriately analyzed with acceptable consequences. In the process of conducting our evaluation of the Unit No. 1 FHAIC which is discussed in more detail below, we noted that, realistically, the consequences of an FHAIC would be affected by closure times for the containment purge isolation valves and that the valves can actually close much faster than the time allowed by the present Technical Specifications. The licensee, partially at our request, has proposed an appropriate change to the Technical Specifications to require faster valve closure times consistent with the times that can be practically obtained at these two facilities.

Discussion

The licensee's analysis concluded that this postulated accident could, using conservative assumptions, result in a dose to an individual at the site boundary of 82.3 rem to the thyroid and 1.3 rem to the whole body, values well within the guidelines of 10 CFR 100.

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Evaluation

We reviewed the licensees' submittals and performed an independent analysis of the FHAIC for the Cook facilities. Our analysis, including assumptions used and resulting consequences, is given in Table 1. Our independent analysis yielded similar results to those obtained by the licensees. We obtained doses of 56 rem to the thyroid and 0.2 rem to the whole body at the site boundary. Both we and the licensee conservatively assumed that the containment purge isolation valves did not close to mitigate the consequences of an FHAIC. However, we both agree that early closure of these valves is likely to reduce the above calculated consequences and, therefore a tightening of the maximum allowed containment purge isolation valve closure times from 10 seconds to 5 seconds is both prudent and acceptable.

As further considerations, we also looked into the likelihood or consequences of a second irradiated fuel assembly being impacted in the reactor core or in the spent fuel pool. For the reasons stated below, we did not find that such potential events warrant remedial action at this time.

A recent study has indicated that dropping a spent fuel assembly into the core during refueling operations may potentially cause damage to more fuel pins than has been assumed for evaluating the FHAIC. This study has indicated that up to all of the fuel pins in two spent fuel assemblies, the one dropped and the one hit, may be damaged because of irradiation caused embrittlement of fuel cladding.

The probability of the postulated fuel handling accident inside containment is small. Not only have there been several hundred reactor-years of plant operating experience with only a few accidents involving spent fuel being dropped into the core, but none of these accidents has resulted in measurable releases of activity. The potential damage to spent fuel estimated by the study was based on the assumption that a spent fuel assembly falls about 14 feet directly onto one other irradiated fuel assembly in the core; an impact which results in the greatest energy available for crushing the fuel pins in both assemblies. This type of impact is unlikely because the falling assembly would be subjected to drag forces in the water which should cause the assembly to skew out of a vertical fall path.

Based on the above, we have concluded that the likelihood of a spent fuel assembly falling into the core and damaging all fuel pins in two assemblies is sufficiently small that refueling inside containment is not a safety concern which requires immediate remedial action. Nevertheless, we also looked at the potential consequences. We assumed for this postulated

accident that the source term for both spent fuel assemblies is that given in Regulatory Guide 1.25. This is conservative because (1) these two assemblies should not have the power peaking factor and clad gap activity recommended in Regulatory Guide 1.25 and (2) the pool decontamination factor for inorganic iodine should be greater than that recommended in Regulatory Guide 1.25. The calculated potential radiological consequences at the exclusion area boundary for the complete rupture of fuel pins in two assemblies are twice the values given in Table 1. Because these potential consequences are within the guidelines of 10 CFR 100 using the conservative assumptions of Regulatory Guide 1.25, we have concluded that the potential consequences of this postulated accident are acceptable and no additional restrictions on fuel handling operations or other plant operating procedures are needed.

We also considered potential fuel pin damage for a similar accident involving two fuel assemblies in the spent fuel pool. For this, a drop of 2-1/2 feet was postulated and the analysis performed in the same manner as previously described. Results indicate that in this scenario damage to the missile or target is minimal. No fuel pins in either fuel assembly were calculated to be ruptured.

Environmental Consideration

The environmental impacts of an accident involving the handling of spent fuel inside containment has been addressed in Section 6.1 of the Final Environmental Statement dated August 1973.

We have determined that the amendments do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendments involve an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of these amendments.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendments do not involve a significant increase in the probability or consequences of accidents previously considered and do not involve a significant decrease in a safety margin, the amendments do not involve a significant hazards consideration, (2)

there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Date: December 7, 1979

References

Letter to the licensees from NRC dated January 17, 1977

Letter to NRC from the licensees dated March 21, 1977

Letter to the licensees from NRC dated August 5, 1977

Letter to NRC from the licensees dated September 30, 1977

Letter to NRC from the licensees dated February 3, 1978

Letter to the licensees from NRC dated January 3, 1979

Letter to NRC from the licensees dated March 7, 1979

J. N. Singh, "Fuel Assembly Handling Accident Analysis," EG&G Idaho
Technical Report RE-A-78-227, October 1978

Table 1

ASSUMPTIONS FOR AND POTENTIAL CONSEQUENCES OF THE POSTULATED
FUEL HANDLING ACCIDENTS AT THE EXCLUSION AREA BOUNDARY
FOR THE DONALD C. COOK NUCLEAR PLANT, UNITS 1 & 2

Assumptions:

Guidance in Regulatory
Guide 1.25

Power Level	3391 Mwt
Fuel Exposure Time	3 years
Power Peaking Factor	1.65
Equivalent Number of Assemblies damaged	1
Number of Assemblies in core	193
Charcoal Filters available	None
Decay time before moving fuel	100 hours
0-2 hours X/Q Value, Exclusion Area Boundary (ground level release)	$2.1 \times 10^{-4} \text{ sec/m}^3$

Doses, Rem

	<u>Thyroid</u>	<u>Whole Body</u>
Exclusion Area Boundary (EAB) Consequences from Accidents Inside Containment	56	0.2

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NOS. 50-315 AND 50-316INDIANA AND MICHIGAN ELECTRIC COMPANYNOTICE OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 34 to Facility Operating License No. DPR-58, and Amendment No. 15 to Facility Operating License No. DPR-74 issued to Indiana and Michigan Electric Company (the licensee), which revised Technical Specifications for operation of Donald C. Cook Nuclear Plant, Unit Nos. 1 and 2 (the facilities) located in Berrien County, Michigan. The amendments are effective as of the date of issuance.

The amendments require the maximum allowable purge isolation valve closure times to be reduced from 10 to 5 seconds.

The application for the amendments complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments. Prior public notice of these amendments was not required since the amendments do not involve a significant hazards consideration.

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- 2 -

The Commission has determined that the issuance of these amendments will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of these amendments.

For further details with respect to this action, see (1) the application for amendments dated February 3, 1978 and March 7, 1979, (2) Amendment Nos. 34 and 15 to License Nos. DPR-58 and DPR-74, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C. and at the Maude Reston Palenske Memorial Library, 500 Market Street, St. Joseph, Michigan 49085. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland, this 7th day of December, 1979

FOR THE NUCLEAR REGULATORY COMMISSION



A. Schwencer, Chief
Operating Reactors Branch #1
Division of Operating Reactors