Docket Nos. 50-315 50-316 JUL 1 0 1981

Mr. John Dolan, Vice President Indiana and Michigan Electric Company Post Office Box 18 Bowling Green Station New York, New York 10004

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Dear Mr. Dolan:

The Commission has issued the enclosed Order for confirming your commitments for TMI related requirements for the Donald C. Cook Nuclear Plant, Units 1 and 2. This Order is based on your letters dated January 8, February 9, and March 6, 1981, to implement those requirements set forth in NUREG-0737 for which the staff requested completion before June 30, 1981. Explicit statements of these requirements are set forth in the Attachment to the Order.

It is the Commission's intent in issuing this Order to encourage completion of items consistent with the staff's recommended schedule. However, certain of your proposed schedule exceptions are acceptable. Therefore, you should proceed with implementation of these items in accordance with your proposed schedule. Your proposed technical and schedule exceptions for other items contained in NUREG-0737 will be the subject of future correspondence.

A copy of this Order is being filed with the Office of the Federal Register for publication.

Sincerely,

Original signed by: S. A. Varga

Steven A. Varga, Chief Operating Reactors Branch #1 Division of Licensing

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Mr. John Dolan Indiana and Michigan Electric Company

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cc: Mr. Robert W. Jurgensen Chief Nuclear Engineer American Electric Power Service Corporation 2 Broadway New York, New York 10004

> Gerald Charnoff, Esquire Shaw, Pittman, Potts and Trowbridge 1800 M Street, N.W. Washington. D. C. 20036

Maude Preston Palenske Memorial Library 500 Market Street St. Joseph, Michigan 49085

Mr. D. Shaller, Plant Manager Donald C. Cook Nuclear Plant P. O. Box 458 Bridgman, Michigan 49106

U. S. Nuclear Regulatory Commission Resident Inspectors Office 7700 Red Arrow Highway Stevensville, Michigan 49127

Mr. Wade Schuler, Supervisor Lake Township Baroda, Michigan 49101

Mr. William R. Rustem (2) Office of the Governor Room 1 - Capitol Building Lansing, Michigan 48913 Honorable James Bemenek, Mayor City of Bridgman, Michigan 49106

U. S. Environmental Protection Agency Federal Activities Branch Region V Office ATTN: EIS COORDINATOR 230 South Dearborn Street Chicago, Illinois 60604

Maurice S. Reizen, M.D. Director Department of Public Health P. O. Box 30035 Lansing, Michigan 48909

William J. Scanlon, Esquire 2034 Pauliné Boulevard Ann Arbor, Michigan 48103

#### UNITED STATES OF AMERICA

## NUCLEAR REGULATORY COMMISSION

In the Matter of
INDIANA AND MICHIGAN ELECTRIC COMPANY
(Donald C. Cook Nuclear Plant,
Units No. 1 and 2)

8107290035 810 PDR ADOCK 0500 Docket Nos. 50-315 50-316

## ORDER CONFIRMING LICENSEE COMMITMENTS ON POST-TMI RELATED ISSUES

Ι.

Indiana and Michigan Electric Company (the licensee) is the holder of Facility Operating Licenses Nos. DPR-58 and DPR-74, which authorize the operation of the Donald C. Cook Nuclear Plant, Units 1 and 2 (the facilities) at steady-state power levels not in excess of 3350 and 3391 megawatts thermal respectively, for each unit. The facilities consist of pressurized water reactors (PWRs) located at the licensee's site in Barrien County, Michigan.

II.

Following the accident at Three Mile Island Unit No. 2 (TMI-2) on March 28, 1979, the Nuclear Regulatory Commission (NRC) staff developed a number of proposed requirements to be implemented on operating reactors and on plants under construction. These requirements include Operational Safety, Siting and Design, and Emergency Preparedness and are intended to provide substantial additional protection for the operation of nuclear facilities based on the experience from the accident at TMI-2 and the official studies and investigations of the accident. The staff's proposed requirements and schedule for implementation are set forth in NUREG-0737, "Clarification of TMI Action Plan Requirements." Among these requirements are a number of items, consisting of hardware modifications, administrative procedure implementation and specific information to be submitted by the licensee, scheduled to be completed on or before June 30, 1981 (see the Attachment to this Order). NUREG-0737 was transmitted to each licensee and applicant by an NRC letter from my office dated October 31, 1980, which is hereby incorporated by reference. In that letter, it was indicated that although the NRC staff expected each requirement to be implemented in accordance with the schedule set forth in NUREG-0737, the staff would consider licensee requests for relief from staff proposed requirements and their associated implementation dates.

### III.

The licensee's submittals dated January 8, February 9 and March 6, 1981 and the references stated therein, which are incorporated herein by reference, committed to complete each of the actions specified in the Attachment. The licensee's submittals included a modified schedule for submittal of certain information. The staff has reviewed the licensee's submittals and determined that the licensee's modified schedule is acceptable based on the following:

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The licensee's schedule for submittal of information in some instances does not meet the staff's specified submittal dates. Most of the information requested by the staff describes how the licensee is meeting the guidance of NUREG-0737. Therefore, this deferral of the licensee submittal will not alter the implementation of plant modifications. Therefore, plant safety is not affected by this modification in schedule for the submittal of information.

I have determined that these commitments are required in the interest of public health and safety, and therefore, should be confirmed by ORDER.

## IV.

Accordingly, pursuant to Sections 103, 161i, 161o, and 182 of the Atomic Energy Act of 1954, as amended, and the Commission's regulations in 10 CFR Parts 2 and 50, IT IS HEREBY ORDERED EFFECTIVE IMMEDIATELY THAT the licensee shall comply with the following conditions:

The licensee shall satisfy the specific requirements described in the Attachment to this Order (as appropriate to the licensee's facility) as early as practicable but no later than 30 days after the effective date of the ORDER.

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Any person who has an interest affected by this Order may request a hearing within 20 days of the date of publication of this Order in the <u>Federal Register</u>. Any request for a hearing shall be addressed to the Director, Office of Nuclear Reactor Regulation, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555. A copy shall also be sent to the Executive Legal Director at the same address. If a hearing is requested by a person other than the licensee, that person shall describe, in accordance with 10 CFR 2.714(a)(2), the nature of the person's interest and the manner in which the interest is affected by this Order. A REQUEST FOR HEARING SHALL NOT STAY THE IMMEDIATE EFFECTIVENESS OF THIS ORDER.

If a hearing is requested by the licensee or other persons who have an interest affected by this Order, the Commission will issue an Order designating the time and place of any such hearing.

If a hearing is held concerning this Order, the issue to be considered at the hearing shall be whether, on the basis of the information set forth in Sections II and III of this Order, the licensee should comply with the conditions set forth in Section IV of this Order. This request for information was approved by OMB under clearance number 3150-0065 which expires June 30, 1983. Comments on burden and duplication may be directed to the Office of Management and Budget, Reports Management, Room 3208, New Executive Office Building, Washington, D. C.

This Order is effective upon issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

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Division of Licensing Office of Nuclear Reactor Regulation

Dated at Bethesda, Maryland this 10th day of July, 1981

Attachment: NUREG-0737 Requirements 7590-01

ATTACHMENT

# NUREG 0737 REQUIREMENTS

Item	<u>Title</u> <u>A</u>	pplicability		Impl/Doc Due Dates
I.A.1.1	STA	ALL	<ul> <li>(a) Provide STA coverage by degreed individuals (or equivalent), trained to licensee's program.</li> </ul>	1/1/81
· ,			(b) Submit description of current training program and demonstrate compliance with 10-30-79 letter.	1/1/81
			(c) Submit description of long term training program.	1/1/81
I.A.2.1	Reactor Operator Qualification	ALL	Submit description of and imple- ment Upgraded Training Program.	8/1/80
I.C.1	Accident Procedures	ALL	<ul> <li>(a) Submit reanalyses of inade- quate core cooling and propose guidelines for emergency procedures, OR propose schedule and justify delays.</li> </ul>	1/1/81
			(b) Submit reanalysis of transients and accidents for emergency procedures, OR propose schedule and justify delays.	1/1/81
1.C.5	Feedback of Operating Experience	ALL	Implement procedures for feedback of operating experience (no documentation submittal is required).	1/1/81
I.C.6	Correct Performance or Operating Activit	ALL ies	Implement procedures to verify correct performance of operating activities (no documentation submittal is required).	1/1/81

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Item	Title_	Applicability	Specific Requirement	Imp1/Doc Subm Due Dates
II.B.2	Shielding	ALL	(a) Have available design details for vital area modifications.	1/1/81
-			(b) Submit technical deviations to staff positions.	1/1/81
II.B.3	Post-Accident Sampling	ALL	Submit description of deviations from staff positions.	1/1/81
II.B.4 .	Training to Mitigate Core Damage	ALL	<ul> <li>(a) Have available for review a training program for mitigating core damage (no documen- tation submittal require</li> </ul>	
			(b) Implement training program (no documentation submittal required).	4/1/81 on
II.D.1	Performance Testing of RV/SRV's	ALL	<pre>(a) Submit test program    (both BWR/PWR's).</pre>	7/1/80 (PWRs 10/1/80 (BWRs
			(b) Submit qualification program for PWR's block valve.	1/1/81 (PWRs)
II.E.1.2	Aux Feed Initiation and Flow	PWRs	<ul> <li>(a) Submit final design and documentation on safety grade flow indication.</li> </ul>	1/1/81
			(b) Submit final design and documentation on safety grade flow initiation.	1/1/81
I.E.4.2	Containment Isolation	ALL	<ul> <li>(a) Submit documentation justifying minimum conta ment pressure setpoint isolation of non-essent penetrations.</li> </ul>	for #
		·	<ul> <li>(b) Submit statement that purge valves not meeting CSB 6-4 (or interim pos- tion)are sealed and ver cation is performed even 31 days.</li> </ul>	i- ifi-
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Item	Title	Applicability	Specific Requirement	Impl/Doc Jbm Due Dates
<u>II.</u> F.1	Post-Accident Monitoring	ALL	<ul> <li>a. For noble gas monitor and Iodine/particulate sampling and analysis submit description and justification for deviations from staff requirements.</li> </ul>	1/1/81
			b. Have available the final design infor- mation for noble gas monitor and iodine/ particulate sampling and analysis.	1/1/81
II.F.2	Inst. for Inadequat Core Cooling	e ALL	Submit a report detailing the planned instrumentation system for monitoring inadequate core cooling.	
II.K.2.10	Antic. Trip on LOFW and TT	B&W	Submit final design for anticipatory trip as described in NUREG 0737.	1/1/81
II.K.2.13	Thermal Mechanical Report	B&W	Submit report on effects of Hi Pressure Injection on vessel integrity for SB LOCA with no Aux Feedwater.	1/1/81
II.K.3.2	PORV/SV Failures	PWR	Submit report on SB LOCA and probability of failure of PORV/SV/RV.	1/1/81
II.K.3.3	SRV/SV Failures & Challenges	ALL	Submit report (historical and annually thereafter) of SRV/SV failures and challenges.	1/1/81
II.K.3.7	PORV Opening Probability	B&W	Submit report on the probability of a PORV opening during an over-pressurization transient.	1/1/81
II.K.3.9		Selected <u>W</u> plants	Modify the Proportional Integral Derivative Controller (as recommended by <u>W</u> ). Advise NRC when modification is comple	12/1/80 eted.
II.K.3.12	Anticipatory Trip on Turbine Trip	W	Submit confirmation of Anticipatory Trip. If not currently implemented, submit modification design and schedu for implementation.	1/1/81

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Item	Title	Applicability	Specific Requirement	Imp1/Doc Subm Dates
II.K.3.13	Separation of HPCI/RCIC Initiation Levels; Auto Restart of RCIC	•	Submit results of evaluation and proposed modification as appropriate.	1/1/81
II.K.3.16	Reduce Challenges to RV's	BWR	Submit report on actions planned to reduce RV challenges	4/1/81
II.K.3.17	ECCS System Outage	s ALL	Submit report on ECCS outages and propose changes " to reduce outages.	1/1/81
II.K.3.18	ADS Logic Modifications	BWR	Submit report of feasibility of ADS system logic changes to eliminate need for manual actuation.	4/1/81
II.K.3.21	CSS/CPCI Restart	BWR	Submit report of evaluation, proposed modifications and analysis to satisfy staff positions.	1/1/81
II.K.3.22	RCIC Suction	BWR w/RCIC	Implement procedures and document verification of this change.	1/1/81
II.K.3.27	Common Reference for H <sub>2</sub> 0 Level Inst	BWR ruments	Implement change and submit documentation of changes.	1/1/81
II.K.3.29	Isolation Condensor Performance	BWR w/ICs	Submit evaluation of I.C. performance.	4/1/81
II.K.3.30	SB LOCA Methods	ALL	Submit outline of program for model.	11/15/80
II.K.3.44	Fuel Failure	BWR	Submit evaluation to verify no fuel failure.	1/1/81
II.K.3.45	Manual Depressur- ization	BWR	Submit evaluation on other than ADS method for depressurization.	1/1/81

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<u>'Item</u>	Title	Applicability	Specific Requirement	·Imp1/Doc Subm Due Dates
III.D.3.3	Improved Inplant Iodine Monitoring	ALL	Have available means to accurately measure airborne radioiodine inplant during an accident.	1/1/81
III.D.3.4	Control Room Habit	ability ALL	a. Submit control room habitability evaluation information.	1/1/81
			b. Submit modifications necessary to assure CR habitability with a schedule for completion	1/1/81 -

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