

AUG 18 1981

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Docket File  
NRC PDR  
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ORB 1 File  
D. Eisenhut  
J. Wetmore  
S. Varga  
J. Thoma  
C. Parrish  
OI&E (5)  
OELD  
G. Deegan (information)  
ACRS (10)



Docket Nos. 50-315  
and 50-316

Mr. John Dolan, Vice President  
Indiana and Michigan Electric Company  
P. O. Box 18  
Bowling Green Station  
New York, New York 10004

Dear Mr. Dolan:

This confirms our telephone authorizations given on August 14, 1981 and August 17, 1981 which temporarily redefined the definition of operable in your Technical Specifications. Basically you discovered certain safety related equipment was not adequately supported for a seismic event and desired to continue operation while repairs were accomplished.

Our authorization allows continued operation at reduced power until temporary bracing is installed around the appropriate cabinets, motor control centers, and switchgear. Then operation at 100% power is authorized. Permanent modifications are expected to be completed by August 21, 1981. Details of the permanent modifications are to be submitted to the staff within thirty days of completing the repairs.

Facility Operating Licenses No. DPR-58 and DPR-74 are amended by making the following Technical Specification changes:

<u>Unit No.</u>	<u>Remove Pages</u>	<u>Insert Pages</u>
1 and 2	1-1 - 1-2	1-1 1-1a 1-2

HA  
4

Copies of the license amendment, our safety evaluation report, and the Federal Register Notice will be sent to you when completed.

Sincerely,

Original [Signature]

Thomas M. Novak, Assistant Director  
for Operating Reactors  
Division of Licensing

Enclosure:  
As stated

OFFICE	cc: See next page	ORB 1	ORB 1	AD:OR			
SURNAME	J. Thoma	J. Thoma	S. Varga	T. Novak			
	8/18/81	8/18/81	8/18/81	8/18/81			

8109110087 810818  
PDR ADOCK 05000315  
PDR

OFFICIAL RECORD COPY

Mr. John Dolan  
Indiana and Michigan Electric Company

cc: Mr. Robert W. Jurgensen  
Chief Nuclear Engineer  
American Electric Power  
Service Corporation  
2 Broadway  
New York, New York 10004

Gerald Charnoff, Esquire  
Shaw, Pittman, Potts and Trowbridge  
1800 M Street, N.W.  
Washington, D. C. 20036

Maude Preston Palenske Memorial  
Library  
500 Market Street  
St. Joseph, Michigan 49085

Mr. D. Shaller, Plant Manager  
Donald C. Cook Nuclear Plant  
P. O. Box 458  
Bridgman, Michigan 49106

U. S. Nuclear Regulatory Commission  
Resident Inspectors Office  
7700 Red Arrow Highway  
Stevensville, Michigan 49127

Mr. Wade Schuler, Supervisor  
Lake Township  
Baroda, Michigan 49101

Mr. William R. Rustem (2)  
Office of the Governor  
Room 1 - Capitol Building  
Lansing, Michigan 48913

Honorable James Bemeneck, Mayor  
City of Bridgman, Michigan 49106

Regional Radiation Representative  
EPA Region V  
230 South Dearborn Street  
Chicago, Illinois 60604

Maurice S. Reizen, M.D.  
Director  
Department of Public Health  
P.O. Box 30035  
Lansing, Michigan 48109

William J. Scanlon, Esquire  
2034 Pauline Boulevard  
Ann Arbor, Michigan 48103

The Honorable Tom Corcoran  
United States House of Representatives  
Washington, D. C. 20515

## 1.0 DEFINITIONS

### DEFINED TERMS -

1.1 The DEFINED TERMS of this section appear in capitalized type and are applicable throughout these Technical Specifications.

### THERMAL POWER

1.2 THERMAL POWER shall be the total reactor core heat transfer rate to the reactor coolant.

### RATED THERMAL POWER

1.3 RATED THERMAL POWER shall be a total reactor core heat transfer rate to the reactor coolant of 3391 MWt.

### OPERATIONAL MODE

1.4 An OPERATIONAL MODE shall correspond to any one inclusive combination of core reactivity condition, power level and average reactor coolant temperature specified in Table 1.1.

### ACTION

1.5 ACTION shall be those additional requirements specified as corollary statements to each principle specification and shall be part of the specifications.

### OPERABLE - OPERABILITY

1.6 A system, subsystem, train, component or device shall be OPERABLE or have OPERABILITY when it is capable of performing its specified function(s). Implicit in this definition shall be the assumption that all necessary attendant instrumentation, controls, normal and emergency electrical power sources, cooling or seal water, lubrication or other auxiliary equipment that are required for the system, subsystem, train, component or device to perform its function(s) are also capable of performing their related support function(s).\*

\*On August 14, 1981 and August 17, 1981 the following safety-related cabinets, motor control centers, and switchgear were discovered to have inadequate seismic support: (a) 600V motor control centers, Unit 2 only numbers MCC-2-AM-A and D, (b) 4 KV Switchgear Busses/Cabinets numbers T11 A,B,C, and D for Unit 1 and T21 A,B,C, and D for Unit 2, and (c) 600V Switchgear Buses/Cabinets numbers 11 A,B,C and D for Unit 1 and 21 A,B,C, and D for Unit 2. The above equipment shall be declared operable providing the following restrictions are met:

\* Continued

- (1) Reactor power shall be decreased until either 60% reactor power is reached or temporary bracing of one train of equipment is completed.
- (2) If the temporary bracing is not installed within 8 hours after reaching 60% power, further approval for operation must be obtained from the NRC Staff.
- (3) The unit may be operated up to 100% power after temporary bracing is installed on one train of equipment. However, the second train must be properly braced within 24 hours of the first train being braced.
- (4) Permanent modifications to provide adequate seismic support for the equipment involved shall be completed by August 21, 1981.

## DEFINITIONS

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### REPORTABLE OCCURRENCE

1.7 A REPORTABLE OCCURRENCE shall be any of those conditions specified as a reportable occurrence in Revision 4 of Regulatory Guide 1.16, "Reporting of Operating Information - Appendix "A" Technical Specifications."

### CONTAINMENT INTEGRITY

1.8 CONTAINMENT INTEGRITY shall exist when:

1.8.1 All penetrations required to be closed during accident conditions are either:

- a. Capable of being closed by an OPERABLE containment automatic isolation valve system, or
- b. Closed by manual valves, blind flanges, or deactivated automatic valves secured in their closed positions, except as provided in Table 3.6-1 of Specification 3.6.3.1.

1.8.2 All equipment hatches are closed and sealed,

1.8.3 Each air lock is OPERABLE pursuant to Specification 3.6.1.3, and

1.8.4 The containment leakage rates are within the limits of Specification 3.6.1.2.

### CHANNEL CALIBRATION

1.9 A CHANNEL CALIBRATION shall be the adjustment, as necessary, of the channel output such that it responds with the necessary range and accuracy to known values of the parameter which the channel monitors. The CHANNEL CALIBRATION shall encompass the entire channel including the sensor and alarm and/or trip functions, and shall include the CHANNEL FUNCTIONAL TEST. The CHANNEL CALIBRATION may be performed by any series of sequential, overlapping or total channel steps such that the entire channel is calibrated.

### CHANNEL CHECK

1.10 A CHANNEL CHECK shall be the qualitative assessment of channel behavior during operation by observation. This determination shall include, where possible, comparison of the channel indication and/or status with other indications and/or status derived from independent instrument channels measuring the same parameter.

## 1.0 DEFINITIONS

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### THERMAL POWER

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### RATED THERMAL POWER

1.3 RATED THERMAL POWER shall be a total reactor core heat transfer rate to the reactor coolant of 3391 Mwt.

### OPERATIONAL MODE

1.4 An OPERATIONAL MODE shall correspond to any one inclusive combination of core reactivity condition, power level and average reactor coolant temperature specified in Table 1.1.

### ACTION

1.5 ACTION shall be those additional requirements specified as corollary statements to each principle specification and shall be part of the specifications.

### OPERABLE - OPERABILITY

1.6 A system, subsystem, train, component or device shall be OPERABLE or have OPERABILITY when it is capable of performing its specified function(s). Implicit in this definition shall be the assumption that all necessary attendant instrumentation, controls, normal and emergency electrical power sources, cooling or seal water, lubrication or other auxiliary equipment that are required for the system, subsystem, train, component or device to perform its function(s) are also capable of performing their related support function(s).\*

\*On August 14, 1981 and August 17, 1981 the following safety-related cabinets, motor control centers, and switchgear were discovered to have inadequate seismic support: (a) 600V motor control centers, Unit 2 only numbers MCC-2-AM-A and D, (b) 4 KV Switchgear Busses/Cabinets numbers T11 A,B,C, and D for Unit 1 and T21 A,B,C, and D for Unit 2, and (c) 600V Switchgear Buses/Cabinets numbers 11 A,B,C and D for Unit 1 and 21 A,B,C, and D for Unit 2. The above equipment shall be declared operable providing the following restrictions are met:

\* Continued

- (1) Reactor power shall be decreased until either 60% reactor power is reached or temporary bracing of one train of equipment is completed.
- (2) If the temporary bracing is not installed within 8 hours after reaching 60% power, further approval for operation must be obtained from the NRC Staff.
- (3) The unit may be operated up to 100% power after temporary bracing is installed on one train of equipment. However, the second train must be properly braced within 24 hours of the first train being braced.
- (4) Permanent modifications to provide adequate seismic support for the equipment involved shall be completed by August 21, 1981.

## DEFINITIONS

### REPORTABLE OCCURRENCE

1.7 A REPORTABLE OCCURRENCE shall be any of those conditions specified in Specifications 6.9.1.8 and 6.9.1.9.

### CONTAINMENT INTEGRITY

1.8 CONTAINMENT INTEGRITY shall exist when:

1.8.1 All penetrations required to be closed during accident conditions are either:

- a. Capable of being closed by an OPERABLE containment automatic isolation valve system, or
- b. Closed by manual valves, blind flanges, or deactivated automatic valves secured in their closed positions, except as provided in Table 3.6-1 of Specification 3.6.3.1.

1.8.2 All equipment hatches are closed and sealed,

1.8.3 Each air lock is OPERABLE pursuant to Specification 3.6.1.3,

1.8.4 The containment leakage rates are within the limits of Specification 3.6.1.2, and

1.8.5 The sealing mechanism associated with each penetration (e.g., welds, bellows or O-rings) is OPERABLE.

### CHANNEL CALIBRATION

1.9 A CHANNEL CALIBRATION shall be the adjustment, as necessary, of the channel output such that it responds with the necessary range and accuracy to known values of the parameter which the channel monitors. The CHANNEL CALIBRATION shall encompass the entire channel including the sensor and alarm and/or trip functions, and shall include the CHANNEL FUNCTIONAL TEST. The CHANNEL CALIBRATION may be performed by any series of sequential, overlapping or total channel steps such that the entire channel is calibrated.

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