

April 10, 2002

NOTE TO: FILE

FROM: Ronald W. Hernan, Senior Project Manager, Section 2 */RA/*
Project Directorate II
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

SUBJECT: REVISED TECHNICAL SPECIFICATION PAGES FOR SEQUOYAH TSC 01-06
(TAC NOS. MB3879 AND MB3880)

The Tennessee Valley Authority (TVA), in response to a telephone request from the U.S. Nuclear Regulatory Commission (NRC) on April 5, 2002, submitted the attached revisions to Technical Specification (TS) 4.4.5.5.c for Sequoyah Nuclear Plant, Units 1 and 2. By letter dated November 8, 2002, TVA requested that a number of reporting requirements be removed from the TSs, including reporting Category C-3 steam generator tube inspection results required by TS 4.4.5.5.c. The NRC staff, in its review of the TS amendment request, concluded that removal of TS 4.4.5.5.c was not justified and plans to deny that portion of TVA's requested changes. The attached pages, electronically sent to the NRC on April 8, 2002, restore TS 4.4.5.5.c to the TSs and changes the reference from TS 6.6.1 to 10 CFR 50.73.

Docket Nos. 50-327 and 50-328

REACTOR COOLANT SYSTEM

SURVEILLANCE REQUIREMENTS (Continued)

- b. The steam generator shall be determined OPERABLE after completing the corresponding actions (plug all tubes exceeding the plugging limit and all tubes containing through-wall cracks) required by Table 4.4-2.

4.4.5.5 Reports

- a. Following each inservice inspection of steam generator tubes, the number of tubes plugged in each steam generator shall be reported to the Commission within 15 days.
- b. The complete results of the steam generator tube inservice inspection shall be submitted to the Commission in a Special Report pursuant to Specification 6.9.2 within 12 months following completion of the inspection. This Special Report shall include:
 - 1. Number and extent of tubes inspected.
 - 2. Location and percent of wall-thickness penetration for each indication of an imperfection.
 - 3. Identification of tubes plugged.
- c. Results of steam generator tube inspections which fall into Category C-3 shall be reported as a degraded condition pursuant to 10 CFR 50.73 prior to resumption of plant operation. The written followup of this report shall provide a description of investigations conducted to determine cause of the tube degradation and corrective measures taken to prevent recurrence.
- d. For implementation of the voltage-based repair criteria to tube support plate intersections, notify the staff prior to returning the steam generators to service should any of the following conditions arise:
 - 1. If estimated leakage based on the projected end-of-cycle (or if not practical using the actual measured end-of-cycle) voltage distribution exceeds the leak limit (determined from the licensing basis dose calculation for the postulated main steam line break) for the next operating cycle.
 - 2. If circumferential crack-like indications are detected at the tube support plate intersections.
 - 3. If indications are identified that extend beyond the confines of the tube support plate.
 - 4. If indications are identified at the tube support plate elevations that are attributable to primary water stress corrosion cracking.
 - 5. If the calculated conditional burst probability based on the projected end-of-cycle (or if not practical, using the actual measured end-of-cycle) voltage distribution exceeds 1×10^{-2} , notify the NRC and provide an assessment of the safety significance of the occurrence.

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