

JUL 17 1975

Docket No. 50-315

Indiana and Michigan Electric Company
Indiana and Michigan Power Company
ATTN: Mr. John A. Tillinghast
Vice President
P. O. Box 18
Bowling Green Station
New York, New York 10004

Gentlemen:

In response to your request of June 3, 1975, the Commission has issued Amendment No. 7 to Facility Operating License DPR-58 to change certain of the Technical Specifications, Appendix A of DPR-58. A signed copy of the amendment is enclosed. A copy of a related notice, which has been forwarded to the Office of the Federal Register for filing and publication, is also enclosed.

Amendment No. 7 to DPR-58 consists of Change No. 7 to the Technical Specifications. The specifications that have been changed are listed in Change No. 7.

The staff has concluded that Change No. 7 will not present any danger to the health and safety of the public nor will it result in (i) any significant increase in the probability of an accident or (ii) a significant increase in the consequences of an accident or (iii) a significant decrease in a safety margin.

Sincerely,

Original signed by
K. Kniel

Karl Kniel, Chief
Light Water Reactors Branch 2-2
Division of Reactor Licensing

o/p
1

Enclosures:

1. Amendment 7 to DPR-58
2. Staff Safety Evaluation
3. Federal Register Notice

ccs: Listed on page 2

cc

OFFICE	LWR 2-2	OELD	LWR 2-2			
SURNAME	<i>MS CAB</i>	<i>J.S. COHEN</i>	KKniel			
DATE	6/25/75	6/24/75	7/7/75			

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 - N. Goodrich
 - A. Steen
 - J. Mc Gough

ccs w/enclosure
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

INDIANA AND MICHIGAN ELECTRIC COMPANY

INDIANA AND MICHIGAN POWER COMPANY

DOCKET NO. 50-315

DONALD C. COOK NUCLEAR PLANT, UNIT 1

FACILITY OPERATING LICENSE

Amendment No. 7
License No. DPR-58

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Indiana and Michigan Electric Company and Indiana and Michigan Power Company (the licensees) dated June 3, 1975, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations; and
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.
2. Accordingly, the license is amended by a change to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C(2) of Facility License No. DPR-58 is hereby amended to read as follows:



"(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised, are hereby incorporated in the license. The licensees shall operate the facility in accordance with the Technical Specifications, as revised by issued changes thereto through Change No. 7."

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION:



Karl Kniel, Chief
Light Water Reactors Branch 2-2
Division of Reactor Licensing

Attachment:
Change No. 7 to the
Technical Specifications

Date of Issuance:

JUL 17 1975

CHANGE NO. 7

to

TECHNICAL SPECIFICATIONS

DPR-58

1. On page 3/4 6-18, change the Function of item 18 to read:
"NESW from Up Containment Vent #1."
2. On page 3/4 6-19, change the Functions of items B.38, B.39, and B.40 to read, respectively, as follows:
B.38 "NESW from Instrument Room East Vent"
B.39 "NESW to Instrument Room West Vent"
B.40 "NESW from Instrument Room West Vent"
3. On page 3/4 6-19, add the following to the list of Phase B Isolation:

<u>Valve No.</u>	<u>Function</u>	<u>Testable During Plant Operation</u>	<u>Isolation Time in sec.</u>
41. WCR-902	NESW from Lower Containment Vent #1	yes	10
42. WCR-906	NESW from Lower Containment Vent #2	yes	10
43. WCR-910	NESW from Lower Containment Vent #3	yes	10
44. WCR-914	NESW from Lower Containment Vent #4	yes	10
45. WCR-922	NESW from Upper Containment Vent #1	yes	10
46. WCR-926	NESW from Upper Containment Vent #2	yes	10
47. WCR-930	NESW from Upper Containment Vent #3	yes	10
48. WCR-934	NESW from Upper Containment Vent #4	yes	10
49. WCR-962	NESW from Instrument Room East Vent	yes	10
50. WCR-966	NESW from Instrument Room West Vent	yes	10

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-315

INDIANA AND MICHIGAN ELECTRIC COMPANY
INDIANA AND MICHIGAN POWER COMPANY

DONALD C. COOK NUCLEAR PLANT UNIT 1

NOTICE OF ISSUANCE OF AMENDMENT TO
FACILITY OPERATING LICENSE

Notice is hereby given that the U.S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 7 to Facility Operating License No. DPR-58 issued to Indiana and Michigan Electric Company and Indiana and Michigan Power Company. The amendment revises the Technical Specifications for operation of the Donald C. Cook Nuclear Plant Unit 1 located in Berrien County, Michigan, and is effective as of its date of issuance.

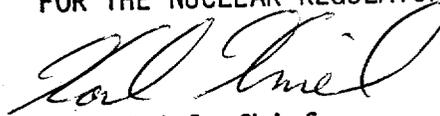
The amendment changes certain Technical Specifications to correct typographical and proofreading errors and to reflect changes made in certain equipment for assuring containment integrity.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings required by the Act and the Commission's rules and regulations in 10 CFR Chapter I. These findings are set forth in the license amendment. Prior public notice of this amendment is not required because the amendment does not involve a significant hazards consideration.

For further details with respect to this action, see (1) the application for amendment dated June 3, 1975, (2) Amendment No. 7 to License No. DPR-58 with Change No. 7, and (3) the Commission's related safety evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, NW, Washington, D. C., and at the St. Joseph Public Library, 500 Market Street, St. Joseph, Michigan 49085. A copy of items (2) and (3) may be obtained upon request addressed to the U.S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Reactor Licensing.

Dated at Bethesda, Maryland this 17th day of July 1975.

FOR THE NUCLEAR REGULATORY COMMISSION



Karl Kniel, Chief
Light Water Reactors Branch 2-2
Division of Reactor Licensing

SAFETY EVALUATION BY OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 7 TO LICENSE NO. DPR-58
CHANGE NO. 7 TO TECHNICAL SPECIFICATIONS
INDIANA & MICHIGAN ELECTRIC COMPANY
INDIANA & MICHIGAN POWER COMPANY
DONALD C. COOK NUCLEAR PLANT, UNIT 1
DOCKET NO. 50-315

INTRODUCTION

By letter dated June 3, 1975, the licensee requested changes to the Technical Specifications for the Donald C. Cook Nuclear Plant, Unit 1.

The changes, which apply to Appendix A to the license, include:

1. Changes to correct typographical and proofreading errors.
2. A change to reflect changes made in certain equipment for assuring containment integrity.

The staff's evaluation of each change is discussed below. The Item Numbers are the same as those in Attachment A to the licensee's June 3, 1975, request. The staff's conclusions follow the discussion.

DISCUSSION

Items 1 and 2: These changes have been made to correct typographical and proofreading errors on these pages.

Item 3: With the originally as-constructed non-essential service water system, the compounded events of loss-of-coolant accident, design basis earthquake, and failure of an existing isolation valve to close could lead to leakage of radioactivity directly from containment to the outside atmosphere. Although the occurrence of such a combination of events is highly unlikely, it is a design basis for the plant and it could produce a potential radioactivity release that has not been previously analyzed.

By the addition of the second isolation valve in each affected line, as proposed by the licensee, isolation capabilities of the affected system are augmented to be equal to those of systems that, by design, are open to the outside atmosphere and to the containment atmosphere. The isolation of such systems, described in the licensee's Final Safety Analysis Report, has been reviewed by the Regulatory Staff and found to be acceptable as reported in the Staff's Safety Evaluation dated September 10, 1973. The additional isolation valving conforms to the accepted isolation capabilities.

CONCLUSIONS

We have concluded, based on the considerations discussed above, that: (1) because the changes do not involve a significant increase in the probability or consequences of accidents previously considered and do not involve a significant decrease in a safety margin, the changes do not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.