

May 4, 1993

Docket No. 50-461

Mr. Frank A. Spangenberg
Licensing and Safety
Clinton Power Station
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Dear Mr. Spangenberg:

SUBJECT: ISSUANCE OF AMENDMENT (TAC NO. M84999)

The U. S. Nuclear Regulatory Commission (Commission) has issued the enclosed Amendment No. 73 to Facility Operating License No. NPF-62 for the Clinton Power Station, Unit No. 1. The amendment is in response to your application dated October 16, 1992 (U-602041).

The amendment makes editorial changes to the Clinton Power Station Facility Operating License and Technical Specifications (TS) to correct typographical errors, provide clarification, and remove provisions which are no longer applicable. The change to TS 6.12.3 that was requested in your application was incorporated in Amendment No. 69 issued on March 29, 1993, and is, therefore, not addressed in this amendment.

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

Original Signed By:

Douglas V. Pickett, Project Manager
Project Directorate III-2
Division of Reactor Projects III/IV/V
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 73 to NPF-62
2. Safety Evaluation

cc w/enclosures:
see next page

ENCLOSURE COPY

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CMoore
4/8/93

PE: PD32
RLaufer:rc
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PM: PD32
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DF01

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Illinois Power Company

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Unit No. 1

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

ILLINOIS POWER COMPANY, ET AL.

DOCKET NO. 50-461

CLINTON POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 73
License No. NPF-62

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Illinois Power Company¹ (IP), and Soyland Power Cooperative, Inc. (the licensees) dated October 16, 1992, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

¹ Illinois Power Company is authorized to act as agent for Soyland Power Cooperative, Inc. and has exclusive responsibility and control over the physical construction, operation and maintenance of the facility.

2. Accordingly, the license² is amended by changes to the Facility Operating License and to the Technical Specifications as indicated in the attachment to this license amendment, and paragraphs 2.C.(2) and 2.D. of Facility Operating License No. NPF-62 are hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. , are hereby incorporated into this license. Illinois Power Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

D. The facility requires exemptions from certain requirements of 10 CFR Part 50 and 10 CFR Part 70. These include: (a) an exemption from the requirements of 10 CFR 70.24 for the criticality alarm monitors around the fuel storage area; (b) an exemption from the requirement of paragraph III.D.2(b)(ii) of Appendix J, substituting the seal leakage test at Pa of paragraph III.D.2(b)(iii) for the entire airlock test at Pa of paragraph III.D.2(b)(ii) of Appendix J when no maintenance has been performed in the airlock that could affect its sealing capability (Section 6.2.6 of SSER 6); (c) an exemption from the requirement of paragraph III.C.3 of Appendix J, exempting the measured leakage rates from the main steam isolation valves from inclusion in the combined leak rate for the local leak rate tests (Section 6.2.6 of SSER 6); and (d) an exemption from the requirements of paragraph III.B.3 of Appendix J, exempting leakage from the valve packing and the body-to-bonnet seal of valve 1E51-F374 associated with containment penetration IMC-44 from inclusion in the combined leakage rate for penetrations and valves subject to Type B and C tests. The special circumstances regarding each exemption, except for Items (a) and (d) above, are identified in the referenced section of the safety evaluation report and the supplements thereto.

* * * * *

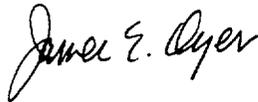
The special circumstances regarding the exemption identified in Item (d) above are identified in the safety evaluation accompanying Amendment No. 62 to this license.

2 Pages 5 and 6 are attached, for convenience, for the composite license to reflect this change.

These exemptions are authorized by law, will not present an undue risk to the public health and safety, and are consistent with the common defense and security. The exemptions in items (b) and (c) above are granted pursuant to 10 CFR 50.12. With these exemptions, the facility will operate, to the extent authorized herein, in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



James E. Dyer, Director
Project Directorate III-2
Division of Reactor Projects - III/IV/V
Office of Nuclear Reactor Regulation

Attachments:

1. License pages 5 and 6
2. Changes to the Technical Specifications

Date of Issuance: May 4, 1993

(8) Post-Fuel Loading Initial Test Program (Section 14, SER, SSER 5 and SSER 6)

Any changes to the initial test program described in Section 14 of the FSAR made in accordance with the provisions of 10 CFR 50.59 shall be reported in accordance with 50.59(b) within one month of such change.

(9) Emergency Response Capabilities (Generic Letter 82-33, Supplement 1 to NUREG-0737, Section 7.5.3.1, SSER 5 and SSER 8, and Section 18, SER, SSER 5 and Safety Evaluation Dated April 17, 1987)

- a. IP in accordance with the commitment contained in a letter dated December 11, 1986, shall install and have operational separate power sources for each of the fuel zone level channels as provided for in Regulatory Guide 1.97 prior to startup following the first refueling outage.
- b. IP shall submit a detailed control room design final supplemental summary report within 90 days of issuance of the full power license that completes all the remaining items identified in Section 18.3 of the Safety Evaluation dated April 17, 1987.

D. The facility requires exemptions from certain requirements of 10 CFR Part 50 and 10 CFR Part 70. These include: (a) an exemption from the requirements of 10 CFR 70.24 for the criticality alarm monitors around the fuel storage area; (b) an exemption from the requirement of paragraph III.D.2(b)(ii) of Appendix J, substituting the seal leakage test at Pa of paragraph III.D.2(b)(iii) for the entire airlock test at Pa of paragraph III.D.2(b)(ii) of Appendix J when no maintenance has been performed in the airlock that could affect its sealing capability (Section 6.2.6 of SSER 6); (c) an exemption from the requirement of paragraph III.C.3 of Appendix J, exempting the measured leakage rates from the main steam isolation valves from inclusion in the combined leak rate for the local leak rate tests (Section 6.2.6 of SSER 6); and (d) an exemption from the requirements of paragraph III.B.3 of Appendix J, exempting leakage from the valve packing and the body-to-bonnet seal of valve 1E51-F374 associated with containment penetration IMC-44 from inclusion in the combined leakage rate for penetrations and valves subject to Type B and C tests. The special circumstances regarding each exemption, except for Items (a) and (d) above, are identified in the referenced section of the safety evaluation report and the supplements thereto.

An exemption was previously granted pursuant to 10 CFR 70.24. The exemption was granted with NRC materials license No. SNM-1886, issued November 27, 1985, and relieved IP from the requirement of having a criticality alarm system. IP is hereby exempted from the criticality alarm system provision of 10 CFR 70.24 so far as this section applies to the storage of fuel assemblies held under this license.

The special circumstances regarding the exemption identified in Item (d) above are identified in the safety evaluation accompanying Amendment No. 62 to this license.

These exemptions are authorized by law, will not present an undue risk to the public health and safety, and are consistent with the common defense and security. The exemptions in items (b) and (c) above are granted pursuant to 10 CFR 50.12. With these exemptions, the facility will operate, to the extent authorized herein, in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission.

- E. The licensees shall fully implement and maintain in effect all provisions of the Commission-approved physical security plan, guard training and qualification, and safeguards contingency plans including amendments made pursuant to provisions of the Miscellaneous Amendments and Search Requirements revisions to 10 CFR 73.55 (51 FR 27817 and 27822) and to the authority of 10 CFR 50.90 and 10 CFR 50.54(p). The plans, which contain Safeguards Information protected under 10 CFR 73.21, are entitled: "Clinton Power Station Physical Security Plan," with revisions submitted through October 8, 1992; "Clinton Power Station Training and Qualification Plan," with revisions submitted through July 2, 1990; and "Clinton Power Station Safeguards Contingency Plan," with revisions submitted through July 2, 1990. Changes made in accordance with 10 CFR 73.55 shall be implemented in accordance with the schedule set forth therein.
- F. IP shall implement and maintain in effect all provisions of the approved fire protection program as described in the Final Safety Analysis Report as amended, for the Clinton Power Station, Unit No. 1, and as approved in the Safety Evaluation Report (NUREG-0853) dated February 1982 and Supplement Nos. 1 thru 8 thereto subject to the following provision:
- IP may make changes to the approved fire protection program without prior approval of the Commission only if those changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire.
- G. Except as otherwise provided in the Technical Specifications or Environmental Protection Plan, IP shall report any violations of the requirements contained in Section 2.C of this license in the following manner: initial notification shall be made within 24 hours to the NRC Operations Center via the Emergency Notification System with written followup within thirty days in accordance with the procedures described in 10 CFR 50.73(b), (c), and (e).

ATTACHMENT TO LICENSE AMENDMENT NO. 73

FACILITY OPERATING LICENSE NO. NPF-62

DOCKET NO. 50-461

Replace the following pages of the Appendix "A" Technical Specifications with the attached pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages, as indicated by an asterisk, are provided to maintain document completeness.

Remove Pages

3/4 5-5
*3/4 5-6
3/4 5-7
*3/4 5-8
*3/4 8-21
3/4 8-22
3/4 9-19
3/4 9-20

Insert Pages

3/4 5-5
*3/4 5-6
3/4 5-7
*3/4 5-8
*3/4 8-21
3/4 8-22
3/4 9-19
3/4 9-20

EMERGENCY CORE COOLING SYSTEMS

3/4.5.2 ECCS - SHUTDOWN

LIMITING CONDITION FOR OPERATION

3.5.2 At least two of the following shall be OPERABLE and capable of being powered from a diesel generator of Specification 3.8.1.2.b.

- a. The low pressure core spray (LPCS) system with a flow path capable of taking suction from the suppression pool and transferring the water through the spray sparger to the reactor vessel.
- b. Low pressure coolant injection (LPCI) subsystem "A" of the RHR system with a flow path capable of taking suction from the suppression pool and transferring the water to the reactor vessel.
- c. Low pressure coolant injection (LPCI) subsystem "B" of the RHR system with a flow path capable of taking suction from the suppression pool and transferring the water to the reactor vessel.
- d. Low pressure coolant injection (LPCI) subsystem "C" of the RHR system with a flow path capable of taking suction from the suppression pool and transferring the water to the reactor vessel.
- e. The high pressure core spray (HPCS) system with a flow path capable of taking suction from one of the following water sources and transferring the water through the spray sparger to the reactor vessel:
 1. From the suppression pool, or
 2. When the suppression pool level is less than the limit or is drained, from the RCIC storage tank containing at least 125,000 available gallons of water.

APPLICABILITY: OPERATIONAL CONDITIONS 4 and 5*.

ACTION:

- a. With one of the above required subsystems/systems inoperable, restore at least two subsystems/systems to OPERABLE status within 4 hours or suspend all operations that have a potential for draining the reactor vessel. The provisions of Specification 3.0.4 are not applicable.
- b. With both of the above required subsystems/systems inoperable, suspend CORE ALTERATIONS and all operations that have a potential for draining the reactor vessel. Restore at least one subsystem/system to OPERABLE status within 4 hours or establish PRIMARY CONTAINMENT INTEGRITY within the next 8 hours.

* The ECCS is not required to be OPERABLE provided that the reactor vessel head is removed, the cavity is flooded, the reactor cavity to steam dryer pool gate is open and water level in these upper containment pools is maintained within the limits of Specification 3.9.8 and 3.9.9.

EMERGENCY CORE COOLING SYSTEMS

3/4.5.3 SUPPRESSION POOL

LIMITING CONDITION FOR OPERATION

3.5.3 The suppression pool shall be OPERABLE:

- a. In OPERATIONAL CONDITIONS 1, 2, and 3 with a contained water volume of at least 146,400 ft³, equivalent to a level of 18'11".
- b. In OPERATIONAL CONDITIONS 4 and 5* with a contained water volume of at least 98,700 ft³, equivalent to a level of 12'8", except that the suppression pool level may be less than the limit or may be drained provided that:
 1. No operations are performed that have a potential for draining the reactor vessel,
 2. The reactor mode switch is locked in the Shutdown or Refuel position,
 3. The RCIC storage tank contains at least 125,000 available gallons of water, and
 4. The HPCS system is OPERABLE per Specification 3.5.2 with an OPERABLE flow path capable of taking suction from the RCIC storage tank and transferring the water through the spray sparger to the reactor vessel.

APPLICABILITY: OPERATIONAL CONDITIONS 1, 2, 3, 4, and 5*.

ACTION:

- a. In OPERATIONAL CONDITION 1, 2, or 3 with the suppression pool water level less than the above limit, restore the water level to within the limit within 1 hour or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.
- b. In OPERATIONAL CONDITION 4 or 5* with the suppression pool water level less than the above limit or drained and the above required conditions not satisfied, suspend CORE ALTERATIONS and all operations that have a potential for draining the reactor vessel and lock the reactor mode switch in the Shutdown position. Establish SECONDARY CONTAINMENT INTEGRITY within 8 hours.

*The suppression pool is not required to be OPERABLE provided that the reactor vessel head is removed, the cavity is flooded, the reactor cavity to steam dryer pool gate is open, and the water level in these upper containment pools is maintained within the limits of Specifications 3.9.8 and 3.9.9.

ELECTRICAL POWER SYSTEMS

DISTRIBUTION - SHUTDOWN

LIMITING CONDITION FOR OPERATION

3.8.3.2 As a minimum, the following power distribution system divisions shall be energized:

- a. For AC power distribution, Division I or Division II, and when the HPCS system is required to be OPERABLE, Division III and Division IV, with:
 1. Division I consisting of:
 - a) 4160-volt AC Bus 1A1.
 - b) 480-volt Unit Substations A and 1A.
 - c) 480-volt AC MCCs
 - 1) Aux. Bldg. MCCs 1A1, 1A2, 1A3, 1A4.
 - 2) SSW MCC 1A.
 - 3) D.G. MCC 1A.
 - 4) Control Bldg. MCCs E1, E2, and G
 - 5) Damper MCC A
 - d) 120-volt AC distribution panels in Auxiliary Building 480-volt MCC 1A1 and Control Building MCC E2.
 - e) 120-volt AC uninterruptible distribution panels energized from 1C71-S001A, which is fed from Control Building MCC E2 and from Auxiliary Building 480-volt MCC 1A1 through 125-volt DC MCC 1DC13E.
 2. Division II consisting of:
 - a) 4160-volt AC Bus 1B1.
 - b) 480-volt Unit Substations B and 1B.
 - c) 480-volt AC MCCs
 - 1) Aux. Bldg. MCCs 1B1, 1B2, 1B3, 1B4
 - 2) SSW MCC 1B
 - 3) D.G. MCC 1B
 - 4) Control Bldg. MCCs F1, F2, and H
 - 5) Damper MCC B
 - d) 120-volt AC distribution panels in Auxiliary Building 480-volt MCC 1B1 and Control Building MCC F2.
 - e) 120-volt AC uninterruptible distribution panels energized from 1C71-S001B, which is fed from Control Building MCC F2 and from Auxiliary Building 480-volt MCC 1B1 through 125-volt DC MCC 1DC14E.

REFUELING OPERATIONS

3/4.9.12 INCLINED FUEL TRANSFER SYSTEM

LIMITING CONDITION FOR OPERATION

3.9.12 The inclined fuel transfer system (IFTS) may be in operation provided that:

- a. The access doors* of all rooms through which the transfer system penetrates are closed and locked.
- b. All access door* interlocks are OPERABLE.
- c. The blocking valve located in the fuel building IFTS hydraulic power unit is OPERABLE.
- d. At least one IFTS carriage position indicator is OPERABLE at each carriage position and at least one liquid level sensor is OPERABLE.
- e. Any keylock switch that provides IFTS access control-transfer system lockout is OPERABLE.

APPLICABILITY: When the IFTS containment blank flange is removed.

ACTION:

With the requirements of the above specification not satisfied, suspend IFTS operation with the IFTS at either terminal point. The provisions of Specification 3.0.3 are not applicable.

SURVEILLANCE REQUIREMENTS

4.9.12.1 Within 1 hour prior to the startup and at least once per 12 hours during operation of the IFTS, verify that no personnel are in areas immediately adjacent to the IFTS and that all access doors* to rooms through which the IFTS penetrates are closed and locked.

4.9.12.2 Within 4 hours prior to the operation of IFTS and at least once per 12 hours thereafter, verify that at least one IFTS carriage position indicator is OPERABLE at each carriage position and at least one liquid level indicator is OPERABLE.

* Includes removable shields.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 73 TO FACILITY OPERATING LICENSE NO. NPF-62

ILLINOIS POWER COMPANY, ET AL.

CLINTON POWER STATION, UNIT NO. 1

DOCKET NO. 50-461

1.0 INTRODUCTION

By letter dated October 16, 1992, the Illinois Power Company (IP, the licensee) requested an amendment to Facility Operating License No. NPF-62 for the Clinton Power Station (CPS). The proposed amendment would make editorial changes to the Clinton Power Station Facility Operating License and Technical Specifications (TS) to correct typographical errors, provide clarification and remove provisions which are no longer applicable.

2.0 EVALUATION

The licensee is proposing the following changes to the CPS Facility Operating License and TS:

1. Delete item (b) from paragraph D of the CPS Facility Operating License and reletter the remaining items, as appropriate.

Item (b) provided exemptions from the requirements of General Design Criteria (GDC) 61 to permit the deferral of a portion of the Fuel Handling System preoperational testing until prior to off-loading fuel from the reactor vessel. Since the testing has been completed and the system declared operable, the staff finds the deletion of this license condition acceptable.

2. Revise TS 3.5.2, "ECCS - SHUTDOWN," item e.2 and TS 3.5.3, "SUPPRESSION POOL," item b.3, to delete the condition "equivalent to a level of 95%."

TS 3.5.2.e.2 and 3.5.3.b.3 identify the minimum required available volume in the Reactor Core Isolation Cooling (RCIC) system storage tank in the event the suppression pool level is less than the applicable limit during operational conditions 4 and 5. The TS currently identify a volume of "at least 125,000 available gallons of water, equivalent to a level of 95%." The available level instrumentation provides a tank level range of 0-30 feet and does not provide level indication in terms of percentage. The phrase "equivalent to a level of 95%" is not defined in terms of total tank volume or usable water volume and is not meaningful to the control room operators. The licensee contends that the requirement to maintain 125,000 available gallons of water is sufficient to determine system operability and, therefore, proposes to

delete the phrase in question. The staff concurs with the licensee and finds the proposed modification acceptable.

3. Revise TS 3.8.3.2, "DISTRIBUTION - SHUTDOWN," item a.3.d to delete the "*" attached to this item.

TS Limiting Condition For Operation (LCO) 3.8.3.2.a.3 identifies the Division III AC power distribution system equipment required to be operable during shutdown conditions. Footnote "*", which states "When handling irradiated fuel in the secondary containment," was intended to define the "*" which appears in the LCO Applicability statement. While TS 3.8.3.2.a.3.d is applicable during the operational condition defined by "*", it was not intended that the "*" be attached to item a.3.d. Since this change will not impact which equipment is required to be operable, and will eliminate the confusion generated by the "*" being inappropriately attached to the LCO, the staff finds this change acceptable.

4. Revise TS 3/4.9.12, "INCLINED FUEL TRANSFER SYSTEM," to delete footnote "**".

Delayed preoperational testing of the Fuel Handling System resulted in the addition of footnote "**" to TS 3/4.9.12 to permit an exception to the operability requirements for the blocking valve in the fuel building Inclined Fuel Transfer System hydraulic power unit and the liquid level indicator prior to off-loading irradiated fuel from the reactor vessel. Since the testing has been completed and the system was declared operable, the staff finds the deletion of the footnote from the referenced TS acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Illinois State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

This amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes surveillance requirements. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released off-site, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration and there has been no public comment on such finding (58 FR 16862). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: R. Laufer

Date: May 4, 1993