

October 18, 1993

Docket No. 50-461

Mr. Richard F. Phares
Director - Licensing
Clinton Power Station
Post Office Box 678
Mail Code V920
Clinton, Illinois 61727

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PDIII-2 r/f
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DPickett
DHagan
CGrimes
OPA
BClayton, RIII

Dear Mr. Phares:

SUBJECT: CORRECTION TO AMENDMENT (TAC NOS. M85815 AND M86270)

Amendment No. 83, issued September 8, 1993, contained an editorial error on page 3/4 6-4 of the technical specifications. The footnote labeled # incorrectly referred to containment penetration IMC-44 as opposed to containment penetration IMC-4. Page 3/4 6-4 is being reissued to correctly identify the containment penetration being temporarily exempted from the local leak rate testing requirements of Appendix J to 10 CFR 50.

In addition, our letter of September 27, 1993, which was to have corrected editorial errors associated with the above license amendment, incorrectly modified page 6 of the operating license. Page 6 is being reissued to clarify that item (a) of paragraph D is granted pursuant to 10 CFR 70.24 whereas items (b) through (e) are being granted pursuant to 10 CFR 50.12.

Sincerely,

Original Signed By:

Douglas V. Pickett, Senior Project Manager
Project Directorate III-2
Division of Reactor Projects - III/IV/V
Office of Nuclear Reactor Regulation

Enclosures:
Corrected pages

cc w/enclosures:
See next page

NRC FILE CENTER COPY

OFC	LA: PDIII-2	PM: PDIII-2	D: PDIII-2			
NAME	CMOORE	DPICKETT	JDYER			
DATE	10/15/93	10/15/93	10/18/93	1 / 93	1 / 93	1 / 93
COPY	YES/NO	YES/NO	YES/NO	YES/NO	YES/NO	YES/NO

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Mr. Richard F. Phares
Illinois Power Company

Clinton Power Station
Unit No. 1

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

ILLINOIS POWER COMPANY
SOYLAND POWER COOPERATIVE, INC.
DOCKET NO. 50-461
CLINTON POWER STATION, UNIT NO. 1
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 83
License No. NPF-62

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The applications for amendment by Illinois Power Company¹ (IP), and Soyland Power Cooperative, Inc. (the licensees) dated February 17, 1993, and April 16, 1993, comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license² is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-62 is hereby amended to read as follows:

¹Illinois Power Company is authorized to act as agent for Soyland Power Cooperative, Inc. and has exclusive responsibility and control over the physical construction, operation and maintenance of the facility.

²Pages 5 and 6 are attached, for convenience, for the composite license to reflect this change.

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 83, are hereby incorporated into this license. Illinois Power Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

- D. The facility requires exemptions from certain requirements of 10 CFR Part 50 and 10 CFR Part 70. These include: (a) an exemption from the requirements of 10 CFR 70.24 for the criticality alarm monitors around the fuel storage area; (b) an exemption from the requirement of paragraph III.D.2(b)(ii) of Appendix J, substituting the seal leakage test at Pa of paragraph III.D.2(b)(iii) for the entire airlock test at Pa of paragraph III.D.2(b)(ii) of Appendix J when no maintenance has been performed in the airlock that could affect its sealing capability (Section 6.2.6 of SSER 6); (c) an exemption from the requirement of paragraph III.C.3 of Appendix J, exempting the measured leakage rates from the main steam isolation valves from inclusion in the combined leak rate for the local leak rate tests (Section 6.2.6 of SSER 6); (d) an exemption from the requirements of paragraph III.B.3 of Appendix J, exempting leakage from the valve packing and the body-to-bonnet seal of valve 1E51-F374 associated with containment penetration 1MC-44 from inclusion in the combined leakage rate for penetrations and valves subject to Type B and C tests; and (e) an exemption from the requirement of paragraph III.D.1.(a) to conduct the third Type A test of each 10-year service period when the plant is shut down for the 10-year plant inservice inspections. The special circumstances regarding each exemption, except for Items (a), (d), and (e) above, are identified in the referenced section of the safety evaluation report and the supplements thereto.

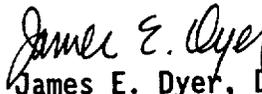
An exemption was previously granted pursuant to 10 CFR 70.24. The exemption was granted with NRC materials license No. SNM-1886, issued November 27, 1985, and relieved IP from the requirement of having a criticality alarm system. IP is hereby exempted from the criticality alarm system provision of 10 CFR 70.24 so far as this section applies to the storage of fuel assemblies held under this license.

The special circumstances regarding the exemption identified in Item (d) above are identified in the safety evaluation accompanying Amendment No. 62 to this license. The special circumstances regarding the exemption identified in Item (e) above are identified in the safety evaluation accompanying Amendment No. 83 to this license.

These exemptions are authorized by law, will not present an undue risk to the public health and safety, and are consistent with the common defense and security. The exemptions in items (b) through (e), above are granted pursuant to 10 CFR 50.12. With these exemptions, the facility will operate, to the extent authorized herein, in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission.

3. This license amendment is effective as of its date of issuance, to be implemented during the fourth refueling outage.

FOR THE NUCLEAR REGULATORY COMMISSION



James E. Dyer, Director
Project Directorate III-2
Division of Reactor Projects - III/IV/V
Office of Nuclear Reactor Regulation

Attachment:

1. License pages 5 and 6
2. Changes to the Technical Specifications

Date of Issuance: September 8, 1993

An exemption was previously granted pursuant to 10 CFR 70.24. The exemption was granted with NRC materials license No. SNM-1886, issued November 27, 1985, and relieved IP from the requirement of having a criticality alarm system. IP is hereby exempted from the criticality alarm system provision of 10 CFR 70.24 so far as this section applies to the storage of fuel assemblies held under this license.

The special circumstances regarding the exemption identified in Item (d) above are identified in the safety evaluation accompanying Amendment No. 62 to this license. The special circumstances regarding the exemption identified in Item (e) above are identified in the safety evaluation accompanying Amendment No. 83 to this license.

These exemptions are authorized by law, will not present an undue risk to the public health and safety, and are consistent with the common defense and security. The exemptions in items (b) through (e) above are granted pursuant to 10 CFR 50.12. With these exemptions, the facility will operate, to the extent authorized herein, in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission.

- E. The licensees shall fully implement and maintain in effect all provisions of the Commission-approved physical security plan, guard training and qualification, and safeguards contingency plans including amendments made pursuant to provisions of the Miscellaneous Amendments and Search Requirements revisions to 10 CFR 73.55 (51 FR 27817 and 27822) and to the authority of 10 CFR 50.90 and 10 CFR 50.54(p). The plans, which contain Safeguards Information protected under 10 CFR 73.21, are entitled: "Clinton Power Station Physical Security Plan," with revisions submitted through May 27, 1993; "Clinton Power Station Training and Qualification Plan," with revisions submitted through May 27, 1993; and "Clinton Power Station Safeguards Contingency Plan," with revisions submitted through May 27, 1993. Changes made in accordance with 10 CFR 73.55 shall be implemented in accordance with the schedule set forth therein.
- F. IP shall implement and maintain in effect all provisions of the approved fire protection program as described in the Final Safety Analysis Report as amended, for the Clinton Power Station, Unit No. 1, and as approved in the Safety Evaluation Report (NUREG-0853) dated February 1982 and Supplement Nos. 1 thru 8 thereto subject to the following provision:
 - IP may make changes to the approved fire protection program without prior approval of the Commission only if those changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire.
- G. Except as otherwise provided in the Technical Specifications or Environmental Protection Plan, IP shall report any violations of the requirements contained in Section 2.C of this license in the following manner: initial notification shall be made within 24 hours to the NRC Operations Center via the Emergency Notification System with written followup within thirty days in accordance with the procedures described in 10 CFR 50.73(b), (c), and (e).

CONTAINMENT SYSTEMS

PRIMARY CONTAINMENT LEAKAGE

SURVEILLANCE REQUIREMENTS (Continued)

4.6.1.2 (Continued)

2. Has duration sufficient to establish accurately the change in leakage rate between the Type A test and the supplemental test.
 3. Requires the quantity of gas injected into the primary containment or bled from the primary containment during the supplemental test to be between 0.75 La and 1.25 La.
- d. Type B and C tests shall be conducted^{***},[#] with gas at Pa, 9.0 psig, at intervals no greater than 24 months except for tests involving:
1. Air locks,
 2. Main steam line isolation valves,
 3. Penetrations using continuous leakage monitoring systems,
 4. All containment isolation valves in hydrostatically tested lines which penetrate the primary containment, and
 5. Purge supply and exhaust isolation valves with resilient material seals.
- e. Air locks shall be tested and demonstrated OPERABLE per Surveillance Requirement 4.6.1.3.
- f. Main steam line isolation valves shall be leak tested with gas at Pa, 9.0 psig, at least once per 18 months.
- g. Type B tests for penetrations employing a continuous leakage monitoring system shall be conducted at Pa, 9.0 psig, at every other reactor shutdown for refueling, but in no case at intervals greater than 3 years.
- h. All containment isolation valves in hydrostatically tested lines which penetrate the primary containment shall be leak tested at 1.10 Pa, 9.9 psig, at least once per 18 months.
- i. Purge supply and exhaust isolation valves with resilient material seals shall be tested and demonstrated OPERABLE per Surveillance Requirement 4.6.1.8.3.
- j. The provisions of Specification 4.0.2 are not applicable to Specifications 4.6.1.2.a, 4.6.1.2.b, 4.6.1.2.d, and 4.6.1.2.g.

^{***}Except as provided in NRC-approved exemption to Appendix J to 10 CFR 50 for containment penetration IMC-44.

[#]The leakage rate for containment penetration IMC-4 is not required to be determined until startup from the fifth refueling outage in accordance with an approved exemption to Appendix J of 10 CFR 50.