

September 25, 1990

Docket No. 50-461

Mr. Frank A. Spangenberg  
Licensing and Safety  
Clinton Power Station  
P. O. Box 678  
Mail Code V920  
Clinton, Illinois 61727

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JCalvo	ACRS(10)
GPA/PA	OC/LFMB
PDIII-2 Gray	

Dear Mr. Spangenberg:

SUBJECT: AMENDMENT NO. 48 TO FACILITY OPERATING LICENSE NO. NPF-62  
(TAC NO. 73800)

The Commission has issued the enclosed Amendment No.48 to Facility Operating License No. NPF-62 for the Clinton Power Station, Unit No. 1. This amendment is in response to your application dated February 5, 1988.

This amendment clarifies the divisional arrangement for the reactor vessel water level and drywell pressure instruments listed on Technical Specification Tables 3.3.2-1 and 3.3.3-1, and clarifies the combinational logic scheme for the Reactor Vessel Water Level-Low, Low Level 2 and Drywell Pressure-High channels on Tables 3.3.2-1 and 3.3.3-1.

A copy of the Safety Evaluation is also enclosed. The notice of issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

Original signed by

John B. Hickman, Project Manager  
Project Directorate III-3  
Division of Reactor Projects - III,  
IV, V and Special Projects  
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 48 to License No. NPF-62
2. Safety Evaluation

cc w/enclosures:  
See next page

DOCUMENT NAME: 73800 AMD

Office: LA/PDIII-3  
Surname: PKreutzer  
Date: 9/14/90

PM/PDIII-3  
JHickman:rc  
9/15/90

PD/PDIII-3  
JHannon  
9/17/90

OGC  
Bm/B  
9/13/90

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PD/PDIII-3  
JHannon  
9/17/90

OGC  
Bm's  
9/13/90

Mr. Frank A. Spangenberg  
Illinois Power Company

Clinton Power Station  
Unit 1

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Chicago, Illinois 60601



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

ILLINOIS POWER COMPANY, ET AL.

DOCKET NO. 50-461

CLINTON POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 48  
License No. NPF-62

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Illinois Power Company\* (IP), and Soyland Power Cooperative, Inc. (the licensees) dated February 5, 1988, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-62 is hereby amended to read as follows:

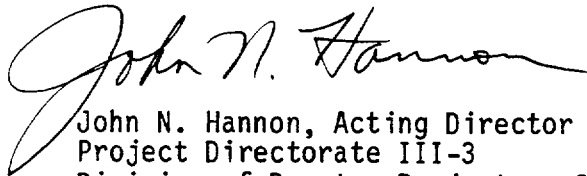
\*Illinois Power Company is authorized to act as agent for Soyland Power Cooperative, Inc. and has exclusive responsibility and control over the physical construction, operation and maintenance of the facility.

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 48 , are hereby incorporated into this license. Illinois Power Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

A handwritten signature in cursive script, reading "John N. Hannon".

John N. Hannon, Acting Director  
Project Directorate III-3  
Division of Reactor Projects - III,  
IV, V and Special Projects  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: September 25, 1990

ATTACHMENT TO LICENSE AMENDMENT NO. 48

FACILITY OPERATING LICENSE NO. NPF-62

DOCKET NO. 50-461

Replace the following pages of the Appendix "A" Technical Specifications with the attached pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change.

<u>Remove</u>	<u>Insert</u>
3/4 3-13	3/4 3-13
3/4 3-18	3/4 3-18
-	3/4 3-18a
3/4 3-36	3/4 3-36
3/4 3-37	3/4 3-37

TABLE 3.3.2-1  
CRVICS INSTRUMENTATION

<u>TRIP FUNCTION</u>	<u>ISOLATION SIGNAL ††</u>	<u>MINIMUM OPERABLE CHANNELS PER TRIP SYSTEM</u>	<u>APPLICABLE OPERATIONAL CONDITION</u>	<u>ACTION</u>
1. <u>PRIMARY AND SECONDARY CONTAINMENT ISOLATION</u>				
a. Reactor Vessel Water Level-Low Low, Level 2	B(b)(f)	2(a)	1, 2, 3 #	20 25
b. Reactor Vessel Water Level-Low Low, Level 2 (ECCS Div. I and II)	B	2(a)	1, 2, 3	29
c. Reactor Vessel Water Level-Low, Low, Level 2 (HPCS-NSPS Div. III and IV)	B	2(a)(m)	1, 2, 3	29
d. Drywell Pressure - High	L(b)(f)	2(a)	1, 2, 3	20
e. Drywell Pressure - High (ECCS Div. I and II)	L	2(a)	1, 2, 3	29
f. Drywell Pressure - High (HPCS-NSPS Div. III and IV)	L	2(a)(n)	1, 2, 3	29
g. Containment Building Fuel Transfer Pool Ventilation Plenum Radiation - High	Z(b)(f)	2(a)	##	25
h. Containment Building Exhaust Radiation - High	M(b)(f)	2(a)	1, 2, 3 #	29 25
i. Containment Building Continuous Containment Purge (CCP) Exhaust Radiation - High	5(b)(f)	2(a)	1, 2, 3 #	29 25
j. Reactor Vessel Water Level-Low Low Low, Level 1	U	2(k)	1, 2, 3 #	29 25
k. Containment Pressure-High	P	1(k)(1)	1, 2, 3 #	29 25

TABLE 3.3.2-1 (Continued)

CRVICS INSTRUMENTATION

<u>TRIP FUNCTION</u>	<u>ISOLATION SIGNAL ††</u>	<u>MINIMUM OPERABLE CHANNELS PER TRIP SYSTEM</u>	<u>APPLICABLE OPERATIONAL CONDITION</u>	<u>ACTION</u>
l. Main Steam Line Radiation - High	C	2(a)	1, 2, 3	23
m. Fuel Building Exhaust Radiation - High	(b)(f)(j)	2(a)	1, 2, 3 #	25 25
n. Manual Initiation	R(b)(f)	1	1, 2, 3 #	26 25
2. <u>MAIN STEAM LINE ISOLATION †</u>	<u>ISOLATION SIGNAL</u>	<u>APPLICABLE TABLE NOTES</u>	<u>APPLICABLE OPERATIONAL CONDITIONS</u>	<u>ACTION</u>
a. Reactor Vessel Water Level-Low Low Low, Level 1	U	NA	1, 2, 3	20
b. Main Steam Line Radiation - High	C	d	1, 2, 3	23
c. Main Steam Line Pressure - Low	H	NA	1	23
d. Main Steam Line Flow - High	D	NA	1, 2, 3	23
e. Condenser Vacuum - Low	J	NA	1, 2**, 3**	23
f. Main Steam Line Tunnel Temp. - High	E	NA	1, 2, 3	23
g. Main Steam Line Tunnel Δ Temp. - High	F	NA	1, 2, 3	23
h. Main Steam Line Turbine Bldg. Temp. - High	G	P	1, 2, 3	23
i. Manual Initiation	R	NA	1, 2, 3	22



TABLE 3.3.2-1 (Continued)

CRVICS INSTRUMENTATION

<u>TRIP FUNCTION</u>	<u>ISOLATION SIGNAL ††</u>	<u>MINIMUM OPERABLE CHANNELS PER TRIP SYSTEM</u>	<u>APPLICABLE OPERATIONAL CONDITION</u>	<u>ACTION</u>
5. <u>RHR SYSTEM ISOLATION</u>				
a. RHR Heat Exchanger A, B Ambient Temperature - High	T	1/room <sup>(k)</sup>	1, 2, 3	28
b. RHR Heat Exchanger A, B ΔTemp. - High	S	1/room <sup>(k)</sup>	1, 2, 3	28
c. Reactor Vessel Water Level - Low, Level 3	A	2 <sup>(a)</sup>	1, 2, 3	28
d. Reactor Vessel Water Level - Low Low Low, Level 1	U	2 <sup>(k)</sup>	1, 2, 3	28
e. Reactor Vessel (RHR Cut-in Permissive) Pressure - High	X	2 <sup>(a)</sup>	1, 2, 3	28
f. Drywell Pressure - High				
1) RHR Test Lines	L	2 <sup>(k)</sup>	1, 2, 3	28
2) Fuel Pool Cooling	L	2 <sup>(a)</sup>	1, 2, 3	28
g. Manual Initiation	R	1	1, 2, 3	26

TABLE 3.3.2-1 (Continued)  
CRVICS INSTRUMENTATION

TABLE NOTATIONS

- # When handling irradiated fuel in the primary or secondary containment and during CORE ALTERATIONS and operations with a potential for draining the reactor vessel.
- ## When handling irradiated fuel in the primary containment (building) and during CORE ALTERATIONS and operations with a potential for draining the reactor vessel.
- \* With any control rod withdrawn. Not applicable to control rods removed per Specification 3.9.10.1 or 3.9.10.2.
- \*\* When any turbine stop valve is greater than 95% open or the reactor mode switch is in the run position.
- † Main steam line isolation trip functions have 2-out-of-4 isolation logic except for the main steam line flow - high trip function which has 2-out-of-4 isolation logic for each main steam line.
- †† See Specification 3.6.4 Table 3.6.4-1 for valves which are actuated by these isolation signals.
- (a) A channel may be placed in an inoperable status for up to 2 hours for required surveillance without placing the trip system in the tripped condition provided at least one other OPERABLE channel in the same trip system is monitoring that parameter.
- (b) Also actuates the standby gas treatment system.
- (c) Deleted
- (d) Also trips and isolates the mechanical vacuum pumps.
- (e) Isolates RWCU valves 1G33-F001 and 1G33-F004 only.
- (f) Also actuates secondary containment ventilation isolation dampers per Table 3.6.6.2-1.
- (g) Manual Switch closes RWCU system inboard isolation valves F001, F028, F053, F040 and outboard isolation valves F004, F039, F034 and F054.
- (h) Vacuum breaker isolation valves require RCIC system steam supply pressure low coincident with drywell pressure high for isolation of vacuum breaker isolation valves.
- (i) A single manual isolation switch isolates outboard steam supply line isolation valve (F064) and the RCIC pump suction from suppression pool valve (F031) only following a manual or automatic (Reactor Vessel Water Level 2) RCIC system initiation.
- (j) Only actuates secondary containment ventilation isolation dampers per Table 3.6.6.2-1. Note †† is not applicable to this Trip Function.
- (k) A channel may be placed in an inoperable status for up to 2 hours for required surveillance without placing the trip system in the trip condition provided that the redundant trip system is OPERABLE and monitoring that parameter.
- (l) Not required to be OPERABLE when valves 1VR002A,B and 1VQ006A,B are sealed closed in accordance with Specification 3.6.4.

TABLE 3.3.2-1 (Continued)  
CRVICS INSTRUMENTATION  
TABLE NOTATIONS

- (m) Four reactor vessel water level trip channels are logically combined in a one-out-of-two-twice configuration. For the purposes of the associated ACTION, each one-out-of-two logic is defined as a separate trip system.
- (n) Four drywell pressure trip channels are logically combined in a one-out-of-two-twice configuration. For the purposes of the associated ACTION, each one-out-of-two logic is defined as a separate trip system.
- (p) Each channel consists of five temperature modules and their associated sensors. A channel is OPERABLE if and only if five temperature modules and their associated sensors are OPERABLE.

TABLE 3.3.3-1 (Continued)

EMERGENCY CORE COOLING SYSTEM ACTUATION INSTRUMENTATION

<u>TRIP FUNCTION</u>	<u>MINIMUM OPERABLE CHANNELS PER TRIP FUNCTION</u>	<u>APPLICABLE OPERATIONAL CONDITIONS</u>	<u>ACTION</u>
<b>B. <u>DIVISION II TRIP SYSTEM</u></b>			
<b>1. <u>RHR B &amp; C (LPCI MODE)</u></b>			
a. Reactor Vessel Water Level - Low Low Low, Level 1	2(b)(a)	1, 2, 3, 4*, 5*	30
b. Drywell Pressure - High	2(b)(a)	1, 2, 3	30
c. Reactor Vessel Pressure-Low (LPCI Injection Valve Permissive)	4(a)	1, 2, 3 4*, 5*	32 33
d. LPCI Pump (B) Start Time Delay Logic Card	1	1, 2, 3, 4*, 5*	32
e. LPCI Pump (B) Discharge Flow - Low <sup>##</sup>	1	1, 2, 3, 4*, 5*	40
f. LPCI Pump (C) Discharge Flow - Low <sup>##</sup>	1	1, 2, 3, 4*, 5*	40
g. Manual Initiation	1(b)	1, 2, 3, 4*, 5*	35
<b>2. <u>AUTOMATIC DEPRESSURIZATION SYSTEM TRIP SYSTEM "2"<sup>#</sup></u> <u>ADS LOGIC "B" AND "F"</u></b>			
a. Reactor Vessel Water Level - Low Low Low, Level 1	2(a)	1, 2, 3	30
b. Drywell Pressure - High	2(a)	1, 2, 3	30
c. ADS Timer	1	1, 2, 3	32
d. Reactor Vessel Water Level - Low, Level 3 (Permissive)	1	1, 2, 3	32
e. LPCI Pump (B and C) Discharge Pressure - High (Permissive)	2/pump <sup>(a)</sup>	1, 2, 3	32
f. ADS Drywell Pressure Bypass Timer	2	1, 2, 3	30
g. Manual Inhibit ADS Switch	1	1, 2, 3	35
h. Manual Initiation	2	1, 2, 3	35

TABLE 3.3.3-1 (Continued)

EMERGENCY CORE COOLING SYSTEM ACTUATION INSTRUMENTATION

<u>TRIP FUNCTION</u>		<u>MINIMUM OPERABLE CHANNELS PER TRIP FUNCTION</u>	<u>APPLICABLE OPERATIONAL CONDITIONS</u>	<u>ACTION</u>	
C. <u>DIVISION III TRIP SYSTEM</u>					
1. <u>HPCS SYSTEM</u>					
a. Reactor Vessel Water Level-Low, Low, Level 2		4(b)(a)(e)(g)	1, 2, 3, 4*, 5*	36	
b. Drywell Pressure - High †		4(b)(a)(f)(g)	1, 2, 3	36	
c. Reactor Vessel Water Level-High, Level 8		2(c)(g)	1, 2, 3, 4*, 5*	32	
d. RCIC Storage Tank Level-Low		2(d)(a)	1, 2, 3, 4*, 5*	37	
e. Suppression Pool Water Level-High##		2(d)(a)	1, 2, 3, 4*, 5*	37	
f. HPCS Pump Discharge Pressure-High##		1	1, 2, 3, 4*, 5*	40	
g. HPCS System Flow Rate-Low##		1	1, 2, 3, 4*, 5*	40	
h. Manual Initiation †		1	1, 2, 3, 4*, 5*	35	
	<u>TOTAL NO. OF CHANNELS</u>	<u>CHANNELS TO TRIP</u>	<u>MINIMUM OPERABLE CHANNELS</u>	<u>APPLICABLE OPERATIONAL CONDITIONS</u>	<u>ACTION</u>
D. <u>LOSS OF POWER</u>					
1. 4.16 kV Emergency Bus Undervoltage (Loss of Voltage)					
a. Divisions I & II	2/Division	2/Division	2/Division	1, 2, 3, 4**, 5**	38
b. Division III	4	2	4	1, 2, 3, 4**, 5**	38
2. 4.16 kV Emergency Bus Undervoltage (Degraded Voltage)					
a. Divisions I & II	2/Division	2/Division	2/Division	1, 2, 3, 4**, 5**	39
b. Division III ††	3	3	3	1, 2, 3, 4**, 5**	39

TABLE 3.3.3-1 (Continued)

EMERGENCY CORE COOLING SYSTEM ACTUATION INSTRUMENTATION

TABLE NOTATIONS

- (a) A channel may be placed in an inoperable status for up to 2 hours during periods of required surveillance without placing the trip system in the tripped condition provided at least one other OPERABLE channel in the same trip system is monitoring that parameter.
- (b) Also actuates the associated division diesel generator.
- (c) Provides signal to close HPCS pump injection valve only.
- (d) Provides signal to HPCS pump suction valves only.
- (e) Four reactor vessel water level trip channels are logically combined in a one-out-of-two-twice configuration. For the purposes of the associated ACTION, each one-out-of-two logic is defined as a separate trip system.
- (f) Four drywell pressure trip channels are logically combined in a one-out-of-two-twice configuration. For the purposes of the associated ACTION, each one-out-of-two logic is defined as a separate trip system.
- (g) One half of these trip channels is associated with Nuclear Systems Protection System (NSPS) Division III; the other half is associated with NSPS Division IV.
- \* When the system is required to be OPERABLE per Specification 3.5.2 or 3.5.3.
- \*\* Required when ESF equipment is required to be OPERABLE.
- # Not required to be OPERABLE when reactor steam dome pressure is  $\leq$  100 psig.
- ## These Trip Functions are not required for ECCS actuation.
- † The HPCS initiation functions of the Drywell Pressure - High and Manual Initiation are not required to be OPERABLE with indicated reactor vessel water level on the wide range instrument greater than the Level-8 setpoint coincident with the reactor steam dome pressure less than 600 psig.
- †† One relay with three inputs in 3 out of 3 logic.

TABLE 3.3.3-1 (Continued)

EMERGENCY CORE COOLING SYSTEM ACTUATION INSTRUMENTATION

ACTION

- ACTION 30 - With the number of OPERABLE channels less than required by the Minimum OPERABLE Channels per Trip Function requirement:
- With one channel inoperable, place the inoperable channel in the tripped condition within 1 hour\* or declare the associated system inoperable.
  - With more than one channel inoperable, declare the associated system inoperable.
- ACTION 31 - Deleted.
- ACTION 32 - With the number of OPERABLE channels less than required by the Minimum OPERABLE Channels per Trip Function requirement, declare the associated ADS trip system or ECCS inoperable.
- ACTION 33 - With the number of OPERABLE channels less than the Minimum OPERABLE Channels per Trip Function requirement, place the inoperable channel(s) in the tripped condition within 1 hour.
- ACTION 34 - Deleted.
- ACTION 35 - With the number of OPERABLE channels less than required by the Minimum OPERABLE Channels per Trip Function requirement, restore the inoperable channel to OPERABLE status within 8 hours or declare the associated ADS valve or ECCS inoperable.
- ACTION 36 - With the number of OPERABLE channels less than required by the Minimum OPERABLE Channels per Trip Function requirement:
- For one trip system, place that trip system in the tripped condition within one hour\* or declare the HPCS system inoperable.
  - For both trip systems, declare the HPCS system inoperable.
- ACTION 37 - With the number of OPERABLE channels less than required by the Minimum OPERABLE Channels per Trip Function requirement, place at least one inoperable channel in the tripped condition within 1 hour\* or declare the HPCS system inoperable.
- ACTION 38 - With the number of OPERABLE channels less than the Total Number of Channels, declare the associated emergency diesel generator inoperable and take the ACTION required by Specifications 3.8.1.1 or 3.8.1.2, as appropriate.
- ACTION 39 - With the number of OPERABLE channels one less than the Total Number of Channels, place the inoperable channel in the tripped condition within 1 hour\*; operation may then continue until performance of the next required CHANNEL FUNCTIONAL TEST.
- ACTION 40 - With the number of OPERABLE channels less than required by the Minimum OPERABLE Channels per Trip Function requirement, place the inoperable channel in the tripped condition within one hour. Restore the inoperable channel to OPERABLE status within 7 days or declare the associated system inoperable.

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\*The provisions of Specification 3.0.4 are not applicable.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 48 TO FACILITY OPERATING LICENSE NO. NPF-62  
ILLINOIS POWER COMPANY, ET AL.  
CLINTON POWER STATION, UNIT NO. 1  
DOCKET NO. 50-461

1.0 INTRODUCTION

By application for amendment dated February 5, 1988, Illinois Power Company requested a revision to the Technical Specification of operating license NPF-62 for the Clinton Power Station. The proposed change revises Tables 3.3.2-1 and 3.3.3-1 to more accurately reflect the divisional assignments and the logic arrangements of the reactor water level and high drywell pressure instrumentation listed under Items 1.c and 1.f of Table 3.3.2-1 and Items C.1.a, C.1.b and C.1.c of Table 3.3.3-1. This evaluation encompasses the referenced changes as applicable to Sections 3.3.2-1 and 3.3.3-1 of the Clinton Power Station Technical Specifications.

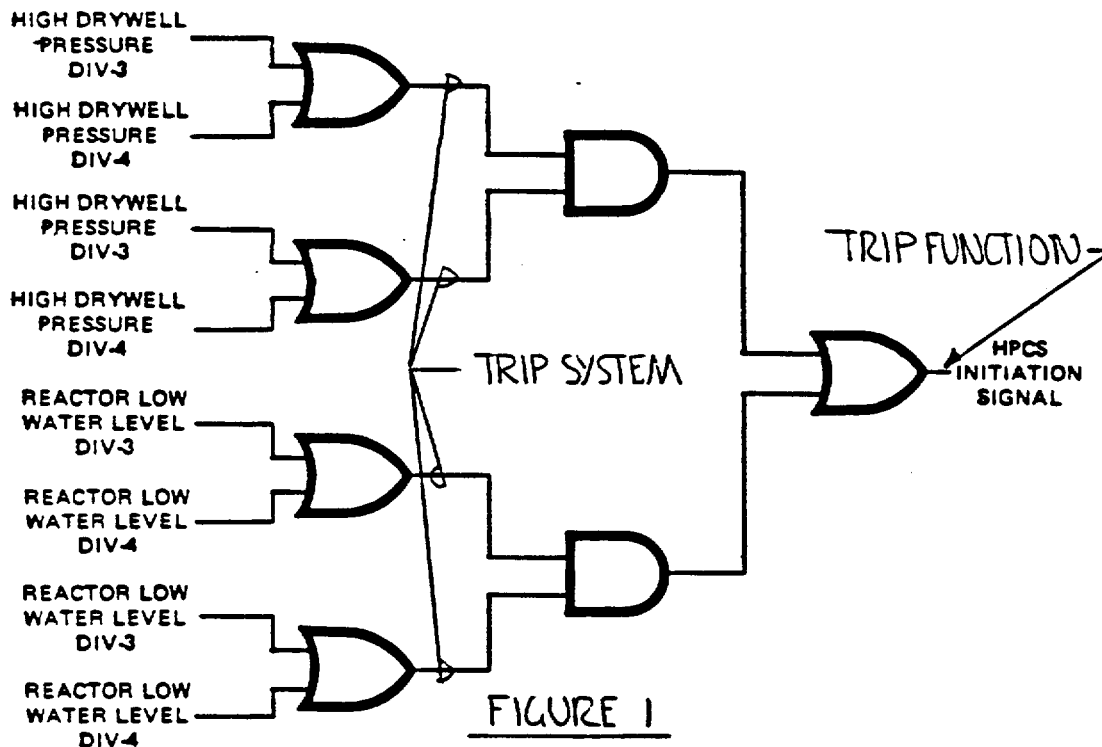
2.0 EVALUATION

The trip function affected by the proposed change provides an isolation trip signal to the Containment Reactor Vessel Isolation Control System (CRVICS) instrumentation and also provides an Emergency Core Cooling System (ECCS) actuation trip. The Divisions 3 and 4 instrumentation associated with the proposed change provides signals to the High Pressure Core Spray System (HPCS). Although the HPCS is generally considered a Division 3 System, the high drywell pressure and reactor water level instrumentation inputs to the HPCS initiation signal are both Divisions 3 and 4. The change by the licensee revises the divisional assignment to read, "(HPCS-NSPS Division III and IV)" for Items 1.c and 1.f on Table 3.3.2-1. Table 3.3.3-1 is modified in a similar manner with the addition of Note "g" which indicates that the instrumentation for HPCS is associated with two divisions. The staff review of the divisional assignments referenced in the proposed change agrees with the logic configuration referenced by the licensee and is found to be acceptable.

The purpose of the second change of this amendment is to clarify the one-out-of-two taken twice logic arrangement and the Minimum Operable Channels Per Trip System requirements of the reactor water level and high drywell pressure channels inputs to the HPCS and CRVICS. These changes clarify the trip system boundary for the four reactor water level and high drywell pressure inputs to the HPCS (CRVICS) as shown in Figure 1. The



change proposed by the licensee involves adding notes "m" and "n" to Table 3.3.2.1 and notes "e" and "f" to Table 3.3.3-1. These notes ensure that the corresponding action statements are implemented properly when one or more channels are declared inoperable. The staff reviewed the proposal and determined that defining the trip system as shown on the licensee's submittal is acceptable.



### 3.0 ENVIRONMENTAL CONSIDERATION

This amendment involves a change to a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 or a change to a surveillance requirement. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental

impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

#### 4.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner; and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: C. Doult, SICB/DST

Dated: September 25, 1990