Docket No. 50-461

Mr. Frank A. Spangenberg Manager - Licensing and Safety Clinton Power Station P. O. Box 678 Mail Code V920 Clinton, Illinois 61727 DISTRIBUTION: Docket File NRC & Local PDRs PDIII-2 p/f J. Roe J. Zwolinski J. Dyer C. Moore D. Pickett D. Hagan G. Hill (2) W. Jones C. Grimes ACRS (10) OPA OC/LFDCB B. Clayton RIII OGC

Dear Mr. Spangenberg:

SUBJECT: ISSUANCE OF AMENDMENT (TAC NO. M85234)

The U. S. Nuclear Regulatory Commission (Commission) has issued the enclosed Amendment No. 70 to Facility Operating License No. NPF-62 for the Clinton Power Station, Unit No. 1. The amendment is in response to your application dated December 15, 1992 (U-602074).

The amendment adds a requirement to Technical Specification (TS) 3/4.4.4, "Chemistry," to perform an engineering evaluation prior to plant restart of the impact on the reactor coolant system of chemistry parameters exceeding their limit for specified time periods during plant shutdown conditions.

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's next biweekly <u>Federal Register</u> notice.

Sincerely,

Original signed by:

Douglas V. Pickett, Project Manager Project Directorate III-2 Division of Reactor Projects - III/IV/V Office of Nuclear Reactor Regulation

Enclosures:

- 1. Amendment No. 70 to NPF-62
- 2. Safety Evaluation

cc w/enclosures:
see next page

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Mr. Frank A. Spangenberg Illinois Power Company

cc:

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UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

ILLINOIS POWER COMPANY

SOYLAND POWER COOPERATIVE, INC.

DOCKET NO. 50-461

CLINTON POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 70 License No. NPF-62

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Illinois Power Company* (IP), and Soyland Power Cooperative, Inc. (the licensees) dated December 15, 1992, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
- 2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-62 is hereby amended to read as follows:

*Illinois Power Company is authorized to act as agent for Soyland Power Cooperative, Inc. and has exclusive responsibility and control over the physical construction, operation and maintenance of the facility.



(2) <u>Technical Specifications and Environmental Protection Plan</u>

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 70, are hereby incorporated into this license. Illinois Power Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

James E. Cyer

James E. Dyer, Director Project Directorate III-2 Division of Reactor Projects - III/IV/V Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance: March 29, 1993

ATTACHMENT TO LICENSE AMENDMENT NO. 70

FACILITY OPERATING LICENSE NO. NPF-62

DOCKET NO. 50-461

Replace the following page of the Appendix "A" Technical Specifications with the attached page. The revised page is identified by amendment number and contains vertical lines indicating the area of change. The corresponding overleaf page, as indicated by an asterisk, is provided to maintain document completeness.

<u>Remove Pages</u>	<u>Insert Pages</u>		
*3/4 4-15	*3/4 4-15		
3/4 4-16	3/4 4-16		

TABLE 3.4.3.2-1

REACTOR COOLANT SYSTEM PRESSURE ISOLATION VALVES

VALVE NUMBER

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<u>SYSTEM</u>

1E12F041A 1E12F041B 1E12F041C 1E12F042A 1E12F042B 1E12F042C 1E12F023 1E21F005 1E21F006 1E22F004 1E22F005 1E51F066 1E51F013 1E12F008 1E12F009 1E12F053A 1E12F053B 1E12F053B	LPCI from RHRA Testable Chk LPCI from RHRB Testable Chk LPCI from RHRC Testable Chk LPCI from RHRA Shutoff LPCI from RHRB Shutoff LPCI from RHRC Shutoff RHR B Supp to Rx Head Spray LPCS Inj Isol LPCS Inj Testable Chk HPCS Inj Isol HPCS Testable Chk RCIC Pmp Disch to Rx Testable Check RCIC Pmp Disch to Rx Otbd Isol RHR Shutdown Cooling Supply RHR Shutdown Cooling Return RHR Shutdown Cooling Return RHR Shutdown Cooling Return
1E12F050A 1E12F050B	RHR Shutdown Cooling Return RHR Shutdown Cooling Return

3/4 4-15

REACTOR COOLANT SYSTEM

3/4.4.4 CHEMISTRY

LIMITING CONDITION FOR OPERATION

3.4.4 The chemistry of the reactor coolant system shall be maintained within the limits specified in Table 3.4.4-1.

APPLICABILITY: At all times.

ACTION:

- a. In OPERATIONAL CONDITION 1:
 - 1. With the conductivity, chloride concentration, or pH exceeding the limit specified in Table 3.4.4-1 for less than 72 hours during one continuous time interval and, for conductivity and chloride concentration, for less than 336 hours per year, but with the conductivity less than 10 μ mho/cm at 25°C and with the chloride concentration less than 0.5 ppm, this need not be reported to the Commission and the provisions of Specification 3.0.4 are not applicable.
 - 2. With the conductivity, chloride concentration, or pH exceeding the limit specified in Table 3.4.4-1 for more than 72 hours during one continuous time interval or with the conductivity and chloride concentration exceeding the limit specified in Table 3.4.4-1, for more than 336 hours per year, be in at least STARTUP within the next 6 hours.
 - 3. With the conductivity exceeding 10 μ mho/cm at 25°C or chloride concentration exceeding 0.5 ppm, be in at least HOT SHUTDOWN within 12 hours and in COLD SHUTDOWN within the next 24 hours.
- b. In OPERATIONAL CONDITIONS 2 and 3 with the conductivity, chloride concentration or pH exceeding the limit specified in Table 3.4.4-1 for more than 48 hours during one continuous time interval, be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.
- c. At all other times:
 - 1. With the:
 - a. Conductivity or pH exceeding the limit specified in Table
 3.4.4-1, restore the conductivity and pH to within the limit within 72 hours, or
 - b. Chloride concentration exceeding the limit specified in Table
 3.4.4-1 restore the chloride concentration to within the limit within 24 hours, or

perform an engineering evaluation to determine the effects of the out-of-limit condition on the structural integrity of the reactor coolant system. Determine that the structural integrity of the reactor coolant system remains acceptable for continued operation prior to proceeding to OPERATIONAL CONDITION 3.

UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555



SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 70 TO FACILITY OPERATING LICENSE NO. NPF-62

ILLINOIS POWER COMPANY

SOYLAND POWER COOPERATIVE, INC.

CLINTON POWER STATION, UNIT NO. 1

DOCKET NO. 50-461

1.0 INTRODUCTION

The Standard Technical Specifications (STS) developed for boiling water reactors (BWR) describes reactor coolant system chemistry limits for conductivity, chloride concentration, and pH for each Operational Condition. The purpose of this specification is to prevent damage to materials in contact with the reactor coolant system and to ensure that the structural integrity of the reactor coolant system is maintained. Action statements are provided to ensure that appropriate actions are taken when a chemistry parameter exceeds its specified limit. When chemistry parameters exceed limits during Operational Conditions 1, 2, or 3, shutdown actions are required. Similarly, when such limits are exceeded during Operational Conditions 4 or 5, an engineering evaluation is required to ensure structural integrity prior to plant restart.

Due to a formatting error when developing the Clinton Power Station (CPS) Technical Specifications (TS), TS 3.4.4 <u>ACTION</u> c.1.a does not specify any action if conductivity or pH limits are exceeded during Operational Conditions 4 or 5. While <u>ACTION</u> c.1.b permits an engineering evaluation to verify structural integrity of the reactor coolant system if chloride concentration exceeds prescribed limits, <u>ACTION</u> c.1.a does not address similar actions.

By letter dated December 15, 1992, the Illinois Power Company (IP, the licensee), requested an amendment to Facility Operating License No. NPF-62 for the CPS. The proposed amendment would add a requirement to perform an engineering evaluation prior to plant restart of the impact on the reactor coolant system structural integrity if the reactor coolant system conductivity or pH exceeded their limit for specified time periods during plant shutdown conditions.

2.0 EVALUATION

A comparison of the CPS TSs with recently licensed BWR facilities identifies an inconsistency regarding specification 3/4.4.4, "Chemistry." TS 3/4.4.4 identifies reactor coolant system limits for chloride concentration,



conductivity, and pH for the different Operational Conditions. The inconsistency involves actions to be taken during Operational Conditions 4 and 5 when chemistry limits are exceeded.

The draft STS for BWRs require an engineering evaluation to be performed to verify the structural integrity of the reactor coolant system prior to plant restart in the event that chemistry parameters exceed the TS limits. However, the CPS TSs were formatted in such a manner that an engineering evaluation was only required if the chloride concentration exceeded prescribed limits. No actions are stated if conductivity or pH limits are exceeded during shutdown conditions.

The licensee's letter states that chemical decontamination of the Reactor Recirculation and Reactor Water Cleanup Systems is planned for the next refueling outage. This is currently scheduled for September 1993. Should the decontamination process result in conductivity or pH limits being exceeded for more than 72 hours during Operational Conditions 4 or 5, the current TSs could not be met.

The licensee's proposal is to make a formatting change so that the CPS TSs are similar to the draft STS. The change will require an engineering evaluation prior to plant restart if any of the chemistry limits of TS 3.4.4 are exceeded during plant shutdown for more than the prescribed time limits. The change does not alter the current chemistry limitations of the reactor coolant system nor does it alter the engineering evaluation that is to be performed.

The staff concurs that the intended action statement was to apply to all three chemistry parameters during shutdown conditions. Correction of this formatting error will introduce new conditions that will require an engineering evaluation and will provide appropriate action statements for the TSs. Therefore, based on our review, the staff finds the proposed change acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Illinois State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

This amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration and there has been no

public comment on such finding (58 FR 7000). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Douglas V. Pickett

Date: March 29, 1993