

October 4, 1990

Docket No. 50-461

Mr. Frank A. Spangenberg
Manager - Licensing and Safety
Clinton Power Station
P. O. Box 678, Mail Code V920
Clinton, Illinois 61727

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Dear Mr. Spangenberg:

SUBJECT: ISSUANCE OF AMENDMENT NO. 50 TO FACILITY OPERATING LICENSE NO. NPF-62 (TAC NO. 72100)

The Commission has issued the enclosed Amendment No. 50 to Facility Operating License No. NPF-62 for the Clinton Power Station, Unit No. 1. This amendment is in response to your application dated December 21, 1988.

This amendment revises Table 3.3.2-1 to more accurately reflect the channel/trip logic configuration of the containment exhaust radiation trip channels listed under Part 1, Item h.

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

original signed by

John B. Hickman, Project Manager
Project Directorate III-2
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 50 to License No. NPF-62
2. Safety Evaluation

cc w/enclosures:
See next page

DOCUMENT NAME: 72100 AMD

Office: LA/PDIII-3
Surname: PKreutzer
Date: 9/12/90

JBH
PM/PDIII-3
JHickman/tg
9/17/90

JH
PD/PDIII-3
JHannon
9/17/90

OGC
R. Bachmann
9/19/90

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Mr. Frank A. Spangenberg
Illinois Power Company

Clinton Power Station
Unit 1

cc: Mr. J. S. Perry
Vice President
Clinton Power Station
P. O. Box 678
Clinton, Illinois, 61727

Illinois Department
of Nuclear Safety
Office of Nuclear Facility Safety
1035 Outer Park Drive
Springfield, Illinois 62704

Mr. J. A. Miller
Manager Nuclear Station
Engineering Dept.
Clinton Power Station
P. O. Box 678
Clinton, Illinois 61727

Mr. Donald Schopfer
Project Manager
Sargent & Lundy Engineers
55 East Monroe Street
Chicago, Illinois 60603

Sheldon Zabel, Esquire
Schiff, Hardin & Waite
7200 Sears Tower
233 Wacker Drive
Chicago, Illinois 60606

Resident Inspector
U. S. Nuclear Regulatory Commission
RR#3, Box 229 A
Clinton Illinois 61727

Mr. L. Larson
Project Manager
General Electric Company
175 Curtner Avenue, N/C 395
San Jose, California 95125

Regional Administrator, Region III
799 Roosevelt Road, Bldg. #4
Glen Ellyn, Illinois 60137

Chairman of DeWitt County
c/o County Clerk's Office
DeWitt County Courthouse
Clinton, Illinois 61727

Robert Neumann
Office of Public Counsel
State of Illinois Center
100 W. Randolph
Suite 11-300
Chicago, Illinois 60601



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

ILLINOIS POWER COMPANY, ET AL.

DOCKET NO. 50-461

CLINTON POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 50
License No. NPF-62

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Illinois Power Company* (IP), and Soyland Power Cooperative, Inc. (the licensees) dated December 21, 1988, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-62 is hereby amended to read as follows:

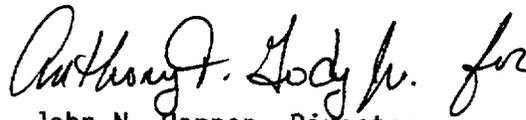
*Illinois Power Company is authorized to act as agent for Soyland Power Cooperative, Inc. and has exclusive responsibility and control over the physical construction, operation and maintenance of the facility.

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 50 , are hereby incorporated into this license. Illinois Power Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



John N. Hannon, Director
Project Directorate III-3
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: October 4, 1990

ATTACHMENT TO LICENSE AMENDMENT NO. 50

FACILITY OPERATING LICENSE NO. NPF-62

DOCKET NO. 50-461

Replace the following pages of the Appendix "A" Technical Specifications with the attached pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are provided to maintain document completeness.

Remove

3/4 3-13

3/4 3-18a

Insert

3/4 3-13

3/4 3-18a

TABLE 3.3.2-1
CRVICS INSTRUMENTATION

<u>TRIP FUNCTION</u>	<u>ISOLATION SIGNAL ††</u>	<u>MINIMUM OPERABLE CHANNELS PER TRIP SYSTEM</u>	<u>APPLICABLE OPERATIONAL CONDITION</u>	<u>ACTION</u>
1. <u>PRIMARY AND SECONDARY CONTAINMENT ISOLATION</u>				
a. Reactor Vessel Water Level-Low Low, Level 2	B ^{(b)(f)}	2 ^(a)	1, 2, 3 #	20 25
b. Reactor Vessel Water Level-Low Low, Level 2 (ECCS Div. I and II)	B	2 ^(a)	1, 2, 3	29
c. Reactor Vessel Water Level-Low, Low, Level 2 (HPCS-NSPS Div. III and IV)	B	2 ^{(a)(m)}	1, 2, 3	29
d. Drywell Pressure - High	L ^{(b)(f)}	2 ^(a)	1, 2, 3	20
e. Drywell Pressure - High (ECCS Div. I and II)	L	2 ^(a)	1, 2, 3	29
f. Drywell Pressure - High (HPCS-NSPS Div. III and IV)	L	2 ^{(a)(n)}	1, 2, 3	29
g. Containment Building Fuel Transfer Pool Ventilation Plenum Radiation - High	Z ^{(b)(f)}	2 ^(a)	##	25
h. Containment Building Exhaust Radiation - High				
1) Outboard (Div. I) Valve Isolation	M ^{(b)(f)}	2 ^{(a)(o)}	1, 2, 3 #	29 25
2) Inboard (Div. II) Valve Isolation	M ^{(b)(f)}	2 ^{(a)(o)}	1, 2, 3 #	29 25
i. Containment Building Continuous Containment Purge (CCP) Exhaust Radiation - High	5 ^{(b)(f)}	2 ^(a)	1, 2, 3 #	29 25
j. Reactor Vessel Water Level-Low Low Low, Level 1	U	2 ^(k)	1, 2, 3 #	29 25
k. Containment Pressure-High	P	1 ^{(k)(1)}	1, 2, 3 #	29 25

TABLE 3.3.2-1 (Continued)
CRVICS INSTRUMENTATION
TABLE NOTATIONS

- (m) Four reactor vessel water level trip channels are logically combined in a one-out-of-two-twice configuration. For the purposes of the associated ACTION, each one-out-of-two logic is defined as a separate trip system.
- (n) Four drywell pressure trip channels are logically combined in a one-out-of-two-twice configuration. For the purposes of the associated ACTION, each one-out-of-two logic is defined as a separate trip system.
- (o) One trip system is associated with the A and B monitors; the other trip system is associated with the C and D monitors.
- (p) Each channel consists of five temperature modules and their associated sensors. A channel is OPERABLE if and only if five temperature modules and their associated sensors are OPERABLE.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 50 TO FACILITY OPERATING LICENSE NO. NPF-62

ILLINOIS POWER COMPANY, ET AL.

CLINTON POWER STATION, UNIT NO. 1

DOCKET NO. 50-461

1.0 INTRODUCTION

By application for amendment dated December 21, 1989, Illinois Power Company requested a revision of the Technical Specifications to Facility Operating License No. NPF-62 for the Clinton Power Station. The proposed change revises Table 3.3.2-1 to more accurately reflect the channel/trip logic configuration of the containment exhaust radiation trip channels listed under Part 1, Item H. This evaluation encompassed the referenced changes as applicable to Section 3.3.2-1 of the Clinton Power Station technical specifications.

2.0 EVALUATION

The containment building exhaust radiation high trip logic for the inboard/outboard containment isolation valves is divided into two parts. One trip system closes the outboard isolation valves (Division 1) and the other trip system closes the inboard isolation valves (Division 2). The inboard/outboard containment isolation valves close upon receiving a LOCA signal (high drywell pressure or low reactor water level) using a one-out-of-two taken twice trip logic as referenced in the licensee submittal. In addition, designated inboard/outboard isolation valves may close upon receipt of a containment building exhaust radiation high signal. Table 3.3.2-1 treats the inboard and outboard containment high radiation isolation valve logics as one trip system with a one-out-of-two logic arrangement. This results in Table 3.3.2-1 requiring two operable channels per trip system. As shown in the licensee submittal, the trip logic for the inboard/outboard isolation valves consists of four radiation monitors (two monitors per division) arranged such that the channels of Division 1 are combined with those of Division 2 (through isolators) to form a one-out-of-two taken twice trip logic for both divisions. If the trip system for containment isolation is considered to include both Division 1 and Division 2 for each isolation logic then the TS operable channels per trip system may be considered unclear for Item h (1) of Table 3.3.2-1.

To resolve this apparent discrepancy the licensee proposed to revise Table 3.3.2-1 to reflect the divisional (outboard/inboard) nature of the containment high radiation exhaust radiation logic. Item h (1) will now list Division 1 (outboard) and Division 2 (inboard) isolation trip functions separately with

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minimum operable channels per trip system designated for each. To clarify the one-out-of-two taken twice logic configuration for each division, a note "o" will be added to define the trip system boundary for the containment building high radiation trip logic. This results in two channels per trip system and two trip systems per trip function. The above revisions are intended to ensure that the requirements of TS 3.3.2 (LCO) are followed when a containment exhaust radiation channel is declared inoperable. The proposed amendment as submitted by the licensee is consistent with the STS requirements for operable channels per trip system and does not alter the exhaust radiation high trip logic to the Containment and Reactor Vessel Isolation Control System (CRVICS). Based on the above evaluation, the proposed change to TS Table 3.3.2-1 Item h (1) is acceptable to the staff.

The proposed revision to the Clinton Power Station technical specifications Table 3.3.2-1, Part 1, Item h is in agreement with the logic shown on attachment 2 of the licensee submittal. The proposed revision resolves an apparent discrepancy between Table 3.3.2-1 Item h (1) and the logic configuration of the containment building high radiation monitoring system. Based on the above evaluation the staff finds the proposed amendment acceptable.

3.0 ENVIRONMENTAL CONSIDERATION

This amendment involves a change to a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 or a change to a surveillance requirement. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

4.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner; and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Cliff Douth, NRR

Dated: October 4, 1990