

March 21, 1988

Docket No. 50-461

Mr. Frank Spangenberg  
Manager - Licensing and Safety  
Clinton Power Station  
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Dear Mr. Spangenberg:

SUBJECT: TECHNICAL SPECIFICATION CHANGE REQUEST TO ADD TEST CONNECTIONS  
UPSTREAM OF CERTAIN EXCESS FLOW CHECK VALVES (TAC NO. 66553)

Re: Clinton Power Station, Unit 1

The Commission has issued the enclosed Amendment No. 2 to the Facility Operating License No. NPF-62 for the Clinton Power Station, Unit No. 1. This amendment consists of changes to the Technical Specifications (TSs) in response to your application dated October 30, 1987.

This amendment revises Technical Specification Section 3.6.4 and Table 3.6.4-1 in order to add test connections upstream of certain excess flow check valves (1CM002B, 1SM008, 1SM011, 1E22-F332, and 1E51-F377B). This plant modification will facilitate the testing of these valves as required by Technical Specification 4.6.4.4.

A copy of our Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

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Janice A. Stevens, Project Manager  
Project Directorate III-2  
Division of Reactor Projects - III,  
IV, V and Special Projects

Enclosures:

1. Amendment No. 2 to License No. NPF-62
2. Safety Evaluation

cc:  
See next page

PDIII-2:PM *[Signature]* PDIII-2:LA *[Signature]*  
 JStevens:bj LLuther JWCraig SPLB *[Signature]* OGC-Rockville *[Signature]* PDIII-2:PD  
 3/14/88 3/14/88 3/14/88 3/16/88 3/14/88  
*No legal objection, subject to FR notice being provided for my review.*

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Mr. Frank A. Spangenberg, III  
Illinois Power Company

Clinton Power Station  
Unit 1

cc:

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ILLINOIS POWER COMPANY, ET AL

DOCKET NO. 50-461

CLINTON POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 2  
License No. NPF-62

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by the Illinois Power Company\* (IP), Soyland Power Cooperative, Inc. and Western Illinois Power Cooperative, Inc. (the licensees) dated October 30, 1987 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-62 is hereby amended to read as follows:

Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised

\*Illinois Power Company is authorized to act as agent for Soyland Power Cooperative, Inc. and Western Illinois Power Cooperative, Inc. and has exclusive responsibility and control over the physical construction, operation and maintenance of the facility.

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through Amendment No. 2, are hereby incorporated into this license. IP shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



for  
Daniel R. Muller, Director  
Project Directorate III-2  
Division of Reactor Projects - III,  
IV, V and Special Projects

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: March 21, 1988

ATTACHMENT TO LICENSE AMENDMENT NO. 2

FACILITY OPERATING LICENSE NO. NPF-62

DOCKET NO. 50-461

Replace the following pages of the Appendix "A" Technical Specifications with the attached pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. Overleaf page(s) are provided to maintain document completeness.

Remove

3/4 6-29

3/4 6-51

3/4 6-52

3/4 6-61

Insert

3/4 6-29

3/4 6-51

3/4 6-52

3/4 6-61

## CONTAINMENT SYSTEMS

### 3/4.6.4 CONTAINMENT ISOLATION VALVES

#### LIMITING CONDITION FOR OPERATION

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3.6.4 The containment isolation valves and the instrumentation line excess flow check valves shown in Table 3.6.4-1 shall be OPERABLE<sup>#</sup> with isolation times less than or equal to those shown in Table 3.6.4-1.

APPLICABILITY: As shown in Table 3.6.4-1.

#### ACTION:

- a. With one or more of the containment isolation valves shown in Table 3.6.4-1 inoperable, maintain at least one isolation valve OPERABLE in each affected penetration that is open and within 4 hours either:
1. Restore the inoperable valve(s) to OPERABLE status, or
  2. Isolate each affected penetration by use of at least one deactivated automatic valve secured in the isolated position,\*† or
  3. Isolate each affected penetration by use of at least one closed manual valve or blind flange.\*†

The provisions of Specification 3.0.4 are not applicable provided the affected penetration is isolated in accordance with ACTION a.2 or a.3 above, and provided the associated system, if applicable, is declared inoperable or appropriate ACTION statements for that system are performed.

Otherwise be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.

Otherwise, in OPERATIONAL CONDITION \*\*, suspend all operations involving CORE ALTERATIONS, handling irradiated fuel in the secondary containment, or with a potential for draining the reactor vessel. The provisions of Specification 3.0.3 are not applicable.

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\*Isolation valves closed to satisfy these requirements may be reopened on an intermittent basis under administrative controls.

\*\*When handling irradiated fuel in the secondary containment and during CORE ALTERATIONS and operations with a potential for draining the reactor vessel.

†Containment Isolation Valves can have dual functions in that they provide both containment isolation and Emergency Core Cooling functions. Any inoperable dual function valve could degrade the valves' other function.

#See Note (h) in Table of Notations for Table 3.6.4-1.

TABLE 3.6.4-1 (Continued)  
CONTAINMENT ISOLATION VALVES

<u>VALVE NUMBER</u>	<u>PENETRATION NUMBER</u>	<u>ISOLATION SIGNAL†</u>	<u>APPLICABLE OPERATIONAL CONDITIONS</u>	<u>MAXIMUM ISOLATION TIME (Seconds)</u>	<u>SECONDARY CONTAINMENT BYPASS PATH (YES/NO)</u>	<u>TEST PRESSURE (psig)*</u>
<u>Test Connections, Vents and Drains</u> <sup>(a)</sup> (Continued)						
64) Drywell Pressure 1CM076 1CM077	151 203	NA	1, 2, 3(a)	NA	No	9.0
65) Reactor Pressure 1CM072 1CM073	151	NA	1, 2, 3(a)	NA	No	9.0
66) Reactor Pressure 1CM074 1CM075	160	NA	1, 2, 3(a)	NA	No	9.0
67) Equipment Hatch 1CM099	1	NA	1, 2, 3(a)	NA	No	9.0
68) Suppression Pool Level 1E51 - F437A <sup>(h)</sup> 1E51 - F437B <sup>(h)</sup>	177	NA	1, 2, 3(a)	NA	No	9.0
69) Suppression Pool Level 1E22 - F381A <sup>(h)</sup> 1E22 - F381B <sup>(h)</sup> 1SM027A <sup>(h)</sup> 1SM027B <sup>(h)</sup>	179	NA	1, 2, 3(a)	NA	No	9.0
70) Suppression Pool Level 1SM026A <sup>(h)</sup> 1SM027B <sup>(h)</sup>	181	NA	1, 2, 3(a)	NA	No	9.0
71) Suppression Pool Level 1CM100A <sup>(h)</sup> 1CM100B <sup>(h)</sup>	183	NA	1, 2, 3(a)	NA	No.	9.0

TABLE 3.6.4-1 (Continued)  
CONTAINMENT ISOLATION VALVES

<u>VALVE NUMBER</u>	<u>PENETRATION NUMBER</u>	<u>ISOLATION SIGNAL†</u>	<u>APPLICABLE OPERATIONAL CONDITIONS</u>	<u>MAXIMUM ISOLATION TIME (Seconds)</u>	<u>SECONDARY CONTAINMENT BYPASS PATH (YES/NO)</u>	<u>TEST PRESSURE (psig)*</u>
<u>Other Isolation Valves (Continued)</u>						
4. <u>Other Isolation Valves</u>						
1) Main Steam Line C 1E32-F001J	5	NA	1, 2, 3	NA	No	9.0
2) Main Steam Line A 1E32-F001A	6	NA	1, 2, 3	NA	No	9.0
3) Main Steam Line D 1E32-F001N	7	NA	1, 2, 3	NA	No	9.0
4) Main Steam Line B 1E32-F001E	8	NA	1, 2, 3	NA	No	9.0
5) Feedwater/RHR Line A 1B21-F010A 1B21-F065A	9	NA	1, 2, 3	NA	Yes	9.0
6) Feedwater/RHR Line B 1B21-F010B 1B21-F065B	10	NA	1, 2, 3	NA	Yes	9.0
7) RHR A Suction Line 1E12-F004A <sup>(e)</sup>	11	NA	1, 2, 3	NA	No	9.9
8) RHR B Suction Line 1E12-F004B <sup>(e)</sup>	12	NA	1, 2, 3	NA	No	9.9

TABLE 3.6.4-1 (Continued)  
CONTAINMENT ISOLATION VALVES

TABLE NOTATIONS

- (a) May be opened on an intermittent basis under administrative control.
- (b) Excess flow check valve actuation differential pressure.
- (c) Isolation valving for instrument lines which penetrate the containment conform to the requirements of NRC Regulatory Guide 1.11. The in-service inspection program will provide assurance of the operability and integrity of these isolation provisions. Type "C" testing will not be performed on the instrument line isolation valves. The instrument lines will be within the boundaries of the Type "A" test, open to the media (containment atmosphere or suppression pool water) to which they will be exposed under postulated accident conditions. Instrument taps from the process line located between the process isolation valves and the penetration, and not themselves penetrating containment, will be Type "A" and/or "C" tested along with the process line isolation valves.
- (d) Excess flow check valve.
- (e) The RHR system may be operating in the shutdown cooling mode during the Type A test. These valves are tested using water but the results are not required to be added to the Type A test results. The LPCS, HPCS, and RHR may be aligned in the normal standby or injection mode during the Type A test. This will expose the closed loop outside containment to containment pressure through the suppression pool. This is the closest valve alignment to the post-LOCA alignment possible. Type C water test results on these suction valves will not be added to the Type A test results.
- (f) Valves shall be closed in accordance with SECONDARY CONTAINMENT INTEGRITY.
- (g) Valves shall be "sealed closed" by utilizing mechanical devices to seal or lock the valve closed or to prevent power from being supplied to the valve operator.
- (h) OPERABILITY of these valves is not required until completion of corresponding plant modification.
- # When handling irradiated fuel in secondary containment and during CORE ALTERATIONS and operations with a potential for draining the reactor vessel.
- ## Isolates on RCIC low steam line pressure only.
- † Isolation signal descriptions are provided in Table 3.6.4-2.
- \* For test pressure = 9.0 psig, the valve(s) shall be pressurized using air or nitrogen, and for test pressure = 9.9 psig, the valve(s) shall be pressurized using water.
- \*\* With any control rod withdrawn. Not applicable to any control rods removed per Specification 3.9.10.1 or 3.9.10.2.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 2 TO FACILITY OPERATING LICENSE NO. NPF-62

ILLINOIS POWER COMPANY, ET AL

CLINTON POWER STATION, UNIT 1

DOCKET NO. 50-461

1.0 INTRODUCTION

By letter dated October 30, 1987 the Illinois Power Company (IP), et al (the licensees) requested an amendment to Facility Operating License No. NPF-62 for the Clinton Power Station, Unit 1. The proposed amendment would allow for a plant modification to add test connections upstream of certain excess flow check valves in order to facilitate the testing of these valves as required by Technical Specification 4.6.4.4. The new test connections to be added would contain isolation valves which must be added to the Test Connections, Vents and Drains section of Table 3.6.4-1. In order to ensure compliance with the Technical Specifications, and yet provide some flexibility in the schedule for completing the modification, IP has proposed that footnotes be added to pages 3/4 6-29 and 3/4 6-61 to allow the proposed Technical Specification changes to become effective once the modification is complete. At that time, the OPERABILITY requirements and provisions of the Technical Specifications would be in effect for the subject valves.

2.0 EVALUATION

Technical Specification 4.6.4.4 requires that each instrument line excess flow check valve shown in Table 3.6.4-1 shall be demonstrated OPERABLE at least once per 18 months by verifying that the valve actuates within

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the differential pressure range provided. The licensees have proposed a plant modification to add test connections upstream of certain excess flow check valves (1CM002B, 1SM008, 1SM011, 1E22-F332, and 1E51-F377B) in order to facilitate the required testing of these valves. These tests are required to verify the capability of the excess flow check valves to check the flow at a particular differential pressure assumed for accident conditions.

These added test connections are double-valved, capped, and meet the applicable design/safety requirements to ensure containment integrity. The addition of these test connections does not affect the functional characteristics of the excess flow check valves and therefore does not affect operation of the associated systems. Since these new test connections contain isolation valves, these valves must be added to the Test Connections, Vents and Drains section of Table 3.6.4-1 in the Technical Specifications. Adding the valves to Table 3.6.4-1 also makes all of the applicable Technical Specification requirements associated with containment integrity applicable to the new valves. Thus, the proposed change does not involve a significant increase in the probability or consequences of an accident previously evaluated.

It has been determined that the proposed change does not create the possibility of a new or different kind of accident from those previously evaluated. It has also been determined that the proposed change does not involve a significant reduction in a margin of safety since it does not involve a change to any trip setpoints, analytical valves, or design limits required or assumed in any safety analysis.

### 3.0 ENVIRONMENTAL CONSIDERATION

This amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. We have determined that the amendment involves no significant increase in the amounts, and no significant changes in the types, of any effluents

that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding.

Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

#### 4.0 CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Janice A. Stevens, NRR/PDIII-2

Dated: March 21, 1988